

February 20, 2003

Mr. Harold W. Keiser
Chief Nuclear Officer & President
PSEG Nuclear LLC - X04
Post Office Box 236
Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2, REQUEST
FOR ADDITIONAL INFORMATION (RAI) RE: REQUEST FOR CHANGE TO
TECHNICAL SPECIFICATIONS ON STEAM GENERATOR LOW-LOW TRIP
SETPOINT (TAC NOS. MB6489 AND MB6490)

Dear Mr. Keiser:

By letter dated September 26, 2002, PSEG Nuclear LLC submitted a request for a revision to Technical Specification (TS) Table 2.2-1, "Reactor Trip System Instrumentation Trip Setpoints," and Table 3.3-4, "Engineered Safety Feature Actuation System Instrumentation Trip Setpoints," for the Steam Generator Low-Low Level Trip setpoint and allowable values at the Salem Nuclear Generating Station, Unit Nos. 1 and 2.

The U.S. Nuclear Regulatory Commission staff is reviewing your request and has determined that additional information is necessary in order to complete its evaluation. We discussed the enclosed RAI with your staff during a telephone call on January 27, 2003. During the call, you agreed to respond to the enclosed RAI within 30 days from the date of this letter. If circumstances result in the need to revise the target date, please contact me at (301) 415-1324.

Sincerely,

/RA/

Robert J. Fretz, Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

Enclosure: RAI

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION

REQUEST FOR CHANGE TO TECHNICAL SPECIFICATION

TABLE 2.2-1, "REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS," AND

TABLE 3.3-4, "ENGINEERED SAFETY FEATURE ACTUATION SYSTEM

INSTRUMENTATION TRIP SETPOINTS," FOR

THE STEAM GENERATOR LOW-LOW LEVEL TRIP

SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2

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Question:

Section 4 of Attachment 1 to the September 26, 2002, letter states that the total calculated channel uncertainties for the low-low level channel are +12.233% and 10.339% for Salem Unit Nos. 1 and 2, respectively.

Describe the setpoint methodology, the calculations for the proposed setpoint, and allowable value for the low-low steam generator trip function. If the setpoint methodology is different from the existing methodology, the NRC will review it. The setpoint calculation for each unit should include the uncertainty values and the bases for all measurement instrument components, rack calibration accuracy, and the bias (pressure drop and equivalent % of narrow range span) assigned to account for the effect of the mid-deck plate pressure differential induced by the primary separator downcomer steam flow.

Enclosure

PSEG Nuclear LLC

cc:

Mr. David F. Garchow
Vice President - Operations
PSEG Nuclear - X04
P.O. Box 236
Hancocks Bridge, NJ 08038

Mr. John T. Carlin
Vice President - Nuclear Reliability and
Technical Support
PSEG Nuclear - N10
P.O. Box 236
Hancocks Bridge, NJ 08038

Mr. Gabor Salamon
Manager - Nuclear Safety and Licensing
PSEG Nuclear - N21
P.O. Box 236
Hancocks Bridge, NJ 08038

Jeffrie J. Keenan, Esquire
PSEG Nuclear - N21
P.O. Box 236
Hancocks Bridge, NJ 08038

Ms. R. A. Kankus
Joint Owner Affairs
PECO Energy Company
Nuclear Group Headquarters KSA1-E
200 Exelon Way
Kennett Square, PA 19348

Lower Alloways Creek Township
c/o Mary O. Henderson, Clerk
Municipal Building, P.O. Box 157
Hancocks Bridge, NJ 08038

Dr. Jill Lipoti, Asst. Director
Radiation Protection Programs
NJ Department of Environmental
Protection and Energy
CN 415
Trenton, NJ 08625-0415

Salem Nuclear Generating Station,
Unit Nos. 1 and 2

Brian Beam
Board of Public Utilities
2 Gateway Center, Tenth Floor
Newark, NJ 07102

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector
Salem Nuclear Generating Station
U.S. Nuclear Regulatory Commission
Drawer 0509
Hancocks Bridge, NJ 08038