

February 28, 2003

Mr. Craig G. Anderson
Vice President, Operations ANO
Entergy Operations, Inc.
1448 S. R. 333
Russellville, AR 72801

SUBJECT: ARKANSAS NUCLEAR ONE, UNITS 1 AND 2 - NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES TO ALLOW THE USE OF THE SPENT FUEL CRANE TO LIFT HEAVY LOADS IN EXCESS OF 100 TONS (TAC NOS. MB7799 AND MB7800)

Dear Mr. Anderson:

The Commission has forwarded the enclosed "Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing," to the Office of the Federal Register for publication.

This notice relates to your February 24, 2003, application to allow use of the spent fuel crane (L-3 crane) to lift heavy loads in excess of 100 tons.

Sincerely,

/RA/

Thomas W. Alexion, Project Manager, Section 1
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-313 and 50-368

Enclosure: Notice

cc w/encl: See next page

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NRR-058

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UNITED STATES NUCLEAR REGULATORY COMMISSION

ENTERGY OPERATIONS, INC.

DOCKET NOS. 50-313 AND 50-368

NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO
FACILITY OPERATING LICENSE, PROPOSED NO SIGNIFICANT HAZARDS
CONSIDERATION DETERMINATION, AND OPPORTUNITY FOR A HEARING

The U.S. Nuclear Regulatory Commission (NRC or the Commission) is considering issuance of amendments to Renewed Facility Operating License (FOL) No. DPR-51 and FOL No. NPF-6, issued to Entergy Operations, Inc. (the licensee), for operation of Arkansas Nuclear One (ANO), Units 1 and 2 (ANO-1 and ANO-2), respectively, located in Pope County, Arkansas.

The proposed amendments would allow the licensee to use the spent fuel crane (L-3 crane) to lift heavy loads in excess of 100 tons. Specifically the licensee is requesting approval to use the upgraded L-3 crane for loads up to a total of 130 tons.

The amendment application was submitted on an exigent basis because the need for a license amendment was identified as a result of recent discussions between the licensee and NRC staff. The licensee had previously believed that prior NRC approval was not required to use the upgraded L-3 crane for heavy loads in excess of 100 tons. Approval to use the upgraded L-3 crane on an exigent basis is necessary for several reasons, including: (1) numerous activities associated with loading and un-loading the cask are required to be demonstrated by the user prior to the first usage with spent fuel, in accordance with the certificate of compliance for the new spent fuel storage cask system; (2) prior to the

certificate-required demonstrations, detailed checkout of the equipment and sufficient training, including on-the-job use of the equipment, must occur to provide assurance of craft and supervisory proficiency; (3) there is insufficient space in the ANO-2 spent fuel pool and dry storage racks to store all of the fuel required for the fall 2003 ANO-2 refueling outage, unless at least one cask is loaded; (4) another cask needs to be loaded prior to the refueling outage to avoid having to perform an in-core shuffle of control element assemblies; and (5) the loading of one more cask (total of three) prior to the fall refueling outage, combined with storage spaces recovered as a result of installation of the new neutron poison panels, will ensure capability of full core discharge to the spent fuel pool following the refueling outage. The licensee provided a detailed timetable of the above activities which demonstrates over the next seven months the complexity involved with managing the spent fuel pool inventories. In addition, the licensee believes that the need to optimize pool storage space, the increased impact on the ANO-2 spent fuel pool activity management, and the possible constraints described above, creates a significant plant cost and fuel control concern. Therefore, the licensee has requested the proposed amendment be issued by March 31, 2003.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

Pursuant to 10 CFR 50.91(a)(6) for amendments to be granted under exigent circumstances, the NRC staff must determine that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by

10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No.

The potential load carrying capability of the new L-3 crane has been increased from 100 tons to 130 tons. The transporting of a spent fuel cask is the maximum load that the crane is designed to handle. The process for transporting of a cask is essentially unchanged from that previously performed. Once a cask is loaded with spent fuel it is lifted from the cask loading pit, transported to the hatch, and lowered to the railroad bay. This arrangement is such that the cask is never carried over the spent fuel pool. The transport height of the cask has been increased to a minimum of 1.5 feet and the impact limiters used under the previous cask transport process have been eliminated. Because the crane is single failure proof, a postulated cask drop is no longer a credible event; therefore, no [a]effects on plant operation are anticipated to occur and the structural integrity of the spent fuel cask will not be impaired.

The probability of a load drop is reduced from that previously analyzed since the crane is single failure proof and the likelihood of a drop is no longer considered credible. If a portion of the L-3 lifting devices malfunction or fail, the crane system is designed such that the load will move a limited distance downward prior to backup restraints becoming engaged. An increased minimum transport height (1.5 feet) is established to accommodate this design feature. [A single malfunction or failure of a portion of the crane will prevent the load from being dropped. This will allow additional restrictions such as impact limiters to be removed. The radiological consequences will not be increased.] The impact on the spent fuel contained in the cask has been analyzed under an assumed dropped cask event and has been determined to be within design basis limits. Heavy loads are restricted from being moved over the spent fuel pools in accordance with ANO technical specifications.

Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No.

The ANO Safety Analysis Reports (SARs) have previously analyzed the drop of a cask up to 100 tons. This was as a result of a potential spent fuel cask drop event. The cask load has been increased to 130 tons under the new single failure proof L-3 crane design for heavier casks being employed at ANO. This

increased load could provide a more severe impact on safety related equipment that exists in areas below the load path if a load drop event were to occur. However, to ensure that no safety related equipment or control rooms are impacted, the construction of a single failure proof crane mitigates the potential for a more severe consequence to that already analyzed in the ANO SARs, since a load drop event is not considered credible.

Therefore, the proposed change does not create the possibility of a new or different kind of accident from any previously evaluated.

3. Does the proposed change involve a significant reduction in a margin of safety?

Response: No.

The L-3 crane has been upgraded to comply with the single failure proof requirements of NUREG-0554, *Single Failure Proof Cranes for Nuclear Power Plants* and Revision 3 of NRC approved Ederer Topical Report EDR-1 dated October 8, 1982. To comply with the requirements of the topical report the crane was modified to provide additional load carrying capability and additional safety features to prevent a cask drop event. The safety margins provided by the new crane design have either remained the same or increased to ensure adequate safety margin to prevent failure of the crane or any lifting devices associated with the lifting of a spent fuel cask.

Therefore, the proposed change does not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 14 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue amendments until the expiration of the 14-day notice period. However, should circumstances change during the notice period, such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendments before the expiration of the 14-day notice

period, provided that its final determination is that the amendments involve no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the Federal Register a notice of issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Chief, Rules and Directives Branch, Division of Administrative Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and should cite the publication date and page number of this Federal Register notice. Written comments may also be delivered to Room 6D59, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Documents may be examined, and/or copied for a fee, at the NRC's Public Document Room (PDR), located at One White Flint North, Public File Area O1 F21, 11555 Rockville Pike, Rockville, Maryland.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By April 7, 2003, the licensee may file a request for a hearing with respect to issuance of the amendments to the subject FOLs and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714,¹ which is available at the Commission's PDR, located at One White Flint North, Public File Area O1 F21, 11555 Rockville Pike, Rockville, Maryland,

¹ The most recent version of Title 10 of the *Code of Federal Regulations*, published January 1, 2002, inadvertently omitted the last sentence of 10 CFR 2.714(d) and subparagraphs (d)(1) and (2), regarding petitions to intervene and contentions. For the complete, corrected text of 10 CFR 2.714(d), please see 67 FR 20885 published April 29, 2002."

and available electronically on the Internet at the NRC Web site <http://www.nrc.gov/reading-rm/doc-collections/cfr/>. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on

which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If the amendments are issued before the expiration of the 30-day hearing period, the Commission will make a final determination on the issue of no significant hazards consideration. If a hearing is requested, the final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendments and make them immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendments.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC

20555-0001, Attention: Rulemakings and Adjudications Staff, or may be delivered to the Commission's PDR, located at One White Flint North, Public File Area O1 F21, 11555 Rockville Pike, Rockville, Maryland, by the above date. Because of continuing disruptions in delivery of mail to United States Government offices, it is requested that petitions for leave to intervene and requests for hearing be transmitted to the Secretary of the Commission either by means of facsimile transmission to 301-415-1101 or by e-mail to hearingdocket@nrc.gov. A copy of the request for hearing and petition for leave to intervene should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and because of continuing disruptions in delivery of mail to United States Government offices, it is requested that copies be transmitted either by means of facsimile transmission to 301-415-3725 or by e-mail to OGCMailCenter@nrc.gov. A copy of the request for hearing and petition for leave to intervene should also be sent to Nicholas S. Reynolds, Esquire, Winston and Strawn, 1400 L Street N.W., Washington, DC 20005-3502, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment dated February 24, 2003, which is available for public inspection at the Commission's PDR, located at One White Flint North, Public File Area O1 F21, 11555 Rockville Pike, Rockville, Maryland. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management System's (ADAMS) Public Electronic Reading Room on the Internet at the NRC web site <http://www.nrc.gov/reading-rm/adams.html>. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS,

should contact the NRC PDR Reference staff by telephone at 1-800-397-4209, 301-415-4737,
or by e-mail to pdr@nrc.gov.

Dated at Rockville, Maryland, this 28th day of February, 2003.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Thomas W. Alexion, Project Manager, Section 1
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Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Arkansas Nuclear One

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March 2001