

April 1, 2003

Mr. John L. Skolds, President
Exelon Nuclear
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2 - ISSUANCE OF
AMENDMENTS RE: ATWS-RPT INSTRUMENTATION SURVEILLANCE
REQUIREMENT (TAC NOS. MB6136 AND MB6137)

Dear Mr. Skolds:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 213 to Facility Operating License No. DPR-29 and Amendment No. 207 to Facility Operating License No. DPR-30 for the Quad Cities Nuclear Power Station, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application dated August 22, 2002.

The amendments modify the required surveillance interval from monthly to quarterly for calibration of the trip units associated with the instrumentation channels of the Anticipated Transient Without Scram-Recirculation Pump Trip (ATWS-RPT) system.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Carl F. Lyon, Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-254 and 50-265

Enclosures: 1. Amendment No. 213 to DPR-29
2. Amendment No. 207 to DPR-30
3. Safety Evaluation

cc w/encls: See next page

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Sincerely,
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Carl F. Lyon, Project Manager, Section 2
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cc w/encls: See next page

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EXELON GENERATION COMPANY, LLC

AND

MIDAMERICAN ENERGY COMPANY

DOCKET NO. 50-254

QUAD CITIES NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 213

License No. DPR-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated August 22, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-29 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 213, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Anthony J. Mendiola, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: April 1, 2003

EXELON GENERATION COMPANY, LLC

AND

MIDAMERICAN ENERGY COMPANY

DOCKET NO. 50-265

QUAD CITIES NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 207
License No. DPR-30

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated August 22, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-30 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 207, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Anthony J. Mendiola, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: April 1, 2003

ATTACHMENT TO LICENSE AMENDMENT NOS. 213 AND 207

FACILITY OPERATING LICENSE NOS. DPR-29 AND DPR-30

DOCKET NOS. 50-254 AND 50-265

Replace the following page of the Appendix "A" Technical Specifications with the attached page. The revised page is identified by number and contains marginal lines indicating the area of change.

Remove Page

3.3.4.1-3

Insert Page

3.3.4.1-3

The following revised TS Bases page is provided for information only:

B.3.3.4.1-9

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 213 TO FACILITY OPERATING LICENSE NO. DPR-29
AND AMENDMENT NO. 207 TO FACILITY OPERATING LICENSE NO. DPR-30
EXELON GENERATION COMPANY, LLC
AND
MIDAMERICAN ENERGY COMPANY
QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2
DOCKET NOS. 50-254 AND 50-265

1.0 INTRODUCTION

By application to the U. S. Nuclear Regulatory Commission (NRC, the staff) dated August 22, 2002, Exelon Generation Company, LLC (the licensee) requested changes to the Technical Specifications (TSs) for the Quad Cities Nuclear Power Station, Units 1 and 2 (QCNPS). The staff's proposed no significant hazards consideration determination was published in the *Federal Register* on October 1, 2002 (67 FR 61682).

The proposed changes would modify the required interval from 31 days to 92 days for performance of Surveillance Requirement (SR) 3.3.4.1.2, calibration of the trip units associated with the instrumentation channels of the Anticipated Transient Without Scram-Recirculation Pump Trip (ATWS-RPT) system. The licensee states that approval of the proposed changes would reduce unnecessary wear on components and unavailability of equipment during surveillance testing, and allow better allocation of plant resources.

2.0 REGULATORY EVALUATION

The staff finds that the licensee in sections B and C of Attachment A of its submittal identified the applicable regulatory requirements. TS Section 3.3.4.1 provides the operability requirements applicable to the protection and monitoring functions of the ATWS-RPT instrumentation. As part of the operability requirements, SR 3.3.4.1.2 specifies performance of trip unit calibrations at a specified interval on the electric analog trip units associated with the ATWS-RPT reactor water level and reactor vessel steam dome pressure instrumentation channels. Performance of SR 3.3.4.1.2 ensures a high degree of safety system reliability.

The ATWS Rule, 10 CFR 50.62, provides requirements for reduction of risk from ATWS events. 10 CFR 50.62(c)(5) requires that each boiling water reactor must have equipment to trip the reactor coolant recirculating pumps automatically under conditions indicative of an ATWS, and

requires that this equipment be designed to perform its function in a reliable manner. The protection and monitoring functions of the ATWS-RPT instrumentation have been designed to ensure safe operation of the reactor by lessening the effects of an ATWS event. The ATWS-RPT instrumentation initiates an RPT to insert negative reactivity into the reactor whenever reactor water level and/or reactor vessel steam dome pressure exceed their specified limits. RPT actuation aids in preserving the integrity of the fuel cladding following events in which a Reactor Protection System scram should occur, but does not.

The regulatory requirement for which the staff based its acceptance is 10 CFR 50.62(c)(5).

3.0 TECHNICAL EVALUATION

3.1 Background

By application dated December 27, 1999, the licensee proposed to extend the surveillance test interval (STI) for selected instruments from monthly to quarterly. These instruments included the analog trip units, primarily for instruments in the reactor protection system and the emergency core cooling system (ECCS). However, the licensee intentionally retained the monthly STI for the ATWS-RPT analog trip units. The NRC approved the licensee's proposal in Amendment Nos. 198/194, dated March 28, 2001. Since that time, the licensee has performed setpoint calculations that support a quarterly STI for the ATWS-RPT reactor vessel level and pressure instrument trip units using NRC-approved calculation methodology.

3.2 Evaluation

The licensee proposes to change the frequency of SR 3.3.4.1.2, "Calibrate the trip units," from 31 to 92 days. General Electric Licensing Topical Report (LTR) GENE-770-06-1-A, "Bases for Changes to Surveillance Test Intervals and Allowed Out-of-Service Times for Selected Instrumentation Technical Specifications," dated December 1992, provides justification for extending the calibration interval for ATWS-RPT trip units from monthly to quarterly. The NRC approved the LTR by letter dated July 21, 1992, which specified that for plant-specific application of the LTR, the licensee must confirm that (1) the generic analysis applies to the plant, and (2) the setpoint drift expected during the extended test intervals is within the existing allowances in the respective setpoint calculations.

The licensee verified in Attachment A of the August 22, 2002, application that GENE-770-06-1-A is applicable to QCNPS. The licensee states that GENE-770-06-1-A is applicable to QCNPS because the LTR analysis is based on one-out-of-two-taken-twice and two-out-of-two-taken-once logics, while QCNPS uses a two-out-of-two-taken-once logic. Therefore, the LTR analysis encompasses the ATWS-RPT design at QCNPS.

The licensee further stated that it has explicitly evaluated the setpoint drift associated with the trip units at QCNPS, demonstrating that extending the current calibration interval from 31 days to 92 days is acceptable and that the drifts are within the existing setpoint allowances. This rationale is consistent with the evaluation of analog trip units in General Electric LTR NEDC-30851P-A, "Technical Specification Improvement Analyses for BWR Reactor Protection System," dated March 1988. The licensee verified in Attachment E of their December 27, 1999, application that the generic analysis of NEDC-30851P-A applied to QCNPS. The licensee was

a participating utility in the development of this LTR. Section 5.7.3 of the LTR states that, "Current vendor drift information on analog trip units indicates that the calibration intervals could be extended to six months." Furthermore, the test interval of 92 days is consistent with the test interval for similar existing trip units in the reactor protection system and ECCS.

The staff has reviewed the licensee's regulatory and technical analyses in support of its proposed license amendment, as described in Attachment A of the licensee's submittal. On the basis of the above evaluation, the staff concludes that the licensee's proposed TS change is acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change an inspection or surveillance requirement. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (67 FR 61682). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Mazumdar

Date: April 1, 2003