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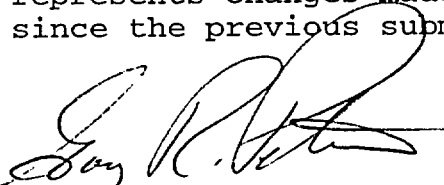
U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555-0001

Subject: Duke Energy Corporation
Catawba Nuclear Station, Units 1 and 2
Docket Nos. 50-413 and 50-414
Technical Specification Bases Changes

Pursuant to 10CFR 50.4, please find attached changes to the Catawba Nuclear Station Technical Specification Bases. These Bases changes were made according to the provisions of 10CFR 50.59.

Any questions regarding this information should be directed to A. P. Jackson, Regulatory Compliance, at (803) 831-3742.

I certify that I am a duly authorized officer of Duke Energy Corporation and that the information contained herein accurately represents changes made to the Technical Specification Bases since the previous submittal.



Gary R. Peterson

Attachment

A001

U.S. Nuclear Regulatory Commission
January 30, 2003
Page 2

xc: L. A. Reyes, Regional Administrator
U.S. Nuclear Regulatory Commission, Region II

R. E. Martin, Project Manager
U.S. Nuclear Regulatory Commission
Office of Nuclear Reactor Regulation, Mail Stop 0-8-G9

E. G. Guthrie
Senior Resident Inspector
Catawba Nuclear Station

January 30, 2003

Re: Catawba Nuclear Station
Technical Specifications (TS) Manual

Please replace the corresponding pages in your copy of the Catawba Technical Specifications Manual as follows:

REMOVE THESE PAGES

INSERT THESE PAGES

List of Effective Pages

Page 22


Page 22

Tab 3.4.16

B 3.4.16-5

B 3.4.16-5 – B 3.4.16-6

If you have any questions concerning the contents of this Technical Specification update, contact Jill Ferguson at (803) 831-3938.



Gary D. Gilbert
Manager, Regulatory Compliance

Page Number	Amendment	Revision Date
B 3.4.13-5	Revision 2	2/18/02
B 3.4.13-6	Revision 1	2/18/02
B 3.4.14-1	Revision 0	9/30/98
B 3.4.14-2	Revision 1	2/26/99
B 3.4.14-3	Revision 0	9/30/98
B 3.4.14-4	Revision 0	9/30/98
B 3.4.14-5	Revision 0	9/30/98
B 3.4.14-6	Revision 1	2/26/99
B 3.4.15-1	Revision 0	9/30/98
B 3.4.15-2	Revision 0	9/30/98
B 3.4.15-3	Revision 0	9/30/98
B 3.4.15-4	Revision 0	9/30/98
B 3.4.15-5	Revision 0	9/30/98
B 3.4.15-6	Revision 1	9/13/01
B 3.4.16-1	Revision 0	9/30/98
B 3.4.16-2	Revision 0	9/30/98
B 3.4.16-3	Revision 0	9/30/98
B 3.4.16-4	Revision 0	9/30/98
B 3.4.16-5	Revision 1	1/28/03
B 3.4.16-6	Revision 0	1/28/03
B 3.4.17-1	Revision 0	9/30/98
B 3.4.17-2	Revision 0	9/30/98
B 3.4.17-3	Revision 1	5/19/00
B 3.5.1-1	Revision 0	9/30/98
B 3.5.1-2	Revision 0	9/30/98
B 3.5.1-3	Revision 1	10/02/00
B 3.5.1-4	Revision 2	10/02/00
B 3.5.1-5	Revision 1	10/02/00
B 3.5.1-6	Revision 0	9/30/98
B 3.5.1-7	Revision 0	9/30/98
B 3.5.1-8	Revision 0	9/30/98
B 3.5.2-1	Revision 0	9/30/98

BASES

SURVEILLANCE REQUIREMENTS (continued)

Trending the results of this Surveillance allows proper remedial action to be taken before reaching the LCO limit under normal operating conditions. The Surveillance is applicable in MODES 1 and 2, and in MODE 3 with T_{avg} at least 500°F. The 7 day Frequency considers the unlikelihood of a gross fuel failure during the time.

SR 3.4.16.2

This Surveillance is performed in MODE 1 only to ensure iodine remains within limit during normal operation and following fast power changes when fuel failure is more apt to occur. The 14 day Frequency is adequate to trend changes in the iodine activity level, considering gross activity is monitored every 7 days. The Frequency, between 2 and 6 hours after a power change $\geq 15\%$ RTP within a 1 hour period, is established because the iodine levels peak during this time following fuel failure; samples at other times would provide inaccurate results. If the power excursion is one continuous process spanning over several hours, there is no need to sample every hour, only 2 to 6 hours after the last major power change of $\geq 15\%$ RTP, since this sample will encompass the maximum potential for additional iodine release to have occurred.

SR 3.4.16.3

A radiochemical analysis for \bar{E} determination is required every 184 days (6 months) with the plant operating in MODE 1 equilibrium conditions. The \bar{E} determination directly relates to the LCO and is required to verify plant operation within the specified gross activity LCO limit. The analysis for \bar{E} is a measurement of the average energies per disintegration for isotopes with half lives longer than 10 minutes, excluding iodines. The Frequency of 184 days recognizes \bar{E} does not change rapidly.

This SR has been modified by a Note that indicates sampling is required to be performed within 31 days after a minimum of 2 effective full power days and 20 days of MODE 1 operation have elapsed since the reactor was last subcritical for at least 48 hours. This ensures that the radioactive materials are at equilibrium so the analysis for \bar{E} is representative and not skewed by a crud burst or other similar abnormal event.

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- REFERENCES
1. 10 CFR 100.11, 1973.
 2. UFSAR, Section 15.6.3.
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