

February 28, 2003

MEMORANDUM TO: Marsha Gamberoni, Deputy Director
New Reactor Licensing Project Office
Office of Nuclear Reactor Regulation

FROM: Lawrence J. Burkhart, AP1000 Project Manager */RA/*
New Reactor Licensing Project Office
Office of Nuclear Reactor Regulation

SUBJECT: DECEMBER 13, 2002, TELEPHONE CONFERENCE CALL SUMMARY

On Friday, December 13, 2002, a telephone conference call was held with Westinghouse Electric Company (Westinghouse) representatives and Nuclear Regulatory Commission (NRC) staff to discuss responses to requests for additional information (RAIs) 210.012, 210.019, and 210.028. Westinghouse submitted its responses to RAIs 210.012 and 210.019 on November 26, 2002 (ADAMS Accession No. ML023360097) and to RAI 210.028 on November 1, 2002 (ADAMS Accession No. ML023110249). The purpose of the telephone conference call was to discuss with Westinghouse representatives those RAIs that require further clarification. A list of participants is included in Attachment 1. Attachment 2 contains NRC staff comments regarding the subject RAIs that were sent to Mr. Mike Corletti of Westinghouse on December 12, 2002, and that were used to facilitate discussions during the telephone conference call.

With respect to RAIs 210.012 and 210.019, the NRC staff's comments state that a more rigorous analysis is needed to support Westinghouse's assumption that the AP600 faulted condition analysis for the reactor vessel internals envelopes the AP1000 design (see Attachment 2 for more details). Westinghouse representatives stated that they understood the issue but were not prepared, at the time of the call, to offer any feedback. The Westinghouse representatives stated that they would review the comments and submit revised RAI responses, if they felt it was appropriate.

With respect to RAI 210.028 concerning classification of design-basis pipe break loads, the Westinghouse representatives stated that they understood the issue but were not yet prepared to respond in the call. The Westinghouse representatives stated that they would review the comments and submit revised RAI responses, if they felt it was appropriate.

Docket No. 52-006

Attachment: As stated

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Docket No. 52-006

Attachment: As stated

Distribution:

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ACCESSION NUMBER: ML030570421 *See previous concurrence

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DATE	2/28/03	2/27/03	2/28/03

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DECEMBER 13, 2002
TELEPHONE CONFERENCE CALL SUMMARY
LIST OF PARTICIPANTS

Nuclear Regulatory Commission

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Dave Terao
Pat Sekerak

Westinghouse

Mike Corletti
Dave Altman
Dale Wizeman
Ed Cummins

Westinghouse AP1000
Design Certification Review

Request for Additional Information (RAI)
Category 210 - Mechanical Engineering

Subject: Review of Westinghouse Responses to RAI Questions

RAI 210.012; response incomplete:

- For purposes of a 10 CFR 52.47 design certification, the staff does not agree that final stress analysis of the RPV [reactor pressure vessel] internals, included the faulted condition combination of LOCA+SSE [loss-of-coolant accident and safe shutdown earthquake], can be deferred to the COL [combined license] stage.
- Applied loads for the AP1000 may be similar to the AP600, but due to differences in AP1000 internals geometry, the AP1000 stresses may be higher.
- The AP600 Prototype design was analyzed for design certification; the AP1000 Prototype design should also be analyzed for design certification.

RAI 210.019; response incomplete:

- Requested summary of results for faulted condition analysis of RPV internals not provided.
- Resolution of this issue will be the same as R210.012.

RAI 210.028, second paragraph not acceptable:

- Table 3.9-5, Design Loading Combinations for ASME Class CS Structures, Level C Service, should include DBPB [design basis pipe break] loading.
- The staff does not agree that DBPB loads are "secondary," nor do they produce stresses that are "secondary" (as defined in ASME III, NB-3213.9).
- DCD [Design Control Document] 3.9.5.2.2 states that DBPB loads are Level C Service Conditions for RPV internals design (which appears to contradict R210.028).

AP 1000

cc:

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