



Entergy Nuclear Operations, Inc.
Pilgrim Nuclear Power Station
600 Rocky Hill Road
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Charles M. Dugger
Vice President - Operations

February 14, 2003

U.S. Nuclear Regulatory Commission
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SUBJECT: Entergy Nuclear Operations, Inc.
Pilgrim Nuclear Power Station
Docket No. 50-293
License No. DPR-35

Request for Amendment to the Technical Specifications
Changes to Post-Accident Monitoring Instrumentation Requirements

REFERENCE: 1) Entergy Letter 2.02.065, R.M. Bellamy to NRC, "Request for
Amendment to the Technical Specifications Relocation of Certain
Control Rod Block Requirements to the Updated Final Safety Analysis
Report," dated August 16, 2002
2) Entergy Letter 2.02.072, R.M. Bellamy to NRC, "Request for
Amendment to the Technical Specifications Changes to Post-Accident
Monitoring Instrumentation Requirements," dated August 19, 2002

LETTER NUMBER: 2.03.019

Dear Sir or Madam:

In accordance with the provisions of 10 CFR 50.90, Entergy Nuclear Operations, Inc. (Entergy) is submitting a supplementary change to the Reference 2 request for an amendment to the Technical Specifications (TS) for Pilgrim Nuclear Power Station. This supplementary information restores the portion of Note 2 on TS page 3/4.2-41 associated with Tables 4.2.D and 4.2.G.

This proposed change does not impact the conclusions of the previously submitted no significant hazards conclusions. Attachment 1 provides the revised proposed mark-up of the affected page.

The mark-up of the TS submitted with this supplementary change restores Note 2 on TS page 3/4.2-41, with the exception of one sentence that was deleted in the original submittal and which is still no longer needed. Deletion of the marked-up sentence from Note 2 is dependant upon the prior approval of the proposed TS change concerning the relocation of certain Control Rod Block functions submitted via Reference 1. If the changes requested in Reference 1 are not approved prior to the approval of this proposed change, then the marked-up portion of Note 2 should not be deleted as part of this change.

If you have any questions, please contact Mr. Bryan Ford at (508) 830-8403.

I declare under penalty of perjury that the foregoing is true and correct. Executed on the
14th day of February 2003.

Sincerely,



Charles M. Dugger

Attachment: 1. Marked-up Proposed Technical Specification page (1 page)

cc:

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ATTACHMENT 1

PROPOSED TECHNICAL SPECIFICATION

CHANGE (MARK-UP)

NOTES FOR TABLES 4.2.A THROUGH 4.2.G

1. Initially once per month until exposure hours (M as defined on Figure 4.1.1) is 2.0×10^5 ; thereafter, according to Figure 4.1.1 with an interval not less than one month nor more than three months.
2. Functional tests, calibrations and instrument checks are not required when these instruments are not required to be operable or are tripped. Functional tests shall be performed before each startup with a required frequency not to exceed once per week. ~~Calibrations of IRMs and SRMs shall be performed during each startup or during controlled shutdowns with a required frequency not to exceed once per week.~~ Instrument checks shall be performed at least once per day during those periods when the instruments are required to be operable.
3. This instrumentation is excepted from the functional test definition. The functional test will consist of injecting a simulated electrical signal into the measurement channel.
4. Simulated automatic actuation shall be performed once each operating cycle. Where possible, all logic system functional tests will be performed using the test jacks.
5. Reactor low water level and high drywell pressure are not included on Table 4.2.A since they are tested on Tables 4.1.1 and 4.1.2.
6. The logic system functional tests shall include a calibration of time delay relays and timers necessary for proper functioning of the trip systems.
7. Calibration of analog trip units will be performed concurrent with functional testing. The functional test will consist of injecting a simulated electrical signal into the measurement channel. Calibration of associated analog transmitters will be performed each refueling outage.