



U.S. NUCLEAR REGULATORY COMMISSION

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# REGULATORY GUIDE

OFFICE OF STANDARDS DEVELOPMENT

REGULATORY GUIDE 1.147  
(Task SC 721-4)

## INSERVICE INSPECTION CODE CASE ACCEPTABILITY ASME SECTION XI DIVISION 1

### A. INTRODUCTION

Section 50.55a, "Codes and Standards," of 10 CFR Part 50, "Domestic Licensing of Production and Utilization Facilities," requires, in part, that each operating license for a boiling or pressurized water-cooled nuclear power facility and each construction permit for a utilization facility be subject to the conditions in paragraph (g), "Inservice Inspection Requirements," of § 50.55a. Paragraph (g) requires, in part, that Classes 1, 2, and 3 components and their supports meet the requirements of Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components,"<sup>1</sup> of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code or equivalent quality standards. Paragraph 50.55a(b), in part, references the latest editions and addenda in effect of Section XI of the Code and any supplementary requirements to that section of the Code. Footnote 6 to § 50.55a states that the use of specific Code Cases may be authorized by the Commission upon request pursuant to paragraph 50.55a(a)(2)(ii), which requires that proposed alternatives to the described requirements or portions thereof provide an acceptable level of quality and safety.

General Design Criterion 1, "Quality Standards and Records," of Appendix A, "General Design Criteria for Nuclear Power Plants," to 10 CFR Part 50 requires, in part, that structures, systems, and components important to safety be designed, fabricated, erected, and tested to quality standards commensurate with the importance of the safety function to be performed. Where generally recognized codes and standards are used, Criterion 1 requires that they be identified and evaluated to determine their applicability, adequacy, and sufficiency and be supplemented or modified as necessary to ensure a quality product in keeping with the required safety function.

<sup>1</sup> Copies may be obtained from the American Society of Mechanical Engineers, United Engineering Center, 345 East 47th Street, New York, N.Y. 10017.

This regulatory guide lists those Section XI ASME Code Cases that are generally acceptable to the NRC staff for implementation in the inservice inspection of light-water-cooled nuclear power plants.

### B. DISCUSSION

The ASME Boiler and Pressure Vessel Committee publishes a document entitled "Code Cases."<sup>1</sup> Generally, the individual Code Cases that make up this document explain the intent of Code rules or provide for alternative requirements under special circumstances.

Most Code Cases are eventually superseded by revision of the Code and then are annulled by action of the ASME Council. In such instances, the intent of the annulled Code Case becomes part of the revised Code, and therefore continued use of the Code Case intent is sanctioned under the rules of the Code. In other instances, the Code Case is annulled because it is no longer acceptable or there is no further requirement for it. A Code Case that was approved for a particular situation and not for a generic application should be used only for the approved situation because annulment of such a Code Case could result in situations that would not meet Code requirements. The ASME considers the use of Code Cases to be optional for the user and not a mandatory requirement.

The Code Cases listed in this guide are limited to those cases applicable to Section XI of the Code.

All published Code Cases that are applicable to Section XI of the Code and were in effect on March 18, 1980, were reviewed for inclusion in this guide. Code Cases that are not listed herein are either not endorsed or will require supplementary provisions on an individual plant basis to attain endorsement status.

The endorsement of a Code Case by this guide constitutes acceptance of its technical position for applications not

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Comments and suggestions for improvements in these guides are encouraged at all times, and guides will be revised, as appropriate, to accommodate comments and to reflect new information or experience. This guide was revised as a result of substantive comments received from the public and additional staff review.

Comments should be sent to the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Docketing and Service Branch.

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precluded by regulatory or other requirements or by the recommendations in this or other regulatory guides. Contingent endorsement if applicable is indicated in regulatory position I for specific cases. However, it is the responsibility of the user to make certain that no regulatory requirements are violated and that there are no conflicts with other recommended limitations resulting from Code Case usage.

Acceptance or endorsement by the NRC staff applies only to those Code Cases or Code Case revisions with the date of "Council Approval" as shown in the regulatory position of this guide. Earlier or later revisions of a Code Case are not endorsed by this guide. New Code Cases will require evaluation by the NRC staff to determine if they qualify for inclusion in the approved list. Because of the continuing change in the status of Code Cases, it is planned that this guide will require periodic updating to accommodate new Code Cases and any revision of existing Code Cases.

### C. REGULATORY POSITION

1. The Section XI ASME Code Cases<sup>2</sup> listed below (by number, date of Council approval,<sup>3</sup> and title) are acceptable to the NRC staff for application in the inservice inspection of components and their supports for water-cooled nuclear power plants. Their use is acceptable within the limitations stated in the "Inquiry" and "Reply" sections of each individual Code Case, within the limitations of such NRC or other requirements as may exist, and within the additional limitations recommended by the NRC staff and given with the individual Code Case in the list.

1551 N-34	11/6/72 11/20/78	Inservice Inspection of Welds on Nuclear Components, Section XI
1646 N-72	8/12/74 11/20/78	Partial Postponement of Category B-C Examination for Class I Components, Section XI
1647 N-73	8/12/74 11/20/78	Partial Postponement of Category B-D Examination for Class I Components, Section XI
1705-1 N-98	3/1/76 11/20/78	Ultrasonic Examination - Calibration Block Tolerances, Section XI
1731 N-113	11/3/75 11/20/78	Basic Calibration Block for Ultrasonic Examination of Welds 10 in. to 13 in. Thick, Section XI, Division I
1738 N-118	12/22/75 11/20/78	Examination - Acceptance Standards for Surface Indications in Cladding, Section XI

<sup>2</sup>A numerical listing of the Code Cases appears in the appendix.

<sup>3</sup>When two dates are given, the earlier date is that on which the Code Case was approved by the ASME Council and the later date is that on which the Code Case was reaffirmed by the ASME Council.

Code Case 1738 (N-118) is acceptable subject to the following condition in addition to those conditions specified in the Code Case. Replace the last sentence of the "Reply" with the following: The provisions of this Code Case may not be applied for the examination of clad surfaces of nozzles, including the inner surface of the nozzle-to-vessel insert welds.

N-210 3/20/78 Exemption to Hydrostatic Tests After Repairs, Section XI, Division I

Code Case N-210 is acceptable subject to the following condition in addition to those conditions specified in the Code Case. Replace paragraph (3) of the "Reply" with the following: Repairs to piping, pumps, and valves where the depth of the repaired cavity does not exceed 25 percent of the wall thickness.

N-211 3/20/78 Recalibration of Ultrasonic Equipment Upon Change of Personnel, Section XI, Division I

N-216 3/10/78 Alternate Rules for Reactor Vessel Closure Stud Examination, Section XI, Division I

N-234 1/8/79 Time Between Ultrasonic Calibration Checks, Section XI, Division I

N-235 1/8/79 Ultrasonic Calibration Checks per Section V, Section XI, Division I

N-252 11/19/79 Low Energy Capacitive Discharge Welding Method for Temporary or Permanent Attachments to Components and Supports, Section III, Division I, and XI

Code Case N-252 is acceptable subject to the following condition in addition to those conditions specified in the Code Case: The applicant should indicate in the Safety Analysis Report the application, the material, and the thickness of the material to which the strain gage or thermocouple will be attached by CD welding.

N-278 3/17/80 Alternative Ultrasonic Calibration Block Configuration I-3131 and T-434.3, Section XI, Division I

Code Case N-278 is acceptable subject to the following conditions in addition to those conditions specified in the Code Case: When a universal calibration block is used and some or all of the reference holes are larger than the reflector holes at comparable depths recommended by Article IV, Section V, 1980 edition of the ASME Code, a correction factor should be used to adjust the DAC level to compensate for the larger reflector holes. Also, if the reactor pressure vessel was previously examined by using a conventional block, a ratio between the DAC curves obtained from the two

blocks should be noted (for reference) with the significant indication data.

2. Code Cases that were endorsed by the NRC in a prior version of this guide and were later annulled by action of the ASME Council should be considered as deleted from the list of acceptable Code Cases as of the date of the ASME Council action that approved the annulment. Such Code Cases that were annulled on or after November 1, 1978, are listed below by number, effective dates,<sup>4</sup> and title.

1730 N-112	11/3/75 7/1/79	Acceptance Standards for Class 2 and 3 Components, Section XI, Division 1
1804 N-167	1/14/77 1/14/80	Minimum Section Thickness Requirements for Repair of Nozzles, Section XI, Division 1

3. Code Cases that are not on the approved list of this guide (paragraph C.1) or other regulatory guides, or for which authorization by the Commission has not been granted, are not acceptable on a generic basis.

<sup>4</sup>Earlier date—date Code Case was approved by ASME Council; later date—date Code Case was annulled.

## D. IMPLEMENTATION

The purpose of this section is to provide information to applicants regarding the use of this regulatory guide.

1. Except for those Code Cases that have been annulled by action of the ASME Council, the NRC staff will authorize appropriate use of the Code Cases listed in this guide under regulatory position 1 upon specific request by the applicant in accordance with footnote 6 to §50.55a, the Codes and Standards rule. Code Cases that are actually used are to be identified in the Inservice Inspection Program.

2. Code Cases to be applied during an inspection interval or Code Cases for a preservice inspection need not be changed because a subsequent revision of the Code Case is listed as the approved version in this guide.

3. Code Cases to be applied during an inspection interval or Code Cases for a preservice inspection that were previously approved for use need not be changed because the Code Case has been subsequently annulled.

4. Code Cases on the approved list may be applied to procedures for an inspection interval and procedures for the inservice inspection that were established prior to the effective date of the Code Case within the limits specified in the Code Case and applicable regulations or recommended in other regulatory guides.

APPENDIX

Numerical Listing of Code Cases

1551	(N-34)	N-210
1646	(N-72)	N-211
1647	(N-73)	N-216
1705-1	(N-98)	N-234
1731	(N-113)	N-235
1738	(N-118)	N-252
		N-278

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