72-1004



February 06, 2003 NUH03-03-09

Ms. Mary Jane Ross-Lee Spent Fuel Project Office, NMSS U. S. Nuclear Regulatory Commission 11555 Rockville Pike M/S O13-D-13 Rockville, MD 20852

- Subject: Submittal of Corrected Pages of Revision 4 of Application for Amendment No. 5 to the NUHOMS[®] Certificate of Compliance No. 1004 (TAC NO. L23343).
- Reference: 1. Revision 4 of Application for Amendment No. 5 to the NUHOMS[®] Certificate of Compliance No. 1004, Submitted January 24, 2002.

Dear Ms. Ross-Lee:

Enclosed herewith are 13 sets of corrected pages which address a pagination error in Chapter M.2 and a minor omission in updating page M.8-6 of Chapter M.8 of Reference 1. Also included herewith is a revised Table of Contents for Appendix M, Revision 4.

Please replace the affected pages of Reference 1 with the corrected pages enclosed herewith.

Should you or your staff require additional information to support review of this application, please do not hesitate to contact me at 510-744-6053.

Sincerely,

WSCIN U.B. Chopra

Licensing Manager

Docket 72-1004 Enclosures: As Stated

NMSSOL

ATTACHMENT C

List of Changed Appendix M Pages

- M.2-2
- *M.2-2a (New Page)*
- M.2-11
- M.2-12
- M.2-40 through M.2-42
- M.3.1-2
- M.3.1-2a (New Page)
- M.3.1-7
- M.4 (Entire Section)
- M.8-6

I

TABLE OF CONTENTS

Page

	•		TABLE OF CONTENTS	
χ				Page
\bigcirc	M.1	Genera	1 Discussion	M.1-1
		M.1.1	Introduction	M.1-2
		M.1.2	General Description of the NUHOMS [®] -32PT DSC	M.1-3
			M.1.2.1 NUHOMS [®] -32PT DSC Characteristics	M.1-3
			M.1.2.2 Operational Features	M.1-4
			M.1.2.3 Cask Contents	M.1-5
		M.1.3	Identification of Agents and Contractors	M.1-6
		M.1.4	Generic Cask Arrays	M.1-7
		M.1.5	Supplemental Data	M.1-8
		M.1.6	References	M.1-9
	M.2	Princip	al Design Criteria	M.2-1
		M.2.1	Spent Fuel To Be Stored	M.2-2
			M.2.1.1 General Operating Functions	M.2-3
		M.2.2	Design Criteria for Environmental Conditions and Natural Phenomena.	M.2-4
			M.2.2.1 Tornado Wind and Tornado Missiles	M.2-4
			M.2.2.2 Water Level (Flood) Design	M.2-4
			M.2.2.3 Seismic Design	M.2-4
			M.2.2.4 Snow and Ice Loading	M.2-4
			M.2.2.5 Combined Load Criteria	M.2-4
		M.2.3	Safety Protection Systems	M.2-8
$\langle \chi \rangle$			M.2.3.1 General	M.2-8
\bigcirc			M.2.3.2 Protection By Multiple Confinement Barriers and Systems.	M.2-8
			M.2.3.3 Protection By Equipment and Instrumentation Selection	M.2-8
			M.2.3.4 Nuclear Criticality Safety	M.2-8
			M.2.3.5 Radiological Protection	M.2-9
			M.2.3.6 Fire and Explosion Protection	M.2-9
		M.2.4	Decommissioning Considerations	M.2-10
		M.2.5	Summary of NUHOMS [®] -32PT DSC Design Criteria	M.2-11
		M.2.6	References	M.2-12
	М.3	Structu	Iral Evaluation	M.3.1-1
		M.3.1	Structural Design	M.3 .1-1
			M.3.1.1 Discussion	M.3 .1-1
			M.3.1.2 Design Criteria	M.3 .1-3
		M.3.2	Weights and Centers of Gravity	M.3 .2-1
		M.3.3	Mechanical Properties of Materials	M.3 .3-1
			M.3.3.1 Material Properties	M.3 .3-1
			M.3.3.2 Materials Durability	M.3 .3-2
		M.3.4	General Standards for Casks	M.3 .4-1
			M.3.4.1 Chemical and Galvanic Reactions	M.3 .4-1
			M.3.4.2 Positive Closure	M.3 .4-7
			M.3.4.3 Lifting Devices	M.3 .4-7
			M.3.4.4 Heat and Cold	M.3 .4-8

	•				
•		M.3.5	Fuel Rods.		M.3.5-1
		M.3.6	Structural .	Analysis (Normal and Off-Normal Operations)	M.3.6-1
			M.3.6.1	Normal Operation Structural Analysis	M.3.6-1
			M.3.6.2	Off-Normal Load Structural Analysis	M.3.6-10
		M.3.7	Structural.	Analysis (Accidents)	M.3.7-1
			M.3.7.1	Reduced HSM Air Inlet and Outlet Shielding	M.3.7-1
			M.3.7.2	Tornado Winds/Tornado Missile	M.3.7-2
			M.3.7.3	Earthquake	M.3.7-2
			M.3.7.4	Flood	M.3.7-6
			M.3.7.5	Accidental Cask Drop	M.3.7-7
			M.3.7.6	Lightning	M.3.7-14
			M.3.7.7	Blockage of Air Inlet and Outlet Openings	M.3.7-14
			M.3.7.8	DSC Leakage	M.3.7-14
			M.3.7.9	Accident Pressurization of DSC	M.3.7-14
			M.3.7.10	Load Combinations	M.3.7-14
			M.3.7.11	Evaluation of Poison Rod Assemblies	M.3.7-16
		M.3.8	References	3	M.3.8-1
	M.4	Therma	al Evaluation	٦	M.4-1
		M.4.1	Discussion		M.4-1
		M.4.2	Summary	of Thermal Properties of Materials	M.4-4
		M.4.3	Specificati	ons for Components	M.4-9
		M.4.4	Thermal E	valuation for Normal Conditions of Storage (NCS) and Trans	fer
			(NCT)		M.4-10
\mathbf{X}			M.4.4.1	NUHOMS [®] -32PT DSC Thermal Models	M.4-10
\smile			M.4.4.2	Maximum Temperatures	M.4-16
			M.4.4.3	Minimum Temperatures	M.4-17
			M.4.4.4	Maximum Internal Pressures	M.4-17
			M.4.4.5	Maximum Thermal Stresses	M.4-20
			M.4.4.6	Evaluation of Cask Performance for Normal Conditions	M.4-20
		M.4.5	Thermal E	valuation for Off-Normal Conditions	M.4-21
			M.4.5.1	Off-Normal Maximum/Minimum Temperatures during Stor	age.M.4-22
			M.4.5.2	Off-Normal Maximum/Minimum Temperatures during	•
				Transfer	M.4-23
			M.4.5.3	Off-Normal Maximum and Minimum Temperatures During	
				Storage/Transfer	M.4-23
			M.4.5.4	Off-Normal Maximum Internal Pressure During	
				Storage/Transfer	M.4-24
			M.4.5.5	Maximum Thermal Stresses	M.4-24
			M.4.5.6	Evaluation of Cask Performance for Off-Normal Conditions	sM.4-24
		M.4.6	Thermal E	valuation for Accident Conditions	M.4-25
			M.4.6.1	Blocked Vent Accident Evaluation	M.4-25
			M.4.6.2	Transfer Accident Evaluation	M.4-26
			M.4.6.3	Hypothetical Fire Accident Evaluation	M.4-26
			M.4.6.4	Fuel Cladding and Basket Materials	M.4-27
			M.4.6.5	Maximum Internal Pressures	M.4-27
\checkmark			M.4.6.6	Evaluation of Cask Performance During Accident Condition	nsM.4-28

	•				
•		M 4 7	Thermal Ex	valuation for Loading/Unloading Conditions	M.4-29
		1*1.7.7	M.4.7.1	Maximum Fuel Cladding Temperatures During Vacuum	
\sim			1.1.1.1.1.1	Drving	M.4-29
			M.4.7.2	Evaluation of Thermal Cycling of Fuel Cladding During	
				Vacuum Drving, Helium Backfilling and Transfer Operations.	M.4-29
			M.4.7.3	Reflooding Evaluation	M.4-30
		M 4 8	Determinat	ion of Minimum Effective Fuel Conductivity	M.4-32
		111.1.0	M 4.8.1	Determination of Bounding Effective Fuel Thermal	
			1011 11011	Conductivity	M.4-32
			M482	Calculation of Fuel Effective Specific Heat and Density	M.4-35
		MAQ	Derivation	of Effective Thermal Conductivity of Water Within the Neutron	n
		1411.2	Shield of th	ne OS197/OS197H Transfer Cask	M.4-37
		M 4 10	References		M.4-43
		M 4 11	Example Ir	nut Files	M.4-45
		1,1, ,,1	M.4.11.1	Example ANSYS Input File for Applying Heat Generation	M.4-45
			M.4.11.2	Example ANSYS Input File for Solar Heat Flux Application	M.4-47
	M.5	Shieldi	ng Evaluatic	n	M.5-1
		M.5.1	Discussion	and Results	M.5-3
		M.5.2	Source Spe	cification	M.5-4
			M.5.2.1	Gamma Source	M.5-5
			M.5.2.2	Neutron Source Term	M.5-6
			M.5.2.3	Axial Peaking	M.5-6
~		M.5.3	Model Spe	cification	M.5-7
\sim			M.5.3.1	Material Densities	M.5-7
		M.5.4	Shielding I	Evaluation	M.5-8
			M.5.4.1	Computer Programs	M.5-8
			M.5.4.2	Spatial Source Distribution	M.5-8
			M.5.4.3	Cross Section Data	M.3-9
			M.5.4.4	Flux-to-Dose-Rate Conversion	M.5-9
			M.5.4.5	Methodology	M.3-9
			M.5.4.6	Assumptions	M.5-10
			M.5.4.7	Source Region Homogenization	M.5-12
			M.5.4.8	HSM Dose Rates for the 125-1 on Configuration	M.5-13
			M.5.4.9	HSM Models for the 100-1 on Configuration	M. S- 14
			M.5.4.10	Data Reduction and HSM Dose Rate Results for the 123-101	M 5 14
			N 5 / 11	USM Doso Potes for the 100 Ton Configuration	WI.J-14 M 5.16
			M.5.4.11	TC Doce Dates for the 125-Ton Configuration	M 5_16
			M 5 / 12	Cask Dase Rates for 100-Ton Configuration	M 5_17
			M 5 / 1/	Supporting 3-D Analysis	M 5_12
		N 6 6	Annendiv	oupporting unarysis	M 5_20
		141.2.2	м 5 5 1	Sample SAS2H/ORIGEN-S Input File	M 5.20
			M 5 5 7	Sample HSM DORT Model (R7 Roof Neutron Model)	M 5_22
			M 5 5 3	Sample TC DORT Model (R7 Transfer Configuration)	M 5_22
			M 5 5 4	Sample MCNP Model	M 5-33
~ /		M 5 6	References		M.5-56
\sim					

	•				
*	M.6	Critical	ity Evaluatio	on	M.6-1
		M.6.1	Discussion	and Results	M.6-2
\mathbf{L}		M.6.2	Package Fu	el Loading	M.6-3
		M.6.3	Model Spe	cification	M.6-4
			M.6.3.1	Description of Calculational Model	M.6-4
			M.6.3.2	Package Regional Densities	
		M.6.4	Criticality (Calculations	
		11101 1	M.6.4.1	Calculational Method	
			M.6.4.2	Fuel Loading Ontimization	
			M.6.4.3	Criticality Results	
		M 6 5	Critical Be	nchmark Experiments	M 6-13
		111.0.0	M 6 5 1	Benchmark Experiments and Applicability	M 6-13
			M.6.5.2	Results of the Benchmark Calculations	M 6-14
		M 6 6	Appendix		M 6-15
		111.0.0	M 6 6 1	References	M 6-15
			M 6 6 2	KENO Plots of Various Cases	M 6-16
			M 6 6 3	Example CSAS25 Input Files	M 6-16
			M664	Design Basis Case CSAS25 Input Deck	M 6-25
			141.0.0.4	Design Dusis Cuse Corroze input Deek	
	M.7	Confin	ement		M.7-1
		M.7.1	Confineme	nt Boundary	M.7-2
			M.7.1.1	Confinement Vessel	M.7-2
			M.7.1.2	Confinement Penetrations	M.7-3
			M.7.1.3	Seals and Welds	M.7-3
\mathbf{X}			M.7.1.4	Closure	M.7-3
\smile		M.7.2	Requireme	nts for Normal Conditions of Storage	M.7-4
			M.7.2.1	Release of Radioactive Material	M.7-4
			M.7.2.2	Pressurization of Confinement Vessel	M.7-4
		M.7.3	Confineme	nt Requirements for Hypothetical Accident Conditions.	M.7-5
			M.7.3.1	Fission Gas Products	M.7-5
			M.7.3.2	Release of Contents	M.7-5
		M.7.4	References		M.7-6
	MQ	Operat	ing Systems		M 8-1
	IVI.8	M 8 1	Procedures	for Loading the Cask	M 8-2
		141.0.1	M 8 1 1	Preparation of the TC and DSC	M 8-2
			M 8 1 2	DSC Fuel I ording	M 8-3
			M 8 1 3	DSC Drving and Backfilling	M 8-5
			M 8 1 /	DSC Drying and Dackrining	M 8-7
			M 8 1 5	TC Downending and Transport to ISESI	M 8-8
			M 8 1 6	DSC Transfer to the HSM	
			M & 1 7	Monitoring Operations	
		Men	Procedures	for Unloading the Cask	ΝΙ.Ο-10 Μ Q_1/
		141.0.2	M & 2 1	DSC Retrieval from the HSM	M Q. 1 <i>Λ</i>
			M Q D D	Doc Nettieval Itolii the Holy	
		MOD	IVI.0.2.2	Achioval of Fuci Holli IIC DOC	۱۷۱.۵-14 ۸۸۰۵ م۸
		IVI.8.3		ing Susteme	IVI.0-23
\sim		IVI.8.4	ruel mandi	ing systems	1v1.8-23

		M.8.5	Other Oper	ating Systems	M.8-23
		M.8.6	Operation S	Support System	M.8-23
/		M.8.7	Control Ro	om and/or Control Areas	M.8-23
		M.8.8	Analytical	Sampling	M.8-23
		M.8.9	References		M.8-24
	M.9	Accepta	ince Tests a	nd Maintenance Program	M.9-1
	11113	M.9.1	Acceptance	e Tests	M.9-1
		1.1.2.1	M.9.1.1	Visual Inspection	M.9-1
			M.9.1.2	Structural Tests	M.9-1
			M.9.1.3	Leak Tests	M.9-1
			M.9.1.4	Component Tests	M.9-1
			M.9.1.5	Shielding Integrity Tests	M.9-2
			M.9.1.6	Thermal Acceptance Tests	M.9-2
			M.9.1.7	Poison Acceptance	M.9-2
		M.9.2	Maintenan	ce Program	M.9-10
		M.9.3	References		M.9.11
	M.10	Radiatio	on Protectio	n	M.10-1
	101.10	M.10.1	Occupation	nal Exposure	M.10-1
		M.10.2	Off-Site De	ose Calculations	M.10-2
			M.10.2.1	Activity Calculations	M.10-4
			M.10.2.2	Dose Rates	M.10-4
		M.10.3	References		M.10-6
٢	M 11	Accide	nt Analyses.		M.11-1
	141.1 1	M 11 1	Off-Norma	1 Operations	M.11-2
		1,1,1,1,1,1,1	M.11.1.1	Off-Normal Transfer Loads	M.11-2
			M.11.1.2	Extreme Temperatures	M.11-2
			M.11.1.3	Off-Normal Releases of Radionuclides	M.11-3
			M.11.1.4	Radiological Impact from Off-Normal Operations	M.11-4
		M.11.2	Postulated	Accidents	M.11-5
			M.11.2.1	Reduced HSM Air Inlet and Outlet Shielding	M.11-5
			M.11.2.2	Earthquake	
			M.11.2.3	Extreme Wind and Tornado Missiles	
			M.11.2.4	Flood	M.11-7
			M.11.2.5	Accidental TC Drop	
			M.11.2.6	Lightning	
			M.11.2.7	Blockage of Air Inlet and Outlet Openings	M.11-8
			M.11.2.8	DSC Leakage	
			M.11.2.9	Accident Pressurization of DSC	
			M.11.2.10	Fire and Explosion	
		M.11.3	References	· · · · · · · · · · · · · · · · · · ·	M.11-12
	M 12	Conditi	ons for Casl	k Use - Operating Controls and Limits or Technical	
	141.12	Specifi	cations		M.12-1

-

.

•

	M.13	Quality Assurance
Ì	M.14	Decommissioning

.

.

LIST OF TABLES

	•		
		LIST OF TABLES	
1 ×			<u>Page</u>
\sim	Table M.1-1	Nominal Dimensions and Weight of the NUHOMS [®] -32PT DSC	M.1-10
	Table M.2-1	Intact PWR Fuel Assembly Characteristics	M.2-13
	Table M.2-2	PWR Fuel Assembly Design Characteristics	M.2-14
	Table M.2-3	Initial Enrichment and Number of PRAs for Various Fuel Assembly Typ	bes.M.2-15
	Table M.2-4	Poison Rod Assembly (PRA) Description	M.2-16
	Table M.2-5	PWR Fuel Qualification Table for 1.2 kW per Assembly for the	
		NUHOMS [®] -32PT DSC (Fuel w/o BPRAs)	M.2-17
	Table M.2-6	PWR Fuel Qualification Table for 0.87 kW per Assembly for the	
		NUHOMS [®] -32PT DSC (Fuel w/o BPRAs)	M.2-18
	Table M.2-7	PWR Fuel Qualification Table for 0.7 kW per Assembly for the	
		NUHOMS [®] -32PT DSC (Fuel w/o BPRAs)	M.2-19
	Table M.2-8	PWR Fuel Qualification Table for 0.63 kW per Assembly for the	
		NUHOMS [®] -32PT DSC (Fuel w/o BPRAs)	M.2-20
	Table M.2-9	PWR Fuel Qualification Table for 0.6 kW per Assembly for the	
		NUHOMS [®] -32PT DSC (Fuel w/o BPRAs)	M.2-21
	Table M.2-10	PWR Fuel Qualification Table for 1.2 kW per Assembly for the	
		NUHOMS [®] -32PT DSC (Fuel w/ BPRAs)	M.2-22
	Table M.2-11	PWR Fuel Qualification Table for 0.87 kW per Assembly for the	
		NUHOMS [®] -32PT DSC (Fuel w/ BPRAs)	M.2-23
	Table M.2-12	PWR Fuel Qualification Table for 0.7 kW per Assembly for the	
		NUHOMS [®] -32PT DSC (Fuel w/ BPRAs)	M.2-24
XI	Table M.2-13	PWR Fuel Qualification Table for 0.63 kW per Assembly for the	
\smile		NUHOMS [®] -32PT DSC (Fuel w/ BPRAs)	M.2-25
	Table M.2-14	PWR Fuel Qualification Table for 0.6 kW per Assembly for the	
		NUHOMS [®] -32PT DSC (Fuel w/ BPRAs)	M.2-26
	Table M.2-15	Summary of 32PT-DSC Load Combinations	M.2-27
	Table M.2-16	Summary of Stress Criteria for Subsection NB Pressure Boundary	
	m 11) (0 17	Components.	M.2-31
	Table M.2-17	Summary of Stress Criteria for Subsection NG Components	M.2-32
	Table M.2-18	Classification of NUHOMS ⁻ -32P1 DSC Components	M.2-33
	Table M.2-19	Additional Design Criteria for NUHOMS \sim -32PT DSC	M.2-34
	Table M.2-20	Summary of NUHOMS [*] -32P1 Component Design Loadings ^{**}	M.2-35
	Table M.3.1-1	Alternatives to the ASME Code for the NUHOMS -32PT DSC	16016
	TILLO 10	Continement Boundary	M.3.1-5
	Table M.3.1-2	Alternatives to the ASME Code Exceptions for the NUHOMS -32P1	10217
	T-1-1- M 2 0 1	DSC Basket Assembly	M.3.1-7
	Table $M.3.2-1$	A SME Code Meteriale Date For SA 240 Time 204 Steinlose Steel	S. WI. 5 . 2-2
	Table $M.3.3-1$	Materials Data For ASTM A36 Steel	C-C.C.NI
	Table $M = 2$	ASME Code Materials Data For SA 240 Type VM 10 Stainlass Starl	IVI.3.3-4
	Table $M = 2$	A SME Code Properties for 6061 Aluminum	C-C. C.IVI
	Table $M = 2.5$	Analysis Properties for Aluminum Transition Dails	1VI
	Table M 2 2 4	Additional Material Droperties	
	1 adie IVI.3.3-0	Auunonai Materiai Fropenies	1VI.3.3-8

•

•	Table M.3.4-1	Summary of Thermal Stress Results - 32PT Basket with Steel Transition
\sim	Table M.3.4-2	Summary of Thermal Stress Results - 32PT Basket with Aluminum
	Table $M 3.6$	NUHOMS [®] Normal Operating Loading Identification M 3 6-13
	Table M $3.6-7$	Maximum NIHOMS [®] -32PT DSC Shell Assembly Stresses for Normal
	14010 141.5.0-2	and Off-Normal Loads M 2 6-14
	Table M 3 6-3	NUHOMS [®] -32PT Basket Model Components Flement Types and
		Materials M 2 6-16
	Table M 3 6-1	Material Properties Used in Normal Condition 32PT Basket Analyses M 3 6-17
	Table M.3.6-5	Summary of Results for 32PT Basket Assembly Deadweight Analyses M 2 6-18
	Table $M.3.0-3$	Summary of Results for 32PT Basket Assembly On-Site Handling (2.0g
	1 aute 101.5.0-0	Loads) M 2 6.10
	$T_{abla} M 2 6 7$	22PT Basket Analyses Used to Determine On-Site Handling Loads M 2 6 20
	Table M.J. $0-7$	MIHOMS [®] Off Normal Operating Loading Identification M.2.6.21
	Table M.3.0-0 Table M $2.7.1$	Maximum NI HOMS [®] 32PT DSC Stresses for Drop Accident L code $M_{2,7,17}$
	Table $M_2 7 2$	List of Drop Condition ANSVS Stress Analyses of the 32PT Basket
	Table 101.5.7-2	Assembly M 2 7 18
	Table M 3 7 3	Summary of Material Properties for Drop Accident Analyses of the 32PT
	1 able 101.5.7-5	Basket Assembly M 2 7 10
	Table M 2 7 A	32PT Basket Enveloping Stress Results - 75g Side Drops M 2 7-20
	Table $M.3.7-4$	32PT Basket, Enveloping Stress Results - 60g Part 71 End Drop M 3 7 21
	Table $M.3.7-3$	Dron Condition ANSVS Stability Analyses for the 32PT Basket Assembly M 2, 7, 22
	Table $M.3.7-0$	Summary of 22PT Basket Stability Analysis Side Drops M 2 7 23
× 1	Table $M.3.7-7$	NUHOMS [®] 32PT DSC Enveloping Load Combination Results for
\smile	1 able 141.5.7-0	Normal and Off Normal Loads M 2.7.24
	Table M 3 7 0	NULHOMS [®] -32PT DSC Enveloping Load Combination Results for
	1 auto 1v1.5.7-9	Accident Loads M 2 7-25
	Table M 3 7-10	NUHOMS [®] -32PT DSC Enveloping Load Combination Results for
	1 abic 141.5.7-10	Accident Loads M 3 7-26
	Table M 3 7-11	DSC Enveloping Load Combination Table Notes M3 7-27
	Table M A_{-1}	Fuel Cladding Long-Term Storage Temperatures M 2-49
	Table M 4-7	Fuel Cladding Short-Term Normal Condition Maximum Temperatures M 4-50
	Table M 4-3	DSC Basket Assembly Maximum Normal Operating Component
	14010 141.4-5	Temperatures: Configuration 1 M 4-51
	Table M 4-4	DSC Basket Assembly Maximum Normal Operating Component
	14010 141.1 4	Temperatures: Configuration 2 $M \Delta -52$
	Table M 4-5	DSC Basket Assembly Maximum Normal Operating Component
	1 4010 141.4-5	Temperatures: Configuration 3 $M \Delta -53$
	Table M 4-6	32PT DSC Initial Helium Fill Molar Quantities $M \Delta .54$
	Table M 4-7	32PT DSC Maximum Normal Operating Condition Pressures $M \Delta_{-55}$
	Table M 4-8	Off-Normal Event Fuel Cladding Maximum Temperatures $M \triangle -56$
	Table M Δ_0	Off-Normal Event DSC Basket Assembly Maximum Component
	1 0010 111.4-2	Temperatures: Configuration 1 MA-57
	Table M 4-10	Off-Normal Event DSC Basket Assembly Maximum Component
•	1 4010 141.4-10	Temperatures: Configuration 2 $M \triangle -58$
۱.		

•			
	Table M.4-11	Off-Normal Event DSC Basket Assembly Maximum Component	
,		Temperatures; Configuration 3	M.4-59
	Table M.4-12	32PT DSC Maximum Off-Normal Operating Condition Pressures	M.4-60
	Table M.4-13	Accident Fuel Cladding Maximum Temperatures	M.4-61
	Table M.4-14	DSC Basket Assembly Maximum Accident Condition Component	
		Temperatures;	IM.4-62
	Table M.4-15	32P1 DSC Maximum Accident Condition Pressures	IVI.4-03
	Table M.4-16	Maximum Component Temperatures for the Hypothetical Fire Accident	
	m 11 37 4 17	Case for the NUHUMS -32P1 DSC in the TC	IVI.4-04
	1 able M.4-17	DSC Packet Assembly Maximum Component Temperatures During	101.4-05
	1 able M.4-18	Vocum Druing at 22 hours ⁽²⁾	NA 4 66
	T-1-1- N/ 5-1	DWD Evol Accombly Design Characteristics ⁽³⁾	$1\sqrt{1}.4-00$
	Table M.S-I	PWK Fuel Assembly Design Characteristics ¹¹	NA 5 58
	Table M.5-2	Doce Potes Due to the 22 DWD Fuel Assemblies with BDD As	101.3-38
	Table M.5-5	Summary of USM Date Pates	NA 5.60
	Table M.J-4	Summary of Onsite TC Dose Pates (Maximum ⁽¹⁾)	M 5-61
	Table M.5.6	PWP Fuel Assembly Materials Weights	M 5-62
	Table M.5-7	Flemental Composition of I WR Fuel-Assembly Structural Materials	M 5-63
	Table M 5-8	Flux Correct Factors By Assembly Region	M 5-64
	Table M 5-9	Gamma Source Term for 41 GWd/MTU, 3.1 wt. % U-235 and 5-Year	
	14010 111.5-5	Cooled Fuel	M.5-65
	Table M 5-10	Gamma Source Term for 30 GWd/MTU. 2.5 wt. % U-235 and 8-Year	
	10010 1110 10	Cooled Fuel	M.5-66
لمعد	Table M.5-11	Gamma Source Term for 45 GWd/MTU, 3.3 wt. % U-235 and 23-Year	
\sim		Cooled Fuel	M.5-67
	Table M.5-12	Design-Basis BPRA Source Terms	M.5-68
	Table M.5-13	Inner/Outer Heat Load Zone Region Volumes	M.5-69
	Table M.5-14	Total Neutron Source Summary	M.5-70
	Table M.5-15	Source Term Peaking Summary	M.5-71
	Table M.5-16	Shielding Material Densities	M.5-72
	Table M.5-17	Neutron Source for 45 GWd/MTU 3.3 wt. % U-235 16 Year Cooled Fuel	M.5-74
	Table M.5-18	Gamma Source Term for 45 GWd/MTU, 3.3 wt. % U-235 and 16-Year	
		Cooled Fuel	M.5-75
	Table M.5-19	Explicit Model Material Densities	M.5-76
	Table M.5-20	Smeared Model Material Densities	M.5-77
	Table M.5-21	Stainless Steel, Aluminum and Air Material Densities	M.5-78
	Table M.5-22	MCNP Explicit and Smeared Model Results	M.5-79
	Table M.5-23	HSM Accident Dose Rates	M.5-80
	Table M.5-24	3-D vs 2-D Comparison for 100-Ton Configuration	M.5-81
	Table M.5-25	Comparison of Calculated Versus Measured Dose Rates for DSC 45	M.5-82
	Table M.5-26	Comparison of Calculated Versus Measured Dose Rates for DSC 46	M.5-83
	Table M.6-1	Maximum Initial Enrichment For Each Configuration	M.6-30
	Table M.6-2	Authorized Contents for NUHOMS ⁻ -32P1 System	M.6-31
	Table M.6-3	Parameters For PWR Assemblies ⁵⁷	M.6-32
	Table M.6-4	Poison Rod Assembly (PRA) Description	M.6-33

•

•	Table M.6-5	Material Property Data	M.6-34
1 1	Table M.6-6	Most Reactive Fuel Type	M.6-35
\checkmark	Table M.6-7	Transition Rail Material Evaluation Results	M.6-36
	Table M.6-8	Fuel Clad OD Evaluation Results	M.6-37
	Table M.6-9	Poison/Aluminum and Aluminum Plate Thickness Evaluation Results.	M.6-38
	Table M.6-10	Basket Grid Structure Plate/Tube Thickness Evaluation Results	M.6-39
	Table M.6-11	Fuel Compartment Width Evaluation Results	M.6-40
	Table M.6-12	Assembly-to-Assembly Pitch Evaluation	M.6-41
	Table M.6-13	WE 17x17 Class Assembly without BPRAs Results	M.6-42
	Table M.6-14	WE 17x17 Class Assembly with BPRAs Results	M.6-43
	Table M.6-15	B&W 15x15 Class Assembly without BPRAs Results	M.6-44
	Table M.6-16	B&W 15x15 Class Assembly with BPRAs Results	M.6-45
	Table M.6-17	CE 15x15 Class Assembly without BPRAs Results	M.6-46
	Table M.6-18	WE 15x15 Class Assembly without BPRAs Results	M.6-47
	Table M.6-19	CE 14x14 Class Assembly without BPRAs Results	M.6-48
	Table M.6-20	WE 14x14 Class Assembly without BPRAs Results	M.6-49
	Table M.6-21	Criticality Results	M.6-50
	Table M.6-22	Benchmarking Results	M.6-51
	Table M.6-23	USL-1 Results	M.6-55
	Table M.6-24	USL Determination for Criticality Analysis	M.6-56
	Table M.10-1	Occupational Exposure Summary (125-ton configuration)	M.10-7
	Table M.10-2	Occupational Exposure Summary (100-ton configuration)	M.10-8
	Table M.10-3	Total Annual Exposure	M.10-9
	Table M.10-4	HSM Gamma-Ray Spectrum Calculation Results	M.10-10
\searrow	Table M.10-5	HSM Neutron Spectrum Calculations	M.10-11
	Table M.10-6	Summary of ISFSI Surface Activities	M.10-12
	Table M.10-7	MCNP Front Detector Dose Rates for 2x10 Array	M.10-13
	Table M.10-8	MCNP Back Detector Dose Rates for the Two 1x10 Arrays	M.10.14
	Table M.10-9	MCNP Side Detector Dose Rates	M.10.15
	Table M.11-1	Comparison of Total Dose Rates for HSM with and without Adjacent	
		HSM Shielding Effects	M.11-13
	Table M.11-2	TC Bounding Accident Dose Rate Results	M.11-14

•

LIST OF FIGURES

<u>Page</u>

	•		
,		LIST OF FICUDES	
		LIST OF FIGURES	Раде
i J			<u>1 450</u>
	Figure M.1-1	NUHOMS [®] -32PT DSC Components	M.1-11
	Figure M.1-2	Poison Rod Assemblies (PRAs)	M.1-12
	Figure M.2-1	Heat Load Zoning Configuration 1	M.2-37
	Figure M.2-2	Heat Load Zoning Configuration 2	M.2-38
	Figure M.2-3	Heat Load Zoning Configuration 3	M.2-39
	Figure M.2-4	Required PRA Locations for Configurations with Four PRAs	M.2-40
	Figure M.2-5	Required PRA Locations for Configurations with Eight PRAs	M.2-41
	Figure M.2-6	Required PRA Locations for Configurations with Sixteen PRAs	M.2-42
	Figure M.3-1	32PT-DSC Pressure and Confinement Boundaries	M.3.1-8
	Figure M.3.4-1	Potential Versus pH Diagram for Aluminum-Water System	M.3.4-15
	Figure M.3.4-2	32PT Basket with Steel Transition Rails, Temperatures and Stress	
		Intensities, -40°F in Cask	M.3.4-16
	Figure M.3.4-3	32PT Basket with Steel Transition Rails, Temperatures and Stress	
		Intensities, 117°F in Cask	M.3.4-17
	Figure M.3.4-4	32PT Basket with Aluminum Transition Rails, Temperatures and Stress	
		Intensities, -40°F in Cask	M.3.4-18
	Figure M.3.4-5	32PT Basket with Aluminum Transition Rails, Temperatures and Stress	
		Intensities, 117°F in Cask	M.3.4-19
	Figure M.3.6-1	32PT Basket Model with Steel Transition Rails	M.3.6-22
	Figure M.3.6-2	32PT Basket Model with Steel Transition Rails	M.3.6-23
•	Figure M.3.6-3	32PT Basket Model with Aluminum Transition Rails	M.3.6-24
\sim	Figure M.3.6-4	32P1 Basket Model with Aluminum Transition Rails	M.3.6-25
	Figure M.3.6-5	Location and Numbering of Stress Cuts for 32PT Basket Analyses	101.3.0-20
	Figure M.3.6-6	Deadweight Stress Intensity, 52PT Basket with Steel Transition Rans	NA 2 6 27
	T') () (7	(Support Rails at ±18.5°)	1VI.3.0-27
	Figure M.3.6-7	Deadweight + Thermal Stress Intensity, 32PT Basket with Steel Transiti	
	T ' 142 (0	Rails (Support Rails at $\pm 18.5^{\circ}$)	1v1.3.0-28
	Figure M.3.6-8	Deadweight Stress Intensity, 52PT Basket with Steel Transition Rans	142620
	Figure M 2 6 0	(Support Rails at ±30 ⁻)	1v1.5.0-29
	Figure M.S.0-9	Deadweight + Therman Stress Intensity, 52FT Dasket with Steel Hanshi Deils (Support Deile et ±209)	M 2 6 20
	$E_{auro} M 2 \in 10$	Deadwoight Stress Intensity 22PT Deaket with Aluminum Transition	1v1.5.0-50
	Figure M.S.0-10	Deadweight Stress Intensity, 52FT Basket with Aluminum Hansiton	NA 2 6 21
	$\Gamma_{\rm cons} M 2 < 11$	Ralls (Support Ralls at $\pm 10.5^{\circ}$)	1v1.5.0-51
	Figure M.S.O-11	Transition Deile (Summert Deile et 119.59)	112622
	Elaura M 2 6 10	Deadweight Stress Intensity 22DT Deaket with Aluminum Transition	1v1.5.0-52
	Figure $M.3.0-12$	Deadweight Stress Intensity, 52PT Basket with Aluminum Transition	NA 2 6 22
	Elaura 34.0 4.10	Rails (Support Kalls at ±30°)	1v1.3.0-33
	r1gure M.3.6-13	Deadweignt + Inernal Stress Intensity, 52P1 Basket with Aluminum	112624
	D ' D A A A A	I ransition Kalls (Support Kalls at $\pm 30^{\circ}$)	NA 2 7 22
	Figure M.3.7-1		1v1.3.7-28
	Figure M.3.7-2	0° Side Drop Stress Intensity, 32PT Basket with Steel Transition Rails	
		(Support Rails at ±18.5°)	M.3.7-29

1

•	Figure M 3 7-3	0° Side Drop Stress Intensity 32PT Basket with Aluminum Transition
	1 igure 141.5.7-5	Rails (Support Rails at +18 5°) M 3 7-30
\sim	Figure M 3 7_{-}	45° Side Dron Stress Intensity 32PT Basket with Steel Transition Bails
	1 iguic 101.5.7-4	(Support Rails at $\pm 18.5^{\circ}$) M 3.7-31
	Figure M 3 7-5	45° Side Dron Stress Intensity 32PT Basket with Aluminum Transition
	1 Igure 11.5.7-5	Rails (Support Rails at +18.5°) M 3.7-32
	Figure M 3 7-6	Displaced Shape at 113g, LS-DYNA Confirmatory Stability Analysis for
	1 Igule 11.5.7 0	0° Side Drop with Steel Transition Rails (Support Rails at +18.5°))
	Figure M.3.7-7	Displacement Time History, LS-DYNA Confirmatory Stability Analysis
	I Igure Miletri v	for 0° Side Drop with Steel Transition Rails
	Figure M.3.7-8	Displaced Shape at 124g, LS-DYNA Confirmatory Stability Analysis for
		180° Side Drop with Steel Transition Rails
	Figure M.3.7-9	Displacement Time History, LS-DYNA Confirmatory Stability Analysis
	0	for 180° Side Drop with Steel Transition Rails
	Figure M.4-1	Heat Load Zoning Configuration 1, Maximum Decay Heat for Various
	C	Assemblies
	Figure M.4-2	Heat Load Zoning Configuration 2, Maximum Decay Heat for Various
		Assemblies
	Figure M.4-3	Heat Load Zoning Configuration 3, Maximum Decay Heat for Various
		Assemblies
	Figure M.4-4	Axial Heat Profile for PWR Fuel
	Figure M.4-5	32PT-DSC Thermal ANSYS Model, Isometric View
	Figure M.4-6	52PT DSC Thermal ANSTS Model, Cross-Section view
\sim	Figure M.4-7	Thermal Model of TC M 4-74
	Figure M 4.0	Results for 100°F Storage Case With Heat I and Zoning Configuration 3 M 4-75
	Figure M 4-10	Results for 100°F Transfer Case With Heat Load Zoning Configuration 3 M A-76
	Figure M_{-11}	Results for 117°F Storage Case With Heat Load Zoning Configuration 3 MA-77
	Figure M $4-12$	Results for 117°F Transfer Case With Heat Load Zoning Configuration 3 M 4-78
	Figure M.4-13	Results for Blocked Vent Case With Heat Load Zoning Configuration 3 at
	i iguio inici io	40 Hours
	Figure M.4-14	NUHOMS [®] -32PT DSC and TC Temperature Response to 15 Minute Fire
	C	Accident Conditions
	Figure M.4-15	Time-History Profile of the Maximum Fuel Cladding Temperature during
		Blocked Vent Case, Configuration #3M.4-81
	Figure M.4-16	Maximum Fuel Temperature during Vacuum Drying with Heat Load
		Zoning Configuration 3
	Figure M.4-17	Temperature Distribution from Bottom to Top of DSC at Cross-Section
		with Highest Temperatures, 70°F HSM Storage Case (Configuration 3)M.4-83
	Figure M.4-18	Finite Element Model of B&W 15x15 Fuel Assembly
	Figure M.4-19	Fuel Axial Effective Conductivity
	Figure M.4-20	Fuel Transverse Effective Conductivity in Helium
	Figure M.4-21	Fuel Hallsverse Effective Conductivity in Vacuum
	Figure M.S-I	Explicit MCNP Model – Avial View
	rigure M.J-2	Explicit MONT MODEL - Axial View

	Figure M.5-3	Explicit MCNP Model – Radial View	M.5-86
,	Figure M.5-4	HSM Roof Model Geometry (125-Ton Configuration)	M.5-87
1	Figure M.5-5	HSM Floor Model Geometry (125-Ton Configuration)	M.5-88
	Figure M.5-6	HSM Side Model Geometry (125-Ton Configuration)	M.5-89
	Figure M.5-7	HSM Top Model Geometry (125- and 100-Ton Configuration)	M.5-90
	Figure M.5-8	HSM Roof Model Geometry (100-Ton Configuration)	M.5-91
	Figure M.5-9	HSM Front Wall Dose Rate Distribution (125-Ton Configuration)	M.5-92
	Figure M.5-10	Geometry for Front Wall Average Dose Rate Calculation	M.5-93
	Figure M.5-11	HSM Back Wall Dose Rate Distribution (125-Ton Configuration)	M.5-94
	Figure M 5-12	HSM Roof Dose Rate Distribution Perpendicular to DSC Axis(125-T	on
	I Iguio Ivito II	Configuration)	
	Figure M 5-13	HSM Roof Dose Rate Distribution Parallel to DSC Axis(125-Ton	
	i iguio inito ito	Configuration)	M.5-96
	Figure M 5-14	Surface Average Calculation Geometry	M.5-97
	Figure M.5-15	HSM Front Wall Dose Rate Distribution (100-Ton Configuration)	M.5-98
	Figure M 5-16	HSM Back Wall Dose Rate Distribution (100-Ton Configuration)	M.5-99
	Figure M.5-17	HSM Roof Dose Rate Distribution Perpendicular to DSC Axis (100-7	Ton
	I Iguio Illio II	Configuration	M.5-100
	Figure M.5-18	HSM Roof Dose Rate Distribution Parallel to DSC Axis (100-Ton	
		Configuration)	M.5-101
	Figure M.5-19	Cask Model Geometry (125-Ton Configuration)	M.5-102
	Figure M.5-20	Dose Rate Distribution Along Cask Side During Onsite Transfer (125	-Ton
	0	Configuration)	M.5-103
	Figure M.5-21	Cask Top-End Dose Rates During Decontamination (125-Ton	
1	e	Configuration)	M.5-104
	Figure M.5-22	Cask Top-End Dose Rates During Inner Cover Welding (125-Ton	
	-	Configuration)	M.5-105
	Figure M.5-23	Cask Top-End Dose Rates During Outer Cover Welding (125-Ton	
	_	Configuratio	M.5-106
	Figure M.5-24	Cask Model Geometry (100-Ton Configuration)	M.5-107
	Figure M.5-25	Dose Rate Distribution Along Cask Side During Onsite Transfer (100	-Ton
		Configuration)	M.5-108
	Figure M.5-26	Cask Top-End Dose Rates During Decontamination (100-Ton	
		Configuration)	M.5-109
	Figure M.5-27	Cask Top-End Dose Rates During Inner Cover Welding (100-Ton	
		Configuration)	M.5–110
	Figure M.5-28	Cask Top-End Dose Rates During Outer Cover Welding (100-Ton	
		Configuration)	M.5–111
	Figure M.5-29	MCNP Model – Cut alone axial centerline of the DSC (100-Ton	
		Configuration)	M.5–112
	Figure M.5-30	MCNP Model – Cut through the centerline of the DSC (100-10n	
		Configuration)	M.5 -113
	Figure M.6-1	NUHOMS [®] -32PT DSC Cross Section	M.6-57
	Figure M.6-2	Required PRA Locations for Configurations with Four PRAs	M. G- 58
	Figure M.6-3	Required PRA Locations for Configurations with Eight PRAs	M. 6 -59
	Figure M.6-4	Required PRA Locations for Configurations with Sixteen PRAs	M. G -60

.

	Figure M.6-5	KENO V.a units and Radial Cross Sections of the Model	M.6-61
;	Figure M.6-6	WE 17x17 Class Assembly	
	Figure M.6-7	CE 16x16 Class Assembly	
	Figure M.6-8	B&W 15x15 Class Assembly	
	Figure M.6-9	CE 15x15 Class Assembly	
	Figure M.6-10	WE 15x15 Class Assembly	
	Figure M.6-11	CE 14x14 Class Assembly	
	Figure M.6-12	WE 14x14 Class Assembly	M.6-85
	Figure M.8-1	NUHOMS [®] System Loading Operations Flow Chart	
	Figure M.8-2	NUHOMS [®] System Retrieval Operations Flow Chart	
	Figure M.10-1	Annual Exposure from the ISFSI as a Function of Distance	M.10.16
	Figure M.11-1	TC Bounding Accident Dose Rate Distribution	M.11-15
	-	-	

Ĵ

 g/cm^2 . The criticality analysis is based on 90% credit or 0.0063 g/cm^2 of B10. The use of 90% credit is allowed because poison material coupons are to be tested via neutron transmission plus statistical analysis of the neutron transmission results.

For calculating the maximum internal pressure in the NUHOMS[®]-32PT DSC, it is assumed that 1% of the fuel rods are damaged for normal conditions, up to 10% of the fuel rods are damaged for off normal conditions, and 100% of the fuel rods will be damaged following a design basis accident event. A minimum of 100% of the fill gas and 30% of the fission gases (e.g., H-3, Kr and Xe) within the ruptured fuel rods are assumed to be available for release into the DSC cavity, consistent with NUREG-1536 [2.1].

The maximum design basis internal pressures for the NUHOMS[®]-32PT DSC are 15, 20 and 105 psig for normal, off-normal and accident conditions of storage, respectively.

M.2.5 Summary of NUHOMS[®]-32PT DSC Design Criteria

The additional principal design criteria for the NUHOMS[®]-32PT DSC are presented in Table M.2-19. The NUHOMS[®]-32PT DSC is designed to store 32 intact standard PWR fuel assemblies with or without BPRAs with assembly average burnup, initial enrichment and cooling time as described in Table M.2-1. The maximum total heat generation rate of the stored fuel is limited to 1.2 kW per fuel assembly and 24 kW per NUHOMS[®]-32PT DSC in order to keep the maximum fuel cladding temperature below the limit [2.7]necessary to ensure cladding integrity. The fuel cladding integrity is assured by the NUHOMS[®]-32PT DSC and basket design which limits fuel cladding temperature and maintains a nonoxidizing environment in the cask cavity as described in Section M.4.

The NUHOMS[®]-32PT DSC (shell and closure) is designed and fabricated as a Class 1 component in accordance with the rules of the ASME Boiler and Pressure Vessel Code, Section III, Subsection NB, and the alternative provisions to the ASME Code as described in Table M.3.1-1.

The NUHOMS[®]-32PT DSC is designed to maintain a subcritical configuration during loading, handling, storage and accident conditions. A combination of fixed neutron absorbers, soluble boron in the pool and favorable geometry are employed to maintain the upper subcritical limit of 0.9411. The fixed neutron absorbers are in the form of borated metallic plates and PRAs which are inserted in the guide tubes of certain assemblies in the basket. The basket is designed and fabricated in accordance with the rules of the ASME Boiler and Pressure Vessel Code, Section III, Subsection NG, Article NG-3200 and the alternative provisions to the ASME Code as described in Table M.3.1-1.

The NUHOMS[®]-32PT DSC design, fabrication and testing are covered by TN's Quality Assurance Program, which conforms to the criteria in Subpart G of 10CFR72.

The NUHOMS[®]-32PT DSC is designed to withstand the effects of severe environmental conditions and natural phenomena such as earthquakes, tornadoes, lightning and floods. Section M.11 describes the NUHOMS[®]-32PT DSC behavior under these accident conditions.

M.2.6 References

2.1	NUREG-1536, "Standard Review Plan for Dry Cask Storage Systems," 1997.
2.2	American Society of Mechanical Engineers, ASME Boiler And Pressure Vessel Code, Section III, Division 1 - Subsections NB, NG and NF, 1998 edition including 2000 Addenda.
2.3	Young, W.C., "Roark's Formulas for Stress and Strain," 6 th Edition, McGraw-Hill Book Company, New York, 1989.
2.4	ANSI N14.5-1997, "Leakage Tests on Packages for Shipment," February 1998.
2.5	Deleted.
2.6	Deleted.
2.7	Interim Staff Guidance No. 11, Revision 2, "Cladding Considerations for the Transportation and storage of Spent Fuel", dated July 30, 2002.

-

. _____



Figure M.2-4 Required PRA Locations for Configurations with Four PRAs







Figure M.2-5 Required PRA Locations for Configurations with Eight PRAs

72-1004 Amendment No. 5



Figure M.2-6 Required PRA Locations for Configurations with Sixteen PRAs

cutting/welding operations. Verify that the measured hydrogen concentration does not exceed a safety limit of 2.4% [8.4]. If this limit is exceeded, stop all welding operations and purge the DSC cavity with 2-3 psig helium (or any other inert medium) via the 1/4 inch tygon tubing to reduce the hydrogen concentration safely below the 2.4% limit.

- 12. Perform dye penetrant weld examination of the inner top cover plate weld in accordance with the Technical Specification 1.2.5 requirements.
- 13. Connect the VDS to the DSC siphon and vent ports.
- 14. Install temporary shielding to minimize personnel exposure throughout the subsequent welding operations as required.
- 15. Engage the compressed air, nitrogen or helium supply and open the valve on the vent port and allow compressed gas to force the water from the DSC cavity through the siphon port.
- 16. Once the water stops flowing from the DSC, close the DSC siphon port and disengage the gas source.
- 17. Connect the hose from the vent port and the siphon port to the intake of the vacuum pump. Connect a hose from the discharge side of the VDS to the plant's radioactive waste system or spent fuel pool. Connect the VDS to a helium source.
- 18. Open the valve on the suction side of the pump, start the VDS and draw a vacuum on the DSC cavity. The cavity pressure should be reduced in steps of approximately 100 mm Hg, 50 mm Hg, 25 mm Hg, 15 mm Hg, 10 mm Hg, 5 mm Hg, and 3 mm Hg. After pumping down to each level, the pump is valved off and the cavity pressure monitored. The cavity pressure will rise as water and other volatiles in the cavity evaporate. When the cavity pressure stabilizes, the pump is valved in to complete the vacuum drying process. It may be necessary to repeat some steps, depending on the rate and extent of the pressure increase. Vacuum drying is complete when the pressure stabilizes for a minimum of 30 minutes at 3 mm Hg or less as specified in Technical Specification 1.2.2.

Caution: The vacuum drying step for 32PT DSC must meet the time duration limits of Technical Specification 1.2.17a.

- 19. Open the valve to the vent port and allow the helium to flow into the DSC cavity.
- 20. Pressurize the DSC with helium to about 24 psia not to exceed 34 psia.
- 21. Helium leak test the inner top cover plate weld for leakage in accordance with ANSI N14.5 to a sensitivity of 1 x 10^{-5} atm cm³/sec.
- 22. If a leak is found, repair the weld, repressurize the DSC and repeat the helium leak test.
- 23. Once no leaks are detected, depressurize the DSC cavity by releasing the helium through the VDS to the plant's spent fuel pool or radioactive waste system.