

February 19, 2003

10 CFR 50.73

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555-0001

PALISADES NUCLEAR PLANT DOCKET 50-255 LICENSE NO. DPR-20 LICENSEE EVENT REPORT 02-003, INOPERABLE CONTAINMENT HYDROGEN MONITORS

Licensee Event Report (LER) 02-003 is attached. The LER describes the discovery of containment hydrogen monitors being inoperable for a period of time in excess of the Technical Specification allowed outage time. This occurrence is reportable in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by Technical Specifications.

SUMMARY OF COMMITMENTS

This letter contains no new commitments and no revisions to existing commitments.

Douglas E. Cooper Site Vice-President, Palisades

CC Regional Administrator, USNRC, Region III Project Manager, USNRC, NRR NRC Resident Inspector, Palisades

Attachment



NRC FORM 366 U.S. NUCLEAR REGULATORY (7-2001) COMMISSION						APPROVED BY OMB NO. 31500104 EXPIRES 7-31-2004 Estimated burden per response to comply with this mandatory information collection request 50 hours Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records Management Branch (T-6 E6), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001. or by intermete-mail to bis1@nrc.gov. and to											
LICENSEE EVENT REPORT (LER) (See reverse for reguired number of digits/characters for each block)							the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202 (3150-0104), Office of Managementand Budget, Washington, DC 20503 If a means used to impose information collection does not display a currently valid OMBcontrol number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.										
1. FACILITY NAME							2. DOCKET NUMBER 3. PAGE										
PALISADES NUCLEAR PLANT							05000255 1 OF 4						4				
4. TITLE																	
INOPERABLE CONTAINMENT HYDROGEN MONITOF							۲S										
5. EVE	NT DATE		6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED								
мо	DAY	YEAR	YEAR	YEAR SEQUENTIAL REV			DAY	YEAR	FA	FACILITY NAME			DOCKET NUMBER				
12	21	2002	2002 - 003 - 00 02			19	2003	FA	CILITY NAME DOCKET NUMBER								
9. OPERAT	ING		11. THIS REPORT			IS SUBM	3MITTED PURSUANT			TO THE REQUIREMENTS OF			10 CFR 3: (Check all that apply)				
MODE		1	20	2201(b)		20 220)3(a)(3)	(II)	50 73(a)(2)(ii)(B)			50 73(a)(2)(ix)(A)					
10. POW	ER	100	20	2201(d)	_	20.220)3(a)(4)	·)		50 73(a)(2)(iii)	50 73(a)(2)(x)					
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			20	2203(a)(2)(iii)		50 46	(a)(3)(11) (a)(2)(1)(A) (a)(2)(i)(B) (a)(2)(1)(C)			50 73(a)(2)(v)(C)		NRC Form 366A					
			20	2203(a)(2)(IV))	50 73				50 73(a)(2)(v)(D)]					
			20.	2203(a)(2)(v)	X	50 73				50 73(a)(2)(v	ii)						
			20	2203(a)(2)(vi		50 73			 	50 73(a)(2)(v	iu)(A)						
			20 2203(a)(3)(i)			50 73(a)(2)(ii)(A)		50.73(a)(2)(viii)(B)		ш)(B)							
12. LICENSEE CONTACT FOR THIS LER																	
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14. SUPPL		EMENTAL REPORT EXPECTE			GIED	<u> </u>			SUBMISSION		MOI		DAT	TEAR			
YES (If yes, complete EXPECTED SUBMISSION DATE) X NO DATE																	
16. ABSTRACT		1400 sp	aces, re	, approximate	aly 15	single-sj	bacediy	/pewnitien		») - AAlman A		!-					
On December 21, 2002, it was discovered during surveillance testing, that the pipe caps for two test																	
taps on the right channel containment hydrogen monitor instrument lines were not installed. The																	
open test taps rendered the right channel containment hydrogen monitor inoperable. The right																	
channel containment hydrogen monitor is believed to have been in this condition for approximately																	
20 months, exceeding the 30-day completion time of Technical Specification 3.3.7.A for restoring an																	
inoperable channel of containment hydrogen monitoring to operable status. A review of the status of																	
the left channel containment hydrogen monitor during the 20-month period that the right channel was																	
linoperable	revea	led or	ne oco	asion, in	Nov	/emb	er 20	02, dur	ing	y which the	e left c	han	nel	was in	operable		
for approx	imately	/ 9 da	ys, ex	ceeding	the '	72-ho	ur co	mpletic	n '	time of Te	chnica	al Sp	eci	ificatio	n 3.3.7.D		
for restorir	ng one	of two	o char	nnels of c	onta	ainme	nt hy	drogen	m	onitoring	to ope	rabl	e sf	tatus.			

Upon discovery, the test tap pipe caps were reinstalled and the right channel containment hydrogen monitor was declared operable.

This occurrence is reportable in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by Technical Specifications.

1. FACILITY NAME	2. DOCKET	6. LER NUMBER		3. PAGE	Ξ
		YEAR SEQUENTIAL REVISION NUMBER NUMBER			
PALISADES NUCLEAR PLANT	05000255	2002 _ 003 _ 00	2	OF	4
17. NARRATIVE (If more space is required, use additional EVENT DESCRIPTION	al copies of NRC Form 3	66A)			
On December 21, 2002, at approximate power, it was discovered during surver channel containment hydrogen monitor would have precluded the ability of the representative post-accident containing containment hydrogen monitor inoper believed to have been in this condition completion time of Technical Specification containment hydrogen monitoring to containment hydrogen monitoring to contain that the right channel was inoperable left channel was inoperable for approx Technical Specification 3.3.7.D for res monitoring to operable status. All oth inoperability during the 20-month peri	ately 2355 hours illance testing, t or [IK] instrumer e right channel on hent atmospheri able. The right n for approximation 3.3.7.A for operable status. nel containment revealed one of ximately 9 days, storing one of two er occasions of od were of dura	with the plant in Mode 1, op hat the pipe caps for two test t lines were not installed. The containment hydrogen monitor c sample, rendering the right channel containment hydrog- ely 20 months, exceeding the restoring an inoperable chan hydrogen monitor during the ccasion, in November 2002, of exceeding the 72-hour comp to channels of containment hydro left channel containment hydro tions less than the allowed 72	erating taps of he oper or to ob chann en mor e 30-da nnel of 20-mo during v oletion ydroge rogen 2 hours	on the on the otain a el nitor is ay onth p which time c en monit	on right taps eriod the of or
This occurrence is reportable in accor by Technical Specifications.	dance with 10 0	CFR 50.73(a)(2)(i)(B) as a co	ndition	prohi	bited
Upon discovery, the test tap pipe cap monitor was declared operable.	s were reinstalle	d and the right channel cont	ainmer	nt hyd	roger
CAUSE OF THE EVENT					
The right channel test tap pipe caps v the containment integrated leak rate t The cause of the failure to reinstall the individual failures involving inadequad verification. Contributing causes were included on the system checklist.	vere apparently est, occurring d e test tap pipe o cies in procedure e that the subjec	not reinstalled during the res uring the 2001 refueling outa aps is attributable to human e compliance, self-checking a st test tap pipe caps were not	toration ge in M perforr and ind labele	n port lay 20 nance lepend d and	ion of 001. e dent I not
The left channel containment hydroge maintenance and calibration under th monitor was operable.	en monitor was a e assumption th	emoved from service in Nov at the right channel containn	ember nent hy	2002 droge	for en

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NRC FORM 366AU.S. NUCLEAR REGULATORY C	OMMISSION						
LICENSEE EVENT REPORT (LI	ER)						
1. FACILITY NAME	2. DOCKET		6. LER NUMBER		3. PAGE		
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PALISADES NUCLEAR PLANT	05000255	2002	- 003 -	00	3	OF	4
17. NARRATIVE (If more space is required, use add CORRECTIVE ACTIONS	tional copies of NRC Form	366A) ·					
Operator performance expectations verification will be reinforced. The checklist.	s for procedure co subject test tap pi	mpliance be caps v	, self-checki vill be labele	ng and ii d and ac	ndepe Ided 1	endent to the	system
SAFETY SIGNIFICANCE							
The containment hydrogen monitor following an accident. This is acco hydrogen monitoring channels (left concentration in the containment re hydrogen recombiner to be placed	rs are used to dete omplished by placir channel procedur eaches between 1 in service.	ect the bu ng in serv ally prefe % and 3%	ildup of hydi ice at least o rred). If mea %, then proce	rogen in one of th asured h edures d	the co e two hydrog lirect a	ontain conta gen at leas	ment iinmen st one
The analysis of record does not cre event initiation. Given that in an ap indications for containment pressur reactor vessel water level, it is reas service within 24 hours regardless indication from the containment hy would also provide the operators a monitor.	edit a hydrogen red oplicable accident, re, containment ter sonable to assume of the existence of drogen monitors. n opportunity to ide	combiner there wo mperatur that a hy a valid o Rising ra entify the	to be in serviced be concu e, reactor co drogen reco containment diation levels inoperable o	vice until urrent ch ore tempe ombiner hydroge s in the a containn	24 h nallen eratur would n con auxilia nent h	ours a ged re, and l be pl centra ary bui nydrog	fter I/or aced in Ition Iding en
Additionally, recent analysis suppo 10 CFR 50.44, "Standards for com (Reference SECY-00-198 and SEC or no safety function in plants with containment, this generic analysis	orting proposed rule bustible gas contro CY-02-0080) demo large, dry containr is judged to be ap	emaking ol in light- onstrates nents. S olicable.	for risk inforr water-coole that hydroge ince Palisad	med cha d power en recorr es has a	nges react ibiner i large	to ors," rs serv e, dry	e little
The uninstalled test tap pipe caps channel containment hydrogen mo	also represented a nitor had been pla	i potentia ced in se	l radiologica rvice followi	l release ng an ac	e path ciden	if the It invol	right iving a

channel containment hydrogen monitor had been placed in service following an accident involving an in-containment release of radioactive iodines. The calculated radiological release results for this condition are bounded by the integrated release in the existing analysis of record, given the expectation that the release path through the open test taps would be recognized and isolated within 12.5 hours of event initiation.

Based on the above, the safety significance of this event was minimal.

2. DOCKET 05000255 opies of NRC Form 3	YEAR	6. LER NUMBER	REVISION		3. PAGE	
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	366A)					

NUCLEAR MANAGEMENT COMPANY PALISADES NUCLEAR PLANT Fax Cover Sheet

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Date:	2/20/03
Pages + Cover.	5
То:	Johnny Eads
Company:	U. S. Nuclear Regulatory Commission
Fax:	(301) 415-1222
Telephone:	(301) 415-1471
From:	Palisades Licensing
Fax:	(269) 764-3265
Phone:	(269) 764-2426
Message:	LICENSEE EVENT REPORT 02-003, INOPERABLE CONTAINMENT HYDROGEN MONITORS

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