

February 11, 2003

Mr. Dale E. Young, Vice President
Crystal River Nuclear Plant (NA1B)
ATTN: Supervisor, Licensing & Regulatory Programs
15760 W. Power Line Street
Crystal River, Florida 34428-6708

SUBJECT: CRYSTAL RIVER UNIT 3 - PROPOSED CHANGE TO THE LOSS-OF-POWER
INSTRUMENTATION TECHNICAL SPECIFICATIONS (TAC NO. MB5384)

Dear Mr. Young:

The Commission has issued the enclosed Amendment No. 206 to Facility Operating License No. DPR-72 for Crystal River Unit 3 (Crystal River, the licensee). The amendment consists of changes to the existing Technical Specifications (TS) in response to your letter dated June 13, 2002.

The licensee proposed the replacement of a Completion Time that was incorrectly specified for Required Action D.2 of the Crystal River Improved TS (ITS) 3.3.8, "Emergency Diesel Generator (EDG) Loss of Power Start (LOPS)." The licensee also proposed that the incorrectly specified completion time be replaced with a completion time that is consistent with completion times specified for similar Required Actions in other sections of the Crystal River ITS, as well as similar Required Actions in NUREG-1430, "Standard Technical Specifications: Babcock and Wilcox Plants." In addition, the licensee requested the correction of a typographical error in Section 3.3.8 of the Crystal River ITS and the clarification of the discussion in Bases Section B 3.3.8 for Actions D.1 and D.2 to recognize the applicability of the ITS 3.3.8 in MODES 5 and 6.

A copy of the Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA by Matthew McConnell for/

Brenda L. Mozafari, Senior Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosures:

1. Amendment No. 206 to DPR-72
2. Safety Evaluation

cc w/enclosures: See next page

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AMENDMENT NO. 206 TO FACILITY OPERATING LICENSE NO. DPR-72 - CRYSTAL RIVER
UNIT 3

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DOCKET NO. 50-302

CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 206
License No. DPR-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power Corporation, et al. (the licensees), dated June 13, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-72 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 206, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Allen G. Howe, Chief, Section 2
Project Directorate II
Division of Project Licensing Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the
Technical Specifications

Date of Issuance: February 11, 2003

ATTACHMENT TO LICENSE AMENDMENT NO. 206

FACILITY OPERATING LICENSE NO. DPR-72

DOCKET NO. 50-302

Replace the following pages of the Appendix "A" Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3.3-20
3.3-21
B 3.3-71

Insert

3.3-20
3.3-21
B 3.3-71

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 206 TO FACILITY OPERATING LICENSE NO. DPR-72
FLORIDA POWER CORPORATION, ET AL.
CRYSTAL RIVER UNIT NO. 3 NUCLEAR GENERATING PLANT
DOCKET NO. 50-302

1.0 INTRODUCTION

By letter dated June 13, 2002, Florida Power Corporation (the licensee) submitted an application for amendment to the Crystal River Unit 3 (Crystal River) Facility Operating License No. DPR-72. The licensee proposed the replacement of a Completion Time that was incorrectly specified for Required Action D.2 of the Crystal River Improved Technical Specifications (ITS) 3.3.8, "Emergency Diesel Generator (EDG) Loss of Power Start (LOPS)." The licensee also proposed that the incorrectly specified completion time be replaced with a completion time that is consistent with completion times specified for similar Required Actions in other sections of the Crystal River ITS, as well as the same and similar Required Actions in NUREG-1430, "Standard Technical Specifications: Babcock and Wilcox Plants." In addition, the licensee requested the correction of a typographical error in Section 3.3.8 of the Crystal River ITS and the clarification of the discussion in Bases Section B 3.3.8 for Actions D.1 and D.2.

In February 2001, the licensee submitted license amendment request (LAR) #265, "Emergency Diesel Generator Loss of Power Start." In LAR #265, the licensee proposed changes to Crystal River ITS 3.3.8 to clarify the actions to be taken in the event that one or more channels of loss of voltage or degraded voltage emergency diesel generator (EDG) start functions become inoperable.

The changes proposed in LAR #265 included a revised Condition D specifying the Actions to be taken in the event that an inoperable channel of loss-of-voltage function cannot be restored to OPERABLE status within 72 hours. Revised Condition D requires the plant to be placed in MODE 3 within 6 hours and MODE 5 within 12 hours.

The Crystal River ITS 3.3.8 in effect at the time LAR #265 was submitted was identical to the EDG loss-of-power start specification contained in NUREG-1430, "Standard Technical Specifications: Babcock and Wilcox Plants." The revised Crystal River ITS 3.3.8 Condition D Completion Times, which were approved as part of LAR #265, were intended to replicate the standard guidance provided in NUREG-1430 and then-existing Crystal River ITS 3.3.8 to address Conditions where Required Actions and associated Completion Times for a specification cannot be met. For limiting conditions of operation (LCOs) that are applicable in MODES 1-4, the model technical specifications in NUREG-1430 provide that the plant be placed in MODE 3 within 6 hours and MODE 5 within 36 hours.

The revised Crystal River ITS 3.3.8 Condition D.2 of LAR #265 incorrectly specified a 12-hour Completion Time for placing the plant in MODE 5. This error was identified by a member of the Crystal River Operations staff following approval of LAR #265 and issuance of the license amendment. The changes proposed by the licensee in this amendment request will correct the error by replacing the 12-hour Completion Time currently specified in Crystal River ITS 3.3.8 Condition D.2 with the standard 36-hour Completion Time for placing the plant in MODE 5.

2.0 REGULATORY EVALUATION

The original licensing basis for emergency power at Crystal River is found in Final Safety Analysis Report Chapter 1 and follows proposed General Design Criterion (GDC) 39, "Emergency Power for Engineered Safety Features (ESF)." This is one of the General Design Criteria proposed by the Atomic Energy Commission in a proposed rulemaking published in the *Federal Register* on July 11, 1967, and was used to evaluate the adequacy of the electric power systems. Proposed GDC 39 provided that sufficient offsite and redundant, independent, and testable standby auxiliary sources of electrical power are available to attain a prompt shutdown and continued maintenance of the plant in a safe condition under all credible circumstances.

3.0 TECHNICAL EVALUATION

Proposed change to the Completion Time for Required Action D.2

The licensee proposed to revise the Crystal River ITS 3.3.8, "Emergency Diesel Generator (EDG) Loss of Power Start (LOPS)," by changing the Completion Time for Required Action D.2 from 12 hours to 36 hours.

As discussed above, NUREG-1430 provides guidance for situations where a Required Action and its associated Completion Time for a specification cannot be met. The time limits specified in this guidance to reach lower MODES of operation permit the plant shutdown to proceed in a controlled and orderly manner that is well within the specified maximum cooldown rate and within the capabilities of the unit, assuming only the minimum required equipment is OPERABLE. This reduces thermal stresses on components of the reactor coolant system and reduces the potential for a plant upset that could challenge plant systems. Previously existing Crystal River ITS provisions incorporate this approach in circumstances similar to those raised here. Replacing the 12-hour Completion Time currently specified in Crystal River ITS 3.3.8 Condition D.2 with the 36-hour Completion Time will allow the above described guidance to be met in the event that a plant shutdown to MODE 5 is required.

The proposed 36-hour Completion Time permits the plant shutdown to proceed in a controlled and orderly manner within the maximum specified cooldown rate and is therefore considered acceptable.

Proposed change to Bases Section B 3.3.8 for Action D.1 and D.2

The current discussion in the Crystal River Bases Section B 3.3.8 for Actions D.1 and D.2 implies that loss of voltage and degraded voltage functions need not be operable with the plant in MODE 5. The licensee has proposed to clarify Section B 3.3.8 for Actions D.1 and D.2 to ensure consistency with the applicability requirements for shutdown operation.

The loss of voltage and degraded voltage functions for both EDGs are required to be OPERABLE in MODES 1 through 4 by Crystal River ITS 3.3.8. In MODES 5 and 6, the loss of voltage and degraded voltage functions must also be operable, in accordance with Crystal River ITS 3.8.2, "AC Sources - Shutdown," for the EDG to be OPERABLE. The current Bases Section B 3.3.8 discussion for Actions D.1 and D.2 implies that these functions need not be operable with the plant in MODE 5.

Current wording: "If the inoperable channel cannot be restored to OPERABLE status within the associated Completion Time, the plant be placed in a MODE in which the LCO does not apply. To achieve this status, the plant must be placed in at least MODE 3 . . ."

Proposed wording: "If the inoperable channel cannot be restored to OPERABLE status within the associated Completion Time, the plant must be placed in at least MODE 3 . . ."

The proposed revised wording eliminates this implication and the NRC staff does not object to it.

Proposed change to the Instrumentation Section number

The Instrumentation Section number of Crystal River ITS 3.3.8 was inadvertently changed from 3.3 to 3.4. The licensee has proposed to correct the typographical error with this amendment request. The proposed change is editorial and is therefore considered acceptable.

4.0 SUMMARY

The proposed 36-hour Completion Time permits plant shutdown to proceed in a controlled and orderly manner. This completion time is within the specified maximum cooldown rate and within the capabilities of the unit, assuming only the minimum required equipment is OPERABLE. The NRC staff finds this to be an acceptable method for meeting proposed GDC 39. The typographical correction to the Instrumentation Section was also found acceptable, and the NRC staff has no objection to the licensee's proposed revision to the Bases.

5.0 STATE CONSULTATION

Based upon a letter dated March 8, 1991, from Mary E. Clark of the State of Florida, Department of Health and Rehabilitative Services, to Deborah A. Miller, Licensing Assistant, U.S. Nuclear Regulatory Commission, the State of Florida does not desire notification of issuance of license amendments.

6.0 ENVIRONMENTAL CONSIDERATIONS

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (67 FR 45570). Accordingly, the amendment meets the eligibility criteria for categorical

exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Knox

Date: February 11, 2003

Mr. Dale E. Young
Florida Power Corporation
cc:

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