UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET SW SUITE 23T85
ATLANTA, GEORGIA 30303-8931

February 6, 2003

Virginia Electric and Power Company
ATTN: Mr. David A. Christian
Senior Vice President and
Chief Nuclear Officer
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060

SUBJECT: NOTIFICATION OF TRIENNIAL FIRE PROTECTION BASELINE INSPECTION, (NRC INSPECTION REPORT NOS. 50-338, 339/2003-006)

Dear Mr. Christian:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a triennial fire protection baseline inspection at the North Anna Power Station in May 2003. The inspection team will be led by Mr. Gerry Wiseman, Senior Reactor Inspector - Fire Protection, of the Region II Office. The team will be composed of personnel from the NRC Region II Office and a contracted national laboratory. The inspection will be conducted in accordance with the NRC's baseline fire protection inspection procedure 71111.05.

The inspection objective will be to evaluate your fire protection program implementation with emphasis on post-fire safe shutdown capability and the fire protection features provided to ensure at least one post-fire safe shutdown success path is maintained free of fire damage. The inspection team will focus its review on the separation of the systems and equipment necessary to achieve and maintain safe shutdown and fire protection features of selected fire areas.

On January 29, 2003, during a telephone conversation with Mr. Jay Leberstien, Licensing Engineer, our respective staffs confirmed arrangements for a three-day information gathering onsite visit and a two-week onsite inspection. The schedule for the inspection is as follows:

- Information gathering onsite visit - April 22 - 24, 2003
- Week 1 of onsite inspection - May 5 - 9, 2003
- Week 2 of onsite inspection - May 19 - 23, 2003

The purposes of the information gathering visit are to obtain information and documentation needed to support the inspection; to become familiar with the North Anna fire protection program, fire protection features, post-fire safe shutdown capabilities and plant layout; and, as necessary, to obtain plant specific site access training and badging for unescorted site access. The types of documents the team will be interested in reviewing, and possibly obtaining, are listed in the Enclosure. Please contact Mr. Wiseman prior to preparing copies of the materials listed in the Enclosure. The inspection team will try to minimize your administrative burden by specifically identifying those documents required for inspection preparation.

During the information gathering visit, the team will also discuss the following inspection support administrative details: office space; specific documents requested to be made available to the team in their office spaces; arrangements for reactor site access (including radiation protection training, security, safety and fitness for duty requirements); and the availability of knowledgeable plant engineering and licensing organization personnel to serve as points of contact during the inspection.

We request that during the inspection weeks you ensure that copies of analyses, evaluations or documentation regarding the implementation and maintenance of the North Anna Power Station fire protection program, including post-fire safe shutdown capability, be readily accessible to the team for review. Of specific interest are those documents which establish that your fire protection program satisfies NRC regulatory requirements and conforms to applicable NRC and industry fire protection guidance. Also, personnel should be available at the site during the inspection who are knowledgeable regarding those plant systems required to achieve and maintain safe shutdown conditions from inside and outside the control room (including the electrical aspects of the relevant post-fire safe shutdown analyses), reactor plant fire protection systems and features, and the North Anna fire protection program and its implementation.

Your cooperation and support during this inspection will be appreciated. If you have questions concerning this inspection, or the inspection team's information or logistical needs, please contact Mr. Wiseman at (404) 562-4542, or me at (404) 562-4605.

Sincerely,

/RA: D. Charles Payne for:/

Charles R. Ogle, Chief
Engineering Branch 1
Division of Reactor Safety

Docket Nos.: 50-338, 50-339
License Nos.: NPF-4, NPF-7

Enclosure: Triennial Fire Protection Inspection
Support Documentation

cc w/encl: (See page 3)

cc w/encl:
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SIGNATURE	**WISEMAN**	**PAYNE**	**PAYNE FOR**	**LANDIS**			
NAME	WISEMAN	PAYNE	OGLE	LANDIS			
DATE	02/05/2003	02/05/2003	02/06/2002	02/06/2003	2/ /2003	2/ /2003	2/ /2003
E-MAIL COPY?	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO

OFFICIAL RECORD COPY DOCUMENT NAME: C:\ORPCheckout\FileNET\ML030380215.wpd

Triennial Fire Protection Inspection Support Documentation

[Note: This is a broad list of the documents the NRC inspection team may be interested in reviewing, and possibly obtaining, during the information gathering site visit. The current version of these documents is expected unless specified otherwise. Electronic media is preferred if readily available (i.e., on CD-ROM or computer disc). If electronic media is offered, we request that an index of files or a simple menu be provided.]

1. The Fire Protection Program and the Fire Hazards Analysis.

2. The fire protection program implementing procedures (e.g., administrative controls, surveillance testing, fire brigade).

3. The fire brigade training program and pre-fire plans.

4. Post-fire safe shutdown systems and separation analysis.

5. Post-fire alternative shutdown analysis.

6. Piping and instrumentation (flow) diagrams showing those systems and components used to achieve and maintain hot standby and cold shutdown for fires outside the control room and those components used for those areas requiring alternative shutdown capability.

7. Plant layout and equipment drawings which identify the physical plant locations of hot standby and cold shutdown equipment.

8. Plant layout drawings which identify plant fire area delineation, areas protected by automatic fire suppression and detection, and the locations of fire protection equipment.

9. Plant layout drawings which identify the general location of the post-fire emergency lighting units.

10. Plant operating procedures which would be used and describe shutdown from inside the control room with a postulated fire occurring in any plant area outside the control room and procedures which would be used to implement alternative shutdown capability in the event of a fire in the control, the cable spreading room or any other Appendix R, Section III.G.3 designated fire area.

11. Maintenance and surveillance testing procedures for alternative shutdown capability and fire barriers, detectors, pumps and suppression systems.

12. Maintenance procedures which routinely verify fuse breaker coordination in accordance with the post-fire safe shutdown coordination analysis.

13. A sample of significant fire protection and post-fire safe shutdown related design change packages and Generic Letter 86-10 evaluations for the past 3 year period.

ENCLOSURE

14. The reactor plant's IPEEE, results of any post-IPEEE reviews, and a list of actions taken or plant modifications conducted in response to IPEEE information.

15. Temporary modification procedure(s).

16. Current organization charts of site personnel down to the level of fire protection staff personnel.

17. If applicable, layout/arrangement drawings of potential reactor coolant pump lube oil system leakage points and associated lube oil collection systems.

18. Copies of the SERs which form the licensing basis for the Fire Protection Program and post-fire safe shutdown configuration.

19. Procedures/instructions that control the configuration of the fire protection program, features, and post-fire safe shutdown methodology and system design.

20. Procedures/instructions that govern the implementation of plant modifications, maintenance, and special operations, and their impact on fire protection.

21. The three most recent fire protection QA audits and/or fire protection self-assessments.

22. Recent QA surveillances of fire protection activities.

23. A list of open and closed fire protection problem identification and resolution reports [also known as action reports/condition reports/problem reports/problem investigation reports/NCRs/EARs] for the past 3 year period.

24. A list of the plant Fire Protection Program licensing basis documents.

25. A list of the applicable NFPA code versions committed to (NFPA codes of record) related to the design of plant fire protection features.

26. A list of plant code deviations from NFPA code commitments and any evaluations of these code deviations.