

**3.0 LIMITING CONDITIONS FOR OPERATION**

**2. Primary Containment Integrity**

- a. (1) Primary Containment Integrity as defined in Section 1, shall be maintained at all times when the reactor is critical or when the reactor water temperature is above 212°F and fuel is in the reactor vessel; except as specified in 3.7.A.2.a.(2) or 3.7.A.2.a.(3).
- (2) Primary Containment Integrity is not required when performing low power physics tests at atmospheric pressure during or after refueling at power levels not to exceed 5 MW(t).
- (3) Primary Containment Integrity is not required when performing reactor vessel hydrostatic or leakage tests with the reactor not critical.
- (4) If requirements of 3.7.A.2.a.(1) cannot be met, restore Primary Containment Integrity within one hour or be in at least Hot Shutdown within the next 12 hours and Cold Shutdown within the following 24 hours.

**4.0 SURVEILLANCE REQUIREMENTS**

**2. Primary Containment Integrity**

- a. Perform required visual examinations and leakage rate testing except for primary containment air lock testing, in accordance with the Primary Containment Leakage Rate Testing Program.

**3.0 LIMITING CONDITIONS FOR OPERATION**

**4.0 SURVEILLANCE REQUIREMENTS**

b. Deleted

b. Deleted

3.7/4.7

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**3.0 LIMITING CONDITIONS FOR OPERATION**

- c. When Primary Containment Integrity is required, the primary containment air lock shall be operable with both doors closed except when the air lock is being used, then at least one air lock door shall be closed.

With the primary containment air lock inoperable, maintain at least one air lock door closed and restore the air lock to Operable status within 24 hours or be in at least Hot Shutdown within the next 12 hours and in Cold Shutdown within the following 24 hours.

**4.0 SURVEILLANCE REQUIREMENTS**

- c. (1) Perform required primary containment air lock leakage rate testing in accordance with the Containment Leakage Rate Testing Program. (\*) (\*\*)
- (2) Once per 24 months, verify that only one door in the primary containment air lock can be opened at a time.
- d. The interior surfaces of the drywell shall be visually inspected each operating cycle for evidence of deterioration.

\* An inoperable air lock door does not invalidate the previous successful performance of the overall air lock leakage test.

\*\* Results shall be evaluated against acceptance criteria applicable to SR 4.7.A.2.a.

Bases 4.7 (Continued):

On September 26, 1995, Regulatory Guide 1.163 became effective providing guidance on performance based testing to the requirements of 10 CFR 50, Appendix J, Option B. Monticello has adopted 10 CFR Part 50, Appendix J, Option B, and a Primary Containment Leakage Rate Testing Program.

Maintaining primary containment integrity requires compliance with the visual examinations and leakage rate test requirements of the Primary Containment Leakage Rate Testing Program. Failure to meet air lock leakage testing, primary containment purge valve resilient seal leakage testing or main steam isolation valve leakage does not necessarily result in a failure of this SR. The impact of the failure to meet these SRs must be evaluated against the Type A, B and C acceptance criteria of the Primary Containment Leakage Rate Testing Program.

Maintaining primary containment air locks requires compliance with the leakage rate testing requirements of the Primary Containment Leakage Rate Testing Program. This SR reflects the leakage rate testing requirements with respect to air lock leakage (Type B leakage tests). The periodic testing requirements verify that the air lock leakage does not exceed the allowed fraction of the overall primary containment leakage rate. The frequency is required by the Primary Containment Leakage Rate Testing Program.

The SR has been modified by two footnotes. Note \* states that an inoperable air lock door does not invalidate the previous successful performance of the overall air lock leakage test. This is considered reasonable since either air lock door is capable of providing a fission barrier in the event of a DBA. Note \*\* has been added to this SR requiring the results to be evaluated against the acceptance criteria which is applicable to Primary Containment Integrity. This ensures that the air lock leakage is properly accounted for in determining the combined Type B and C primary containment leakage.

The air lock interlock mechanism is designed to prevent simultaneous opening of both doors in the air lock. Since both the inner and outer doors of an air lock are designed to withstand the maximum expected post accident primary containment pressure, closure of either door will support primary containment integrity. Thus, the interlock feature supports primary containment integrity while the air lock is being used for personnel transit in and out of the containment. Periodic testing of the interlock demonstrates that the interlock will function as designed and that the simultaneous inner and outer door opening will not inadvertently occur. Due to the purely mechanical nature of this interlock, and given that the interlock mechanism is not normally challenged when primary containment air lock door is used for entry and exit (procedures require strict adherence to single door opening), this test is only required to be performed every 24 months. The 24 month frequency is based on the need to perform this Surveillance under the conditions that apply during a plant outage, and the potential for loss of primary containment integrity if the Surveillance were performed with the reactor at power. The 24 month frequency is based on engineering judgment and is considered adequate given that the interlock is not challenged during the use of the air lock.

**6.8.J - RESERVED**

**K. Technical Specifications (TS) Bases Control Program**

This program provides a means for processing changes to the Bases of these Technical Specifications.

1. Changes to the Bases of the TS shall be made under appropriate administrative controls and reviews.
2. Changes to Bases may be made without prior NRC approval provided the changes do not involve either of the following:
  - a. a change in the TS incorporated in the license; or
  - b. a change to the USAR or Bases that requires NRC approval pursuant to 10 CFR 50.59.
3. The Bases Control Program shall contain provisions to ensure that the Bases are maintained consistent with the USAR.
4. Proposed changes to the Bases that involve changes as described in a. or b. of Specification 6.8.K.2 above shall be reviewed and approved by the NRC prior to implementation. Changes to the Bases implemented without prior NRC approval shall be provided to the NRC on a frequency consistent with 10 CFR 50.71(e).

**6.8.L - RESERVED**

**M. Primary Containment Leakage Rate Testing Program**

1. This program shall establish the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR Part 50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," dated September 1995.
2. The calculated peak containment internal pressure for the design basis loss of coolant accident,  $P_a$ , is 42 psig. The containment design pressure is 56 psig.

3. The maximum allowable containment leakage rate,  $L_a$ , at  $P_a$ , shall be 1.2% of containment air weight per day.
4. Leakage rate acceptance criteria are:
  - a. Containment leakage rate acceptance criterion is  $\leq 1.0 L_a$ . During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria are  $< 0.60 L_a$  for the Type B and C tests and  $\leq 0.75 L_a$  for Type A tests.
  - b. Air lock testing acceptance criteria are:
    - 1) Overall air leakage rate is  $\leq 0.05 L_a$  when tested at  $\geq P_a$ .
    - 2) For each door, leakage rate is  $\leq 0.007 L_a$  when pressurized to  $\geq 10$  psig.
5. The provisions of SRs 4.0.D and 4.0.E are applicable to the Primary Containment Leakage Rate Testing Program.
6. Nothing in these Technical Specifications shall be construed to modify the testing Frequencies required by 10 CFR 50, Appendix J.