



Entergy Nuclear Northeast
Entergy Nuclear Operations, Inc
Entergy Nuclear Indian Point 2, LLC
P O Box 249
Buchanan, NY 10511

February 25, 2003

Re: Indian Point Unit No. 2
Docket No. 50-247
NL-03-026

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555-0001

Subject: 2002 Refueling Outage Inservice Inspection (ISI) Program Summary Report –
Third Outage, Second Period, Third Interval.

Reference: 1) Con Edison Letter to NRC dated April 2, 2001

Pursuant to the requirements of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, 1989 Edition without Addenda, Entergy Nuclear Operations, Inc. (ENO) hereby submits the 2002 Refueling Outage Inservice Inspection (ISI) Program Summary Report. In accordance with IWA-6230, this report is provided within 90 days of completion of the inspections. Pursuant to Indian Point Unit No. 2 Technical Specification Section 4.2.2, inspections of the primary and secondary side components were performed by ENO during the 2002 refueling outage. A report of previous inspections (2000 refueling outage) was provided by Reference 1.

The following report contents are provided as attachments to this letter:

- Non-Destructive Examination Report and Form NIS-1 Owner's Report for Inservice Inspections
- Form NIS-2 Owner's Report for Repair and Replacement
- Inservice Inspection Program Summary – Snubbers
- Inservice Inspection Program Summary – Pressure Tests

Detailed information pertaining to these reports is available at IP-2 for review.

No new regulatory commitments are being made by ENO in this correspondence.

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Should you or your staff have any concerns regarding this matter, please contact Mr. John McCann at 914-734-5074.

Sincerely yours,



Fred Dacimo
Vice President
Indian Point Energy Center

Attachments:

1. Non-Destructive Examination Report and Form NIS-1 Owner's Report for Inservice Inspections
2. Form NIS-2 Owner's Report for Repair and Replacement
3. Inservice Inspection Program Summary – Snubbers
4. Inservice Inspection Program Summary – Pressure Tests

C: Mr. Hubert J. Miller
Regional Administrator-Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
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Project Directorate I
Division of Licensing Project Management
U.S. Nuclear Regulatory Commission
Mail Stop O-8-C2
Washington, DC 20555

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
PO Box 38
Buchanan, NY 10511

ATTACHMENT 1 TO NL-03-026

Non-Destructive Examination Report
and
Form NIS-1 Owner's Report for Inservice Inspections

Entergy Nuclear Operations, Inc.
Indian Point Unit No. 2
Docket No. 50-247

Indian Point Unit No. 2
Third Outage, Second Period, Third Interval
Non-Destructive Examination Report

Introduction

Inservice inspections were performed between October 2002 and November 2002 at Indian Point Unit No. 2 during the 2002 refueling outage.

Examinations were performed in accordance with the requirements of:

1. The Entergy Nuclear Operations, Inc. (ENO) Third Ten Year Inservice Inspection Program
2. Technical Specifications
3. ASME Boiler & Pressure Vessel Code, Section XI, 1989 Edition without Addenda

The following items were examined:

1. Reactor Coolant Pump 23 main flange nuts
2. Class 2 piping and integral attachments

The examinations performed are summarized on the attached Form NIS-1.

Prior to these examinations, certification documents relative to personnel, equipment, and materials were reviewed and determined to be satisfactory.

Personnel from the Hartford Steam Boiler of Connecticut, and ENO conducted surveillances of and witnessed examinations and related activities.

There were no recordable indications noted during the inspection program.

FORM NIS-1 (Back)

8. Examination Dates 10/21/2002 to 10/29/2002

9. Inspection Period Identification November 1, 1997 to December 5, 2002

10. Inspection Interval Identification July 1, 1994 to April 5, 2006

11. Applicable Edition of Section XI 1989 Addenda None

12. Date/Revision of Inspection Plan January 19, 1994 Rev. 0

13. Abstract of Examinations and Tests. Include a list of examinations and tests and a statement concerning status of work required for the Inspection Plan.

See NIS-1 Examinations. attached.

14. Abstract of Results of Examinations and Tests.

All Items accepted.

15. Abstract of Corrective Measures.

N/A

We certify that a) the statements made in this report are correct, b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI, and c) corrective measures taken conform to the rules of the ASME Code, Section XI.

Certificate of Authorization No. (if applicable) None Expiration Date N/A

Date February 18, 2003 Signed Entergy by P. E. Dool
Owner

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of New York and employed by HSBCT of Hartford, CT have inspected the components described in this Owner's Report during the period January 3, 2001 to November 27, 2002, and state that to the best of my knowledge and belief, the Owner has performed examinations and tests and taken corrective measures described in this Owner's Report in accordance with the Inspection Plan and as required by the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations, tests, and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB 10011 'I' NYS 3084
Inspector's Signature National Board, State, Province, and Endorsements

Date 2/19/03, 2003

Inservice Inspection Report

Item	Category	System	Comp ID	Description	ISO No.	Method	Results
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Class 1

B6 200	B-G-1	RCS	RCPN 23-17	F NUTS	206923	VT	NRI
B6 200	B-G-1	RCS	RCPN 23-18	F NUTS	206923	VT	NRI
B6 200	B-G-1	RCS	RCPN 23-19	F NUTS	206923	VT	NRI
B6 200	B-G-1	RCS	RCPN 23-20	F NUTS	206923	VT	NRI
B6.200	B-G-1	RCS	RCPN 23-21	F NUTS	206923	VT	NRI
B6 200	B-G-1	RCS	RCPN 23-22	F NUTS	206923	VT	NRI
B6 200	B-G-1	RCS	RCPN 23-23	F NUTS	206923	VT	NRI
B6 200	B-G-1	RCS	RCPN 23-24	F NUTS	206923	VT	NRI

Class 2

C3.20	C-C	MS	3 S	IW MSR 26	206660	MT	NRI
C5 11N	C-F-1	SIS	15 16	CIRC WELD	206677	PT, UT	NRI
C5 11N	C-F-1	SIS	15 17	CIRC WELD	206677	PT, UT	NRI
C5 11N	C-F-1	SIS	15 26	CIRC WELD	206677	PT, UT	NRI
C5 11N	C-F-1	SIS	15 27	CIRC WELD	206677	PT, UT	NRI
C5.11N	C-F-1	SIS	15 28	CIRC WELD	206677	PT, UT	NRI
C5 11N	C-F-1	SIS	15 29	CIRC WELD	206677	PT, UT	NRI
C5 11N	C-F-1	SIS	15 30	CIRC WELD	206677	PT, UT	NRI
C5 11N	C-F-1	SIS	15 31	CIRC WELD	206677	PT, UT	NRI
C5 11N	C-F-1	SIS	51 16	CIRC WELD	206694	PT, UT	NRI
C5.11N	C-F-1	SIS	51 18	CIRC WELD	206694	PT, UT	NRI
C5 11N	C-F-1	SIS	51 29	CIRC WELD	206694	PT, UT	NRI
C5 11N	C-F-1	SIS	51 30	CIRC WELD	206694	PT, UT	NRI
C5 30	C-F-1	HIS	56 37	SOCKET WELD	206702	PT	NRI
C5 11N	C-F-1	SIS	57 5	CIRC WELD	206703	PT	NRI

ATTACHMENT 2 TO NL-03-026

Form NIS-2 Owner's Report for Repair and Replacement

Entergy Nuclear Operations, Inc.
Indian Point Unit No. 2
Docket No. 50-247

Indian Point Unit No. 2
Third Outage, Second Period, Third Interval
Repair/Replacement Report

Introduction

Repair/replacement work was performed at Indian Point Unit No. 2 during the time period from December 2000 through November 2002.

Work was performed in accordance with the requirements of:

1. Maintenance Administrative Directive (MAD) 36
2. ASME Boiler & Pressure Vessel Code, Section XI, 1989 Edition without Addenda

The following items were replaced on 18 different components:

1. Valves, valve bolting, pipe and elbow, valve bonnet, weld neck flanges, pipe tee and canopy seal clamp assemblies

The following item was repaired on 1 component:

1. 2" butt weld

The work performed is summarized on the attached Form NIS-2.

Personnel from the Hartford Steam Boiler of Connecticut and Entergy Nuclear Operations were given proper notifications so they could choose whether or not to conduct surveillances of and witness examinations.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Entergy Nuclear Operations, Inc.
 Name
295 Broadway, Suite 1, Buchanan, New York 10511-0249
 Address

Date February 13, 2003
 Sheet N/A of N/A

2. Plant Indian Point Nuclear Generating Station
 Name
295 Broadway, Suite 1, Buchanan, New York 10511-0249
 Address

Unit 2
None
 Repair Organization P.O. No., Job No., etc.

3. Work Performed by Owner
 Name
295 Broadway, Suite 1, Buchanan, New York 10511-0249
 Address

Type Code Symbol Stamp None
 Authorization No. None
 Expiration Date N/A

4. Identification of Systems AFW. CCW. CVCS. FW. MS. RCS. SIS. SW

5. (a) Applicable Construction Code: As identified in the plant Operating Equipment (OE) history
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
See Attached 2001-2002 Repair and Replacement List							

7. Description of Work _____

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 ½ in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks No ASME Section III or Section VIII components were repaired or replaced this outage. A listing of

Applicable Manufacturer's Data Reports to be attached

Additional non ASME III or VIII components which are included in the Section XI Code Repair & Replacement

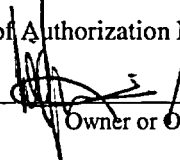
Program is contained in the attached NIS-2 table.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and these repairs/replacements conform to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp None

Certificate of Authorization No. None Expiration Date N/A

Signed  Date 2/13/03, 2003
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of New York and employed by HSBCT of Hartford, CT have inspected the components described in this Owner's Report during the period January 3, 2001 to November 27, 2002, and state that to the best of my knowledge and belief, the Owner has performed examinations and tests and taken corrective measures described in this Owner's Report in accordance with the Inspection Plan and as required by the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations, tests, and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

 Commissions NB 10011 'N'
Inspector's Signature National Board, State, Province, and Endorsements

Date 2/19/03, 2003

2001-2002 Repair and Replacement List

WORK ORDER #	CODE	TAG #	CORRECTIVE ACTION	WORK DESCRIPTION	COMP DATE	SYS
IP2-02-25690	A	21RV	REPLACEMENT	CANOPY SEAL CLAMP ASSY.	18-Nov-02	RCS
IP2-02-41202	A	897D	REPLACEMENT	2 STUDS AND NUTS	19-Nov-02	RCS
IP2-00-15613	B	326	REPLACEMENT	VALVE BONNET	10-Nov-02	CVCS
IP2-01-23583	B	218	REPLACEMENT	VALVE	08-Nov-02	CVCS
IP2-01-24563	B	236	REPLACEMENT	VALVE BONNET	15-Oct-02	CVCS
IP2-98-04855	B	310	REPLACEMENT	2" TEE	31-Aug-01	CVCS
IP2-00-16085	B	bfd-641	REPLACEMENT	STEM AND BONNET	05-Nov-02	FW
IP2-01-23396	B	MS-2A	REPLACEMENT	1 STUD AND NUT	02-Nov-02	MS
IP2-97-93691	B	MS-1-22	REPLACEMENT	STUDS AND NUTS	19-Nov-02	MS
IP2-00-18799	B	1863	REPLACEMENT	VALVE	06-Nov-02	SIS
IP2-02-61859	C	PSUPSR-480	REPLACEMENT	RESTRAINT REMOVED	20-Nov-02	AFW
IP2-02-61859	C	PSUPSR-481	REPLACEMENT	RESTRAINT REMOVED	20-Nov-02	AFW
IP2-02-02441	C	774D	REPLACEMENT	VALVE AND PIPE	16-Nov-02	CCW
IP2-00-16620	C	22MBDE	REPLACEMENT	STUDS AND NUTS	04-Jun-01	CVCS
IP2-01-21239	C	1299	REPLACEMENT	BOLTS AND NUTS	23-Apr-01	CVCS
IP2-01-23584	C	MS-52	REPLACEMENT	VALVE	12-Nov-02	MS
IP2-01-24386	C	MST-65	REPLACEMENT	NUTS	31-Oct-01	MS
IP2-00-17302	C	21SWPS	REPLACEMENT	STUDS AND NUTS	03-Aug-01	SW
IP2-01-22771	C	SWN-62-1	REPLACEMENT	STUDS	03-Nov-02	SW
IP2-01-23619	C	SWN-42-1	REPLACEMENT	VALVE	03-Nov-01	SW
IP2-01-23620	C	SWN-42-2	REPLACEMENT	VALVE	17-Nov-02	SW
IP2-01-23621	C	SWN-42-3	REPLACEMENT	VALVE	12-Nov-02	SW
IP2-01-23622	C	SWN-42-4	REPLACEMENT	VALVE	12-Nov-02	SW
IP2-01-24337	C	22CRFMC	REPLACEMENT	PIPE AND ELBOW	31-Oct-01	SW
IP2-02-00629	C	SWN-66	REPLACEMENT	NUTS AND BOLTS	24-Jul-02	SW
IP2-99-06789	C	SWN-4	REPLACEMENT	NUTS AND BOLTS	04-Nov-02	SW
IP2-02-59907	C*	LINE11C	REPLACEMENT	2" WELD NECK FLANGE	08-Nov-02	SW
IP2-02-60010	C*	LINE11A	REPLACEMENT	2" WELD NECK FLANGE	08-Nov-02	SW
IP2-02-60011	C*	LINE11B	REPLACEMENT	2" WELD NECK FLANGE	13-Nov-02	SW
IP2-02-54555	C	SWN-41-5A	REPAIR	2" FILLET WELD	17-Nov-02	SW

* ASME Code Case N-416-1 was applied on these work orders

ATTACHMENT 3 TO NL-03-026

Inservice Inspection Program Summary – Snubbers

Entergy Nuclear Operations, Inc.
Indian Point Unit No. 2
Docket No. 50-247

Indian Point Unit No. 2
Third Outage, Second Period, Third Interval
Snubber Inspection Report

Inservice inspections were performed between October 2002 and November 2002 at Indian Point Unit No. 2 during the 2002 refueling outage.

Examinations were performed to satisfy the requirements of:

1. Indian Point No. 2 Surveillance Test Procedures PI-V1A and PI-V1B
2. Indian Point No. 2 Shock Suppressor Initial Functional Test PT-R34
3. Indian Point No. 2 Steam Generator Shock Suppressor Functional Test PT-R34A
4. Technical Specifications Section 4.12.A and Section 3.12
5. ASME Boiler & Pressure Vessel Code, Section XI, 1989 Edition without Addenda.

The following items were examined:

1. 5 pipe support hydraulic shock arrestors from Quality Group "A" (small bore)
2. 6 component support hydraulic shock arrestors from Quality Group "A" (large bore S/G)
3. 10 pipe support hydraulic shock arrestors from Quality Group "B" (small bore)
4. 19 non-Section XI pipe support hydraulic shock arrestors (small bore)

Certification documents relative to personnel, equipment and materials were reviewed and determined to be satisfactory prior to the start of examinations.

Visual examinations of the "as-found" and "as-left" conditions of the hydraulic shock arrestors were conducted by certified personnel from the Wyle Laboratories, under contract to ENO and certified personnel from ENO's "in house" Quality Control Department.

The work performed is summarized on the attached Form NIS-2.

Personnel from the Hartford Steam Boiler of Connecticut and ENO were given proper notifications to allow them the option of conducting surveillances or witnessing examinations.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Entergy Nuclear Operations, Inc.
 Name
295 Broadway, Suite 1, Buchanan, New York 10511-0249
 Address

Date February 18, 2003
 Sheet N/A of N/A

2. Plant Indian Point Nuclear Generating Station
 Name
295 Broadway, Suite 1, Buchanan, New York 10511-0249
 Address

Unit 2
 None
 Repair Organization P.O. No., Job No., etc.

3. Work Performed by Owner
 Name
295 Broadway, Suite 1, Buchanan, New York 10511-0249
 Address

Type Code Symbol Stamp None
 Authorization No. None
 Expiration Date N/A

4. Identification of Systems ACS. CVCS. MS. RCS. RHR. SGBD. SIS

5. (a) Applicable Construction Code: As identified in the plant Operating Equipment (OE) history
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1989, No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
See Attached 2002 Snubber Repair and Replacement List							

7. Description of Work See Attached Snubber List

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 ½ in. x 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

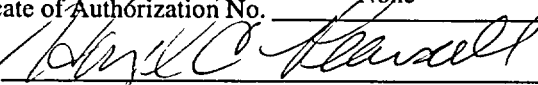
9. Remarks This NIS-2 covers the repair and replacement of snubbers during the 2R15 refueling outage.
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and these repairs/replacements conform to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp None

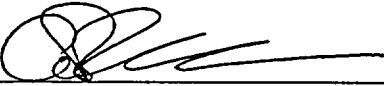
Certificate of Authorization No. None Expiration Date N/A

Signed  Date 2/18, 2003
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of New York and employed by HSBCT of Hartford, CT have inspected the components described in this Owner's Report during the period January 3, 2001 to November 27, 2002, and state that to the best of my knowledge and belief, the Owner has performed examinations and tests and taken corrective measures described in this Owner's Report in accordance with the Inspection Plan and as required by the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations, tests, and corrective measures described in the Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

 Commissions NB 10011 'N'
Inspector's Signature National Board, State, Province, and Endorsements

Date 2/19/03, 2003

2002 REFUELING OUTAGE

INSERVICE INSPECTION PROGRAM SUMMARY – SNUBBERS

AREA AND EXTENT OF EXAMINATIONS

IWB-2500 REFERENCE		EXAMINATION PROCEDURE
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QUALITY GROUP A PIPE SUPPORTS

BF1.10	Line 43	SR-1024A	Visual-Note 1
	Line 44	SR-1072	
	Line 62	62-SR-2	
	Line 79	SR-902	
	Line 351	PWR-127	

QUALITY GROUP A COMPONENT SUPPORTS

BF1.40	<u>21 Steam Generator</u> SG-21-1, SG-21-3	Visual-Note 1
	<u>23 Steam Generator</u> SG-23-1, SG-23-3	
	<u>24 Steam Generator</u> SG-24-1, SG-24-3	

QUALITY GROUP B PIPE SUPPORTS

BF1.20	Line 2	SR-M3A, SR-M1, SR-M2	Visual-Note 1
	Line 4	SR-M10A, SR-M9	
	Line 9	SR-55	
	Line 60	SR-703-1, SR-746B	
	Line 356	SR-720	
	Line V-3	SR-M29	

Note 1: Visual examination of snubbers was performed per plant Technical Specification 4.12.A. The visual inspection was performed in accordance with surveillance test procedure PI-V1A and PI-V1B.

FOR INFORMATION ONLY

The following NON-SECTION XI snubbers were visually inspected in accordance with surveillance test procedure PI-V1A and PI-V1B.

Line 38	38-SR-21
Line 70	70-SR-3, RCS-5
Line MS-3	MS-SR-129
Line V-3	SR-M33
Line V-4	SR-M52
Line V-5	SR-M34, SR-M37, SR-M39
Line 4	SR-M55
Line 8	SR-B5
Line 46	46-SR-30
Line 353	SR-737A
Line MS-3	SR-499
Line 14	14-SR-1
Line 14A	SR-1001, SR-1076
Line MS-3	SR-501
Line 361	361-SR-10

EXAMINATION INDICATIONS & DISPOSITION

ISI INDICATIONS- QUALITY GROUP A PIPE SUPPORT

62-SR-2 Pipe clamp was found out of alignment with the snubber. The misalignment between the pipe clamp and the snubber was more than the design allowable of + or - 5 degrees. The clamp has been realigned.

ISI INDICATIONS- QUALITY GROUP A COMPONENT SUPPORTS

21 Steam Generator

SG-21-1 There was a trace of fluid on the isolation screw and reservoir. Also one ear of the isolation screw was broken. This snubber was replaced due to its service life. The snubber was removed and a replacement snubber was installed. An as-left was performed to ensure all the requirements of the procedure are satisfied.

SG-21-3 Two top anchor bolts to the steam generator snubber base plate were found loose. The bolts are still fully engaged with no gap between the nut and base plate (nut is in full contact with the base plate). In addition there was some fluid on the reservoir. The snubber was removed and a replacement snubber was installed. The snubber was

functionally tested. An as-left was performed to ensure all the requirements of the procedure are satisfied.

23 Steam Generator

SG-23-1 There was a trace of fluid on the isolation screw and reservoir. Also one ear of the isolation screw was broken. This snubber was replaced due to its service life. The snubber was removed and a replacement snubber was installed. An as-left was performed to ensure all the requirements of the procedure are satisfied.

SG-23-3 Three of four anchor bolts to the base plate, bolts were found loose. Applied Loctite to the bolts and nuts to prevent loosening of the joints. This snubber was replaced due to its service life. The snubber was removed and a replacement snubber was installed. The snubber was functionally tested. An as-left was performed to ensure all the requirements of the procedure are satisfied.

24 Steam Generator

SG-24-1 This snubber was replaced due to its service life. The snubber was removed and a replacement snubber was installed. An as-left was performed to ensure all the requirements of the procedure are satisfied.

SG-24-3 The lower left anchor bolt to the base plate was found loose. Applied Loctite to the bolts and nuts to prevent loosening of the joints. This snubber was replaced due to its service life. The snubber was removed and a replacement snubber was installed. An as-left was performed to ensure all the requirements of the procedure are satisfied.

ISI INDICATIONS- QUALITY GROUP B PIPE SUPPORTS

SR-M2 There was debris on the piston rod. The snubber has been replaced with a spare so there is no adverse condition in the field. There was no indication of leakage; this snubber was replaced due to its service.

SR-M10A The piston rod appears scored. The snubber has been replaced with a spare. Snubber SR-M10A has been functionally tested in accordance with the testing program. The results of PT-R34 were successful for this snubber; therefore the snubber was operable for past service life. The score or indication had no impact on the operability of the snubber.

SR-55 A lack of thread engagement on one end of the turnbuckle of restraint. The snubber was replaced but the condition still exists. Approximately one thread is not engaged. This restraint will perform its design function in the as-found condition.

- SR-M29 The spherical bearing in paddle to pipe clamp is bound up. The snubber was subsequently removed for service life considerations and a replacement snubber was installed. The condition still exists in the as-left inspection.
- SR-M3A Could not see bottom clam bolt due to insulation but clamp was tight. Condition was evaluated by Engineering and found acceptable.
- SR-M9 Insufficient thread engagement. Condition evaluated by Engineering and found to meet design requirements in the as-installed configuration.

Reference:

Test Procedure No. TP-SQ-11035, Revision 5

2002 Snubber Repair/Replacement List

NL-03-026
Attachment 3

Snubber Id.	Sys.	Line #	IWB-2500	Removed		Replacement	
				Serial #	Mfg.	Serial #	Mfg.
SR-1024A	CVCS	43	F1.10	2500-3-940	Bergen Patt	2500-3-855	Bergen Patt.
SR-1072	CVCS	44	F1.10	G15198-1	Bergen Patt	2500-3-860	Bergen Patt.
SR-902	CVCS	79	F1.10	2500-3-1209	Bergen Patt	2500-3-966	Bergen Patt.
62-SR-2	RCS	62	F1.10	2500-3-830	Bergen Patt.	2500-3-1029	Bergen Patt
PWR-127	SIS	351	F1.10	2500-10-1279	Bergen Patt	2500-10-930	Bergen Patt
SG-21-1	MS	21SG	F1.40	2500-250-6	Bergen Patt.	2500-250-16	Bergen Patt
SG-21-3	MS	21SG	F1.40	2500-250-21	Bergen Patt.	2500-250-10	Bergen Patt
SG-23-1	MS	23SG	F1.40	2500-250-5	Bergen Patt.	2500-250-9	Bergen Patt
SG-23-3	MS	23SG	F1.40	2500-250-15	Bergen Patt	2500-250-13	Bergen Patt.
SG-24-1	MS	24SG	F1.40	2500-250-14	Bergen Patt	2500-250-7	Bergen Patt.
SG-24-3	MS	24SG	F1.40	2500-250-24	Bergen Patt	2500-250-3	Bergen Patt.
SR-M29	MS	V-3	F1.20	2500-3-1252	Bergen Patt	2500-10-1210	Bergen Patt
SR-M1	MS	2	F1.20	2500-10-1277	Bergen Patt.	2500-10-219	Bergen Patt.
SR-M10A	MS	4	F1.20	G43864-01-1	Bergen Patt.	2500-30-308	Bergen Patt.
SR-M2	MS	2	F1.20	2500-10-1276	Bergen Patt.	2500-10-218	Bergen Patt
SR-M3A	MS	2	F1.20	2500-50-253	Bergen Patt	2500-50-92	Bergen Patt.
SR-M9	MS	4	F1.20	2500-10-1283	Bergen Patt.	2500-10-928	Bergen Patt.
SR-55	RHR	9	F1.20	2500-10-1285	Bergen Patt	2500-10-921	Bergen Patt.
SR-703-1	SIS	60	F1.20	G43861-05-6	Bergen Patt	2500-3-866	Bergen Patt.
SR-720	SIS	356	F1.20	G20966-1-40	Bergen Patt.	2500-3-868	Bergen Patt.
SR-746B	SIS	60	F1.20	2500-3-878	Bergen Patt	2500-3-871	Bergen Patt.
46-SR-30	SGBD	46	N/A	2500-3-1373	Bergen Patt.	2500-3-309	Bergen Patt
SR-499	MS	MS-3	N/A	2500-3-1245	Bergen Patt	2500-3-889	Bergen Patt.
SR-737A	MS	353	N/A	2500-3-1229	Bergen Patt	2500-3-827	Bergen Patt
14-SR-1	ACS	14	N/A	2500-3-991	Bergen Patt	2500-3-843	Bergen Patt.
SR-1001	ACS	14A	N/A	G39776-5A	Bergen Patt	2500-3-921	Bergen Patt.
SR-1076	ACS	14A	N/A	G20966-1-23	Bergen Patt	2500-3-922	Bergen Patt
SR-501	MS	MS-3	N/A	2500-3-1248	Bergen Patt.	2500-3-310	Bergen Patt.
SR-B5	FW	8	N/A	2500-20-701	Bergen Patt.	2500-20-485	Bergen Patt
38-SR-21	CVCS	38	N/A	G20966-1-189	Bergen Patt.	2500-3-926	Bergen Patt
MS-SR-129	MS	MS-3	N/A	G43861-03-27	Bergen Patt	2500-3-1010	Bergen Patt
SR-M33	MS	V-3	N/A	2500-3-1237	Bergen Patt	2500-3-892	Bergen Patt
SR-M34	MS	V-5	N/A	2500-3-1242	Bergen Patt	2500-3-909	Bergen Patt.
SR-M37	MS	V-5	N/A	2500-3-1234	Bergen Patt.	2500-3-906	Bergen Patt.
SR-M39	MS	V-5	N/A	2500-3-1243	Bergen Patt	2500-3-920	Bergen Patt
SR-M52	MS	V-4	N/A	2500-3-1214	Bergen Patt.	2500-3-904	Bergen Patt
SR-M55	MS	4	N/A	2500-10-1273	Bergen Patt.	2500-10-914	Bergen Patt
70-SR-3	RCS	70	N/A	39935R-1BA	Bergen Patt.	2500-3-877	Bergen Patt.
RCS-5	RCS	70	N/A	2500-10-1278	Bergen Patt	2500-10-918	Bergen Patt
361-SR-10	RHR	361	N/A	2500-3-1226	Bergen Patt	2500-3-842	Bergen Patt.

ATTACHMENT 4 TO NL-03-026

Inservice Inspection Program Summary – Pressure Tests

Entergy Nuclear Operations, Inc.
Indian Point Unit No. 2
Docket No. 50-247

Summary of Inservice Inspection Pressure Tests

Performed on Quality Group A and B (ASME Section XI

Classes 1 and 2) Pressure Retaining Components

Inservice system pressure tests of Quality Groups A and B systems and components were conducted at the Indian Point Unit No. 2 Nuclear Power Plant in order to meet ASME Section XI requirements for the interval. This testing was completed to conclude the pressure testing requirements for the second period of the ten-year interval and coincident with the completion of refueling outage No. 15 in November 2002. The Quality Group A and B systems and components were inspected using procedures updated to the current program requirements.

This program utilized visual examination methods in accordance with the requirements of:

1. ASME B&PV Code, Section XI, 1989 Edition
2. Technical Specifications, and
3. IP-2 Ten Year Inservice Inspection Program including relief requests

The areas tested and indications reported are summarized in Tables 1 and 2 of this attachment.

The Quality Group A examinations revealed seven indications at fittings and flanges (mechanical pressure boundary) and three indications at valve packing (mechanical non-pressure boundary). Four of the indications were immediately repaired. Six indications were deferred to the next available system depressurization. Of these six, two were leaks in valve internal stem pressure-bellows where the leakage was stopped by valve back-seating in one case and a temporary alteration of a leak-off line in the other case. The remaining four deferrals were for minor packing or instrument line fitting weepage.

The Quality Group B examinations revealed thirteen indications. Seven of these were repaired and six were deferred to future repair availability. Of these six, five were valve packing indications and one was a threaded connection at a pump casing vent line.

TABLE 1
AREAS TESTED

<u>TEST NO.</u>	<u>TITLE</u>	<u>QUALITY GROUP</u>	<u>DRAWINGS</u>
PT-R75	RCS INTEGRITY INSPECTION	A	9321-F-2738, 9321-F-2745, 208168
PI-3Y17	CVCS BORIC ACID MAKEUP ISI	B	9321-F-2736
PI-3Y17A	EMERGENCY BORATION LINE ISI	B	9321-F-2736
PI-3Y18	PW TO CVCS MAKEUP ISI	B	9321-F-2736
PI-3Y25	CONTAINMENT SPRAY PUMPS ISI	B	9321-F-2735
PI-3Y26	CONTAINMENT SPRAY DISCHARGE ISI	B	9321-F-2735 235296
PI-3Y28	RECIRCULATION PUMPS ISI	B	235296
PI-3Y38	MAIN STEAM TRAPS ISI	B	9321-F-2041
PI-3Y40	SAFETY INJECTION PUMPS ISI	B	9321-F-2735
PI-3Y41	SAFETY INJECTION DISCHARGE ISI	B	9321-F-2735 235296
PI-3Y41A	21 ACCUMULATOR TANK ISI	B	235296
PI-3Y41B	22 ACCUMULATOR TANK ISI	B	235296
PI-3Y41C	23 ACCUMULATOR TANK ISI	B	235296
PI-3Y41D	24 ACCUMULATOR TANK ISI	B	235296
PI-3Y41E	SAFETY INJECTION TOPPING PUMP ISI	B	9321-F-2735
PI-3Y45	RHR PUMPS SUCTION VALVE 882 ISI	B	9321-F-2735
PI-3Y46	REFUELING WATER STORAGE TANK ISI	B	9321-F-2735
PI-3Y48	SEAL RETURN OUTSIDE CONTAINMENT ISI	B	9321-F-2736
PI-3Y48A	SEAL RETURN INSIDE CONTAINMENT ISI	B	208168
PI-3Y49	LETDOWN OUTSIDE CONTAINMENT ISI	B	9321-F-2736

PI-3Y50	VCT BYPASS LINE ISI	B	9321-F-2736
PI-3Y51	LETDOWN INSIDE CONTAINMENT ISI	B	208168
PI3Y57	CHARGING OUTSIDE CONTAINMENT ISI	B	9321-F-2736
PI3Y57A	CHARGING INTSIDE CONTAINMENT ISI	B	208168
PI-3Y58	PURIFICATION BOOSTER PUMP ISI	B	208168
PI-3Y13	RECICULATION PUMP SAMPLE TEST	B	235296
PT-R12	RHR SYSTEM INSERVICE HYDRO	B	251783 235296 9321-F-2745 9321-F-2735 9321-F-2720

TABLE 2

SUMMARY OF REPORTED INDICATIONS

QUALITY GROUP A AND B PRESSURE RETAINING COMPONENTS

QUALITY GROUP A PRESSURE RETAINING COMPONENTS			
LEAKAGE TYPE	# FOUND	# REPAIRED	# DEFERRED
THROUGH WALL	0	0	0
MECHANICAL PRESSURE BOUNDARY	7	3	4
MECHANICAL NON-PRESSURE BOUNDARY	3	1	2
TOTAL ITEMS BY STATUS	10	4	6 *

* Includes those locations where corrective measures were implemented; however, a permanent repair was deferred.

QUALITY GROUP B PRESSURE RETAINING COMPONENTS			
LEAKAGE TYPE	# FOUND	# REPAIRED	# DEFERRED
THROUGH WALL	0	0	0
MECHANICAL PRESSURE BOUNDARY	1	0	1
MECHANICAL NON-PRESSURE BOUNDARY	12	7	5
TOTAL ITEMS BY STATUS	13	7	6