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10 CFR 50.55a

February 3, 2003
5928-03-20022

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 2055

Three Mile Island, Unit 1
Operating License No. DPR-50
NRC Docket No. 50-289

Subject: Response to Request for Additional Information Concerning Relief Requests
RR-02-19 and RR-02-20

References: 1. Letter from M. P. Gallagher (AmerGen Energy Company, LLC) to U. S.
Nuclear Regulatory Commission, dated March 5, 2002
2. Letter from T. G. Colburn (U. S. Nuclear Regulatory Commission) to J. L.
Skolds (AmerGen Energy Company, LLC), dated January 17, 2003

Dear Sir/Madam:

In the Reference 1 letter, AmerGen Energy Company, LLC requested approval of Relief Requests RR-02-19 and RR-02-20 for Three Mile Island, Unit 1. In the Reference 2 letter, the U. S. Nuclear Regulatory Commission provided a request for additional information regarding these relief requests. Attached is our response.

If you have any questions, please contact us.

Very truly yours,



Michael P. Gallagher
Director, Licensing and Regulatory Affairs
AmerGen Energy Company, LLC

Attachment - Response to Request for Additional Information

cc: H. J. Miller, Administrator, Region I, USNRC
USNRC Senior Resident Inspector, TMI
T. G. Colburn, USNRC Senior Project Manager
File No. 01086

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RESPONSE TO REQUEST FOR ADDITIONAL INFORMATION

THREE MILE ISLAND NUCLEAR STATION, UNIT 1 (TMI-1)

RELIEF REQUESTS RR-02-19 AND RR-02-20

DOCKET NO. 50-289

QUESTION 1:

There are several reactor vessel beltline welds for which the licensee obtained coverage that met current American Society for Mechanical Engineers (ASME) Code requirements (i.e., greater than 90% of the examination volume of each weld). Discuss the results obtained from these examinations and the relevance of these results to the vessel beltline welds discussed in the March 5, 2002, submittal, Parts A, B, and C.

ANSWER:

Reactor Vessel welds Nos. RCT0001RV0012WELD, RCT0001RV0013WELD, and RCT0001RV0014WELD were examined with greater than 90% of the examination volume of each weld. Weld No. RV0012 obtained 94% coverage, and weld Nos. RV0013 and RV0014 obtained 100% coverage. The results of these examinations were acceptable to ASME Code, Section XI acceptance criteria. There were no service induced degradation indications evaluated for these welds. These three (3) welds are in the beltline area of the reactor vessel. The welds in the subject submittal are not in the beltline region. These welds are in the lower section of the reactor vessel, which operates in lower fluence. The acceptability of the welds in the beltline region which achieved a coverage of greater than 90%, provides a high degree of confidence that the weld areas which were not able to be examined as described in the subject submittal, would not identify any service induced indications.

QUESTION 2:

For most of the welds discussed in the March 5, 2002, submittal, indications were identified and found to be within the ASME Code, Section XI, acceptance criteria. Confirm that none of these indications were the result of service-induced degradation.

ANSWER:

With regard to the welds discussed in the subject submittal, and the indications, which were identified and found to be within the ASME Code, Section XI, acceptance criteria, none of the indications were the result of service-induced degradation. When an indication is evaluated by Non-Destructive Examination (NDE) personnel, they consider: 1) the Pre-Service Inspection (PSI) of the weld; 2) any degradation mechanism that might be applicable to the component; 3) the original construction code acceptance criteria; 4) the location of the indication within the weld; and 5) the welding process or processes used. If it is determined the indication is service-induced, additional examinations would be performed in accordance with ASME Code, Section XI.