

FINAL AS-ADMINISTERED ADMINISTRATIVE JPMS

FOR THE MONTICELLO INITIAL EXAMINATION

OCTOBER 2002

MONTICELLO NUCLEAR GENERATING PLANT		JPM-C.2-05.B.1-002
TITLE:	CORE THERMAL LIMITS MONITORING	Revision 0
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JOB PERFORMANCE MEASURE

SRO: _____ SRO/RO: X SRO/RO/NLO: _____ NLO: _____

Plant Reference: C.2-05, Rev. 14

Task Standard: Perform Core Thermal Limits monitoring.

K/A Reference: Generic Task No.: 2.1.19 Rating RO/SRO: 3.0/3.0

Recent Events: None

Probabilistic Risk Assessment Human Error: None

Monticello Specific Task List Reference: CR200.129

METHOD OF TESTING		TIME FOR COMPLETION
Simulator: <u> X </u> In-Plant: _____	Normal: _____ Alternate Path: <u> X </u> _____	Estimated Time to Complete: <u> 10 min. </u> Maximum Time to Complete: <u> 20 min. </u> Time Critical: _____ Yes <u> X </u> No

Prepared By:	Date:
Reviewed By:	Date:
Shift Supv/Shift Mgr Review By:	Date:
Approved By:	Date:

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JPM SET-UP	
<p>Simulator Setup: Not Applicable</p> <p>Copy of Operations Daily Log – Part E 3D Monitor printout Place a red pencil on the control room desk.</p>	
<p>Initial Conditions: Rx Power has just been increased from 90% power to 98% with the Recirc. Flow Control system per C.2 –05 Load Following.</p>	
<p>Initiating Cues: The Control Room Supervisor has requested reactor power be increased to 100%. You have been directed to perform the Core Thermal Limits Monitoring for MFLCPR, MAPRAT, and MFLPD, per Ops Man C.2-05, Section B.1, Parts A, B, and C. Record data on Forms 0207, 0208, and 0225, of Operations Daily Log 0000 – E.</p>	

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PERFORMANCE INFORMATION

Performance Objective	Standard						
<p>STEP 1</p> <p>Locates procedure C.2-05, "Power Operation".</p>	<p>Standard: Locates appropriate procedure.</p> <p>Cue: None</p> <p>Comments: Parts A, B, and C may be performed in any order.</p>						
<p>STEP 2 (Procedure Part A, Step 1)</p> <p><u>IF</u> process computer is not anticipated to be available during a 24 hour period, <u>THEN</u> contact a Nuclear Engineer to arrange determination of MCPR.</p>	<p>Standard: None</p> <p>Cue: None</p> <p>Comments: Process computer is available.</p>						
<p>STEP 3 "C" (Procedure Part A, Step 2.a)</p> <p>When 3D Monicore computer is available, perform the following:</p> <p>a. Demand Official 3D Monitor to calculate core nodal power distribution, <u>AND</u> use MCPR data for the fuel assemblies that are closest to the limiting CPR.</p> <p align="center">NOTE 4: The most limiting MCPR is edited as MFLCPR on the Periodic Log</p> <p align="center">NOTE 5: MFLCPR is the Core Maximum ratio of power and flow dependent CPR limit to the bundle's actual CPR.</p>	<p>Standard: None</p> <p>Cue: Provide candidate with copy of 3D Monitor edit.</p> <p>Comments: A 3D Monitor edit cannot be demanded on the simulator.</p>						
<p>STEP 4 "C" (Procedure Part A, Step 2.b)</p> <p>b. Record the following data for the most limiting fuel bundle on Form 0225:</p> <p>a. The value of MFLCPR</p> <p>b. Core location of MFLCPR</p> <p>c. Percent rated core flow</p>	<p>Standard:</p> <p>Records data on Form 0225.</p> <table border="0"> <tr> <td>a. MFLCPR</td> <td>.883</td> </tr> <tr> <td>b. Core location</td> <td>13 - 32</td> </tr> <tr> <td>c. % rated core flow</td> <td>89.9%</td> </tr> </table> <p>Cue: None</p> <p>Comments: None</p>	a. MFLCPR	.883	b. Core location	13 - 32	c. % rated core flow	89.9%
a. MFLCPR	.883						
b. Core location	13 - 32						
c. % rated core flow	89.9%						

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Performance Objective	Standard
<p>STEP 5 "C"(Procedure Part A, Step 2.c)</p> <p>c. Verify MFLCPR is below the Action Limit (0.97 times the Tech Spec Limit) and the Tech Spec Limit.</p> <p>1) <u>IF</u> MFLCPR is below both limits, <u>THEN</u> initial Form 0225.</p> <p>2) <u>IF</u> MFLCPR is not below both limits, <u>THEN</u> proceed as follows:</p> <p>a) <u>IF</u> MFLCPR is above the Action Limit, <u>THEN</u> contact a Nuclear Engineer.</p> <p style="text-align: center;"><u>NOTE:</u> Reducing power with recirc flow will not decrease MFLCPR by a significant fraction and may increase MFLCPR. Rod pattern alteration will always be required to return MFLCPR to an acceptable value.</p> <p>b) <u>IF</u> MFLCPR exceeds the Tech Spec Limit, <u>THEN</u> immediately contact a Nuclear Engineer. <u>AND</u> take actions required by Tech Spec 3.11.C and 4 AWI-04.08.01 (EVENT NOTIFICATIONS).</p>	<p>Standard:</p> <p>Verifies MFLCPR is below the Action and Tech Spec limits and checks "YES" on Form 0225</p> <p>Cue: None</p> <p>Comments: None</p>

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Performance Objective	Standard
<p>STEP 6 (Procedure Part A, Step 2.d)</p> <p>NOTE: SRI 00-024, Operating With One Pressure Regulator In Service, provides justification for operating with one pressure regulator out-of-service.</p> <p>d. <u>IF</u> MPR or EPR is out of service, <u>THEN</u> verify that guidance for additional actions for Core Thermal Limits Monitoring in Operations Manuals C.4-B.05.09.A (ABNORMAL PROCEDURES – MAIN STEAM PRESSURE REGULATOR FAILURE CAUSING DECREASING PRESSURE) and C.4-B.05.09.B (ABNORMAL PROCEDURES – MAIN STEAM PRESSURE REGULATOR FAILURE CAUSING INCREASING PRESSURE) is followed.</p>	<p>Standard: Determines that MPR and EPR are not out of service.</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 7 (Procedure Part B, Step 3)</p> <p>Demand Official 3D Monitor to calculate the nodal power distribution, <u>AND</u> use the data for the fuel assembly that has its MAPRAT closest to 1.0.</p>	<p>Standard:</p> <p>Evaluates nodal power distribution to find the fuel assembly that has its MAPRAT closest to 1.0.</p> <p>Cue: None</p> <p>Comments:</p> <p>All three thermal limits are printed out when a 3D Monitor is requested.</p>
<p>STEP 8 “C” (Procedure Part B, Step 3)</p> <p>Record the following data for the most limiting fuel bundle on Form 0225:</p> <ul style="list-style-type: none"> a. The value of MAPRAT b. Core location of MAPRAT c. Percent rated core flow 	<p>Standard:</p> <p>Records data on Form 0225.</p> <ul style="list-style-type: none"> a. MAPRAT .880 b. Core location 35 – 14 - 5 c. % rated core flow 89.9% <p>Cue: None</p> <p>Comments: None</p>

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Performance Objective	Standard
<p>STEP 9 "C" (Procedure Part B, Step 5.a)</p> <p>Verify MAPRAT is below the Action Limit (0.97 times the Tech Spec Limit) and the Tech Spec Limit.</p> <p>a. <u>IF</u> MAPRAT is below both limits, <u>THEN</u> initial FORM 0207.</p>	<p>Standard:</p> <p>Verifies MAPRAT is below the Action and Tech Spec limits and checks "YES" on Form 0207.</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 10 (Procedure Part B, Step 5.b)</p> <p>b. <u>IF</u> MAPRAT is not below both limits, <u>THEN</u> proceed as follows:</p> <p>1) <u>IF</u> MAPRAT is above the Action Limit, <u>AND</u> below the Tech Spec Limit, <u>THEN</u> contact a Nuclear Engineer.</p> <p>2) <u>IF</u> MAPRAT exceeds the Tech Spec Limit, <u>THEN</u> immediately contact a Nuclear Engineer, <u>AND</u> take action required by Tech Spec 3.11.A and 4 AWI-04.08.01.</p> <p>NOTE: Reducing power with recirc flow will not decrease MAPRAT by a significant fraction and may increase MAPRAT. Rod pattern alterations will always be required to return MAPRAT to an acceptable value.</p>	<p>Standard: None</p> <p>Cue: None</p> <p>Comments: MAPRAT is acceptable, thus no action is required.</p>
<p>STEP 11 (Procedure Part B, Step 5.c)</p> <p>NOTE: SRI 00-024, Operating With One Pressure Regulator In Service, provides justification for operating with one pressure regulator out-of-service.</p> <p>c. <u>IF</u> either the MPR or the EPR is out of service, <u>THEN</u> verify that guidance for additional actions for Core Thermal Limits Monitoring in Operations Manuals C.4-B.05.09.A and C.4-B.05.09.B is followed.</p>	<p>Standard: None</p> <p>Cue: None</p> <p>Comments: MPR and EPR are not out of service.</p>

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Performance Objective	Standard
<p>STEP 12 (Procedure Part C, Step 6)</p> <p>Demand the Official 3D Monitor to calculate the nodal power distribution.</p>	<p>Standard:</p> <p>Evaluates nodal power distribution.</p> <p>Cue:</p> <p>Comments: All three thermal limits are printed out when a 3D Monitor is requested.</p>
<p>STEP 13 (Procedure Part C, Step 7)</p> <p>Scan the Periodic Log to find the highest MFLPD, <u>AND</u> note the MFLPD and the location in the core where it exists.</p>	<p>Standard:</p> <p>Scans Periodic Log and finds the highest MFLPD and its location in the core.</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 14 "C" (Procedure Part C, Step 8)</p> <p>Record the following data for the most limiting fuel bundle on Form 0208:</p> <p>a. The value of MFLPD.</p> <p>b. The location of MFLPD.</p>	<p>Standard:</p> <p>Records the data on Form 0208.</p> <p>a. MFLPD .990</p> <p>b. Location of MFLPD 35 – 12 - 5</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 15 (Procedure Part C, Step 9.a)</p> <p>Verify MFLPD is below Action Limit (0.97 times the Tech Spec Limit) and Tech Spec Limit.</p> <p>a. <u>IF</u> MFLPD is below both limits, <u>THEN</u> initial Form 0208</p>	<p>Standard:</p> <p>Determines MFLPD is not below both limits.</p> <p>Cue: None</p> <p>Comments: None</p>

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Performance Objective	Standard
<p>STEP 16 "C" (Procedure Part C, Step 9.b)</p> <p><u>IF</u> MFLPD is not below both limits, <u>THEN</u> proceeds as follows:</p> <ol style="list-style-type: none"> 1) <u>IF</u> MFLPD is above the Action Limit, <u>AND</u> below the Tech Spec Limit, <u>THEN</u> contact the Nuclear Engineer. 2) <u>IF</u> MFLPD exceeds the Tech Spec Limit, <u>THEN</u> immediately contact a Nuclear Engineer. <p style="text-align: center;">CAUTION</p> <p>Reducing power can cause an increase in MFLCPR or MAPRAT. If any bundle has a MAPRAT or MFLCPR greater than the corresponding Action Limit (0.97 for MAPRAT and 0.97 for MFLCPR), do not reduce power more than 3% prior to contacting a Nuclear Engineer.</p> <ol style="list-style-type: none"> a) Reduce power at the maximum allowable rate until MFLPD meets the Tech Spec Limit. b) Take actions required by Tech Spec 3.11.B. c) Inform personnel according to 4 AWI-04.08.01. <p style="text-align: center;">NOTE:</p> <p>A rapid estimate of the MFLPD can be made by assuming that a power decrease of 1% leads to a MFLPD decrease of 0.01.</p>	<p>Standard:</p> <p>Observes that MFLPD is above Action Limit but below the Tech Spec limit, and checks "NO" on Form 0208.</p> <ol style="list-style-type: none"> 1) Immediately contacts the Nuclear Engineer 2) Evaluates that power needs to be reduced approximately 3%. Notifies SS of required actions. <p>Cue:</p> <p>Another operator will decrease reactor power within acceptable limits.</p> <p>Comments: It is not expected for the candidates to make the required notifications.</p>
<p>STEP 17</p> <p>INFORM EVALUATOR THAT THE TASK HAS BEEN COMPLETED.</p>	<p>Standard:</p> <p>Operator informs evaluator that the task is completed.</p> <p>Cue: None</p> <p>Comments: DO NOT PROMPT!</p>

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JPM SET-UP	
<p>Simulator Setup: Not Applicable</p> <p>Copy of Operations Daily Log – Part E 3D Monitor printout. Place a red pencil on the Control Room Desk Have a copy of Form 3389 (EVENT NOTIFICATIONS) available.</p>	
<p>Initial Conditions: Reactor power has just been increased from 90% power to 98%, with the Recirc Flow Control system, per C.2-05 Load Following.</p>	
<p>Initiating Cues: You have been asked to perform the Core Thermal Limits Monitoring for MFLCPR, MAPRAT, and MFLPD, per Ops Man C.2-05, Section B.1, Parts A, B, and C, and determine any required actions if necessary. Record data on Forms 0207, 0208, and 0225, of Operations Daily Log 0000 – E. Report when the task is complete.</p>	

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PERFORMANCE INFORMATION

Performance Objective	Standard
<p>STEP 1</p> <p>Locates procedure C.2-05, "Power Operation".</p>	<p>Standard:</p> <p>Locates appropriate procedure.</p> <p>Cue: None</p> <p>Comments: Parts A, B, and C may be performed in any order.</p>
<p>STEP 2 (Procedure Part A, Step 1)</p> <p><u>IF</u> process computer is not anticipated to be available during a 24 hour period, <u>THEN</u> contact a Nuclear Engineer to arrange determination of MCPR.</p>	<p>Standard: None</p> <p>Cue: None</p> <p>Comments: Process Computer is available.</p>
<p>STEP 3 "C" (Procedure Part A, Step 2.a)</p> <p>When 3D Monitor computer is available, perform the following:</p> <p>e. Demand Official 3D Monitor to calculate core nodal power distribution, <u>AND</u> use MCPR data for the fuel assemblies that are closest to the limiting CPR.</p> <p align="center">NOTE 4: The most limiting MCPR is edited as MFLCPR on the Periodic Log</p> <p align="center">NOTE 5: MFLCPR is the Core Maximum ratio of power and flow dependent CPR limit to the bundle's actual CPR.</p>	<p>Standard:</p> <p>Cue: Provide candidate with a copy of 3D Monitor Edit</p> <p>Comments: A 3D Monitor edit cannot be demanded on the simulator.</p>
<p>STEP 4 "C" (Procedure Part A, Step 2.b)</p> <p>f. Record the following data for the most limiting fuel bundle on Form 0225:</p> <p>1) The value of MFLCPR 2) Core location of MFLCPR 3) Percent rated core flow</p>	<p>Standard:</p> <p>Records data on Form 0225.</p> <p>1) The value of MFLCPR .883 2) Core location of MFLCPR 13 - 32 3) Percent rated core flow 89.9%</p> <p>Cue: None</p> <p>Comments: None</p>

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Performance Objective	Standard
<p>STEP 5 "C" (Procedure Part A, Step 2.c)</p> <p>g. Verify MFLCPR is below the Action Limit (0.97 times the Tech Spec Limit) and the Tech Spec Limit.</p> <p>3) <u>IF</u> MFLCPR is below both limits, <u>THEN</u> initial Form 0225.</p> <p>4) <u>IF</u> MFLCPR is not below both limits, <u>THEN</u> proceed as follows:</p> <p>c) <u>IF</u> MFLCPR is above the Action Limit, <u>THEN</u> contact a Nuclear Engineer.</p> <p style="text-align: center;">NOTE: Reducing power with recirc flow will not decrease MFLCPR by a significant fraction and may increase MFLCPR. Rod pattern alteration will always be required to return MFLCPR to an acceptable value.</p> <p>d) <u>IF</u> MFLCPR exceeds the Tech Spec Limit, <u>THEN</u> immediately contact a Nuclear Engineer. <u>AND</u> take actions required by Tech Spec 3.11.C and 4 AWI-04.08.01 (EVENT NOTIFICATIONS).</p>	<p>Standard:</p> <p>Verifies MFLCPR is below the Action and Tech Spec limits and checks "YES" on Form 0225</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 6 (Procedure Part A, Step 2.d)</p> <p>NOTE: SRI 00-024, Operating With One Pressure Regulator In Service, provides justification for operating with one pressure regulator out-of-service.</p> <p>h. <u>IF</u> MPR or EPR is out of service, <u>THEN</u> verify that guidance for additional actions for Core Thermal Limits Monitoring in Operations Manuals C.4-B.05.09.A (ABNORMAL PROCEDURES – MAIN STEAM PRESSURE REGULATOR FAILURE CAUSING DECREASING PRESSURE) and C.4-B.05.09.B (ABNORMAL PROCEDURES – MAIN STEAM PRESSURE REGULATOR FAILURE CAUSING INCREASING PRESSURE) is followed.</p>	<p>Standard: Determines MPR and EPR are not out of service.</p> <p>Cue: None</p> <p>Comments: None</p>

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Performance Objective	Standard
<p>STEP 7 (Procedure Part B, Step 3)</p> <p>Demand Official 3D Monitor to calculate the nodal power distribution, <u>AND</u> use the data for the fuel assembly that has its MAPRAT closest to 1.0.</p>	<p>Standard:</p> <p>Evaluates nodal power distribution to find the fuel assembly that has its MAPRAT closest to 1.0.</p> <p>Cue: None</p> <p>Comments: All three thermal limits printout on 3D Monitor.</p>
<p>STEP 8 "C" (Procedure Part B, Step 3)</p> <p>Record the following data for the most limiting fuel bundle on Form 0225:</p> <ul style="list-style-type: none"> a. The value of MAPRAT b. Core location of MAPRAT c. Percent rated core flow 	<p>Standard:</p> <p>Records data on Form 0225.</p> <ul style="list-style-type: none"> a. MAPRAT .880 c. Core location 35 – 14 - 5 c. % rated core flow 89.9% <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 9 "C" (Procedure Part B, Step 5.a)</p> <p>Verify MAPRAT is below the Action Limit (0.97 times the Tech Spec Limit) and the Tech Spec Limit.</p> <p>d. <u>IF</u> MAPRAT is below both limits, <u>THEN</u> initial FORM 0207.</p>	<p>Standard:</p> <p>Verifies MAPRAT is below the Action and Tech Spec limits and checks "YES" on Form 0207</p> <p>Cue: None</p> <p>Comments: None</p>

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Performance Objective	Standard
<p>STEP 10 (Procedure Part B, Step 5.b)</p> <p>e. <u>IF</u> MAPRAT is not below both limits, <u>THEN</u> proceed as follows:</p> <p>3) <u>IF</u> MAPRAT is above the Action Limit, <u>AND</u> below the Tech Spec Limit, <u>THEN</u> contact a Nuclear Engineer.</p> <p>4) <u>IF</u> MAPRAT exceeds the Tech Spec Limit, <u>THEN</u> immediately contact a Nuclear Engineer, <u>AND</u> take action required by Tech Spec 3.11.A and 4 AWI-04.08.01.</p> <p style="text-align: center;">NOTE: Reducing power with recirc flow will not decrease MAPRAT by a significant fraction and may increase MAPRAT. Rod pattern alterations will always be required to return MAPRAT to an acceptable value.</p>	<p>Standard: None</p> <p>Cue: None</p> <p>Comments: MAPRAT is acceptable and no action is required.</p>
<p>STEP 11 (Procedure Part B, Step 5.c)</p> <p>NOTE: SRI 00-024, Operating With One Pressure Regulator In Service, provides justification for operating with one pressure regulator out-of-service.</p> <p>f. <u>IF</u> either the MPR or the EPR is out of service, <u>THEN</u> verify that guidance for additional actions for Core Thermal Limits Monitoring in Operations Manuals C.4-B.05.09.A and C.4-B.05.09.B is followed.</p>	<p>Standard: None</p> <p>Cue: None</p> <p>Comments: MPR and EPR are not out of service.</p>
<p>STEP 12 (Procedure Part C, Step 6)</p> <p>Demand the Official 3D Monitor to calculate the nodal power distribution.</p>	<p>Standard:</p> <p>Evaluates nodal power distribution.</p> <p>Cue:None</p> <p>Comments: All three thermal limits printout on 3 D Monitor.</p>

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Performance Objective	Standard
<p>STEP 13 (Procedure Part C, Step 7)</p> <p>Scan the Periodic Log to find the highest MFLPD, <u>AND</u> note the MFLPD and the location in the core where it exists.</p>	<p>Standard:</p> <p>Scans Periodic Log and finds the highest MFLPD and its location in the core.</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 14 "C" (Procedure Part C, Step 8)</p> <p>Record the following data for the most limiting fuel bundle on Form 0208:</p> <p>c. The value of MFLPD.</p> <p>b. The location of MFLPD.</p>	<p>Standard:</p> <p>Records the data on Form 0208.</p> <p>a. The value of MFLPD. 1.010</p> <p>b. The location of MFLPD. 35 – 12 - 5</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 15 (Procedure Part C, Step 9.a)</p> <p>Verify MFLPD is below Action Limit (0.97 times the Tech Spec Limit) and Tech Spec Limit.</p> <p>b. <u>IF</u> MFLPD is below both limits, <u>THEN</u> initial Form 0208</p>	<p>Standard:</p> <p>Determines MFLPD is not below both limits.</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 16 "C" (Procedure Part C, Step 9.b.1)</p> <p><u>IF</u> MFLPD is not below both limits, <u>THEN</u> proceeds as follows:</p> <p>3) <u>IF</u> MFLPD is above the Action Limit, <u>AND</u> below the Tech Spec Limit, <u>THEN</u> contact the Nuclear Engineer.</p>	<p>Standard: None</p> <p>Cue: None</p> <p>Comments: Determines MFLPD is above Action Limit and the Tech Spec Limit and checks "NO" on Form 0208.</p>
<p>STEP 17 (Procedure Part C, Step 9.b.2)</p> <p>4) <u>IF</u> MFLPD exceeds the Tech Spec Limit, <u>THEN</u> immediately contact a Nuclear Engineer.</p>	<p>Standard:</p> <p>Immediately contacts the Nuclear Engineer.</p> <p>Cue: None</p> <p>Comments: None</p>

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Performance Objective	Standard
<p>STEP 18 "C"(Proc Part C, Step 9.b.2.a)</p> <p style="text-align: center;"><u>CAUTION</u></p> <p>Reducing power can cause an increase in MFLCPR or MAPRAT. If any bundle has a MAPRAT or MFLCPR greater than the corresponding Action Limit (0.97 for MAPRAT and 0.97 for MFLCPR), do not reduce power more than 3% prior to contacting a Nuclear Engineer.</p> <p>a) Reduce power at the maximum allowable rate until MFLPD meets the Tech Spec Limit.</p>	<p>Standard:</p> <p>Identifies that power needs to be reduced by approximately 1% and may reduce power 5% to get below the action limit.</p> <p>Cue (as applicable for action taken above): Reactor power has been reduced approximately 1% (5%) and MFLPD is now at 1.000 (.960).</p> <p>Comments: None</p>
<p>STEP 19 "C"(Proc Part C, Step 9.b.2.b & c)</p> <p>b) Take actions required by Tech Spec 3.11.B.</p> <p>c) Inform personnel according to 4 AWI-04.08.01.</p> <p style="text-align: center;"><u>NOTE:</u></p> <p>A rapid estimate of the MFLPD can be made by assuming that a power decrease of 1% leads to a MFLPD decrease of 0.01.</p>	<p>Standard:</p> <p>Describes actions that need to be taken per Tech Spec 3.11.B.</p> <p>If at any time during operation it is determined that the limiting value for LHGR is being exceeded, action shall be initiated within 15 minutes to restore operation to within the prescribed limits. Surveillance and corresponding action shall continue until reactor operation is within the prescribed limits. If the LHGR is not returned to within the prescribed limits within 2 hours, reduce thermal power to less than 25% within the next 4 hours.</p> <p>Describes who needs to be notified per 4 AWI-04.08.01.</p> <p>Part A of Form 3389 (EVENT NOTIFICATIONS)</p> <p>General Superintendent Operations General Superintendent Engineering</p> <p>May include the following as a courtesy call:</p> <p>Plant Manager Site Vice President Superintendent Nuclear Engineering</p> <p>Cue: None</p> <p>Comments: None</p>

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Performance Objective	Standard
<p>STEP 20</p> <p>INFORM EVALUATOR THAT THE TASK HAS BEEN COMPLETED.</p>	<p>Standard:</p> <p>Operator informs evaluator that the task is completed.</p> <p>Cue: None</p> <p>Comments: DO NOT PROMPT!</p>

3MONTICELLO NUCLEAR GENERATING PLANT		JPM-3139-001
TITLE:	CONTROL ROOM PANEL WALKDOWN	Revision 0
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JOB PERFORMANCE MEASURE

SRO: _____ SRO/RO: X SRO/RO/NLO: _____ NLO: _____

Plant Reference: Procedure Checklist 3139, Rev. 24

Task Standard: Perform a Control Room Shift Turnover Checklist and panel walkdown and identify any discrepancies.

K/A Reference: Generic Task No.: 2.1.31 Rating RO/SRO: 4.2/3.9

Recent Events: None

Probabilistic Risk Assessment Human Error: None

Monticello Specific Task List Reference: CR298.103

METHOD OF TESTING		TIME FOR COMPLETION
Simulator: <u> X </u> In-Plant: _____	Normal: <u> X </u> Alternate Path: _____	Estimated Time to Complete: <u> 30 min. </u> Maximum Time to Complete: <u> 60 min. </u> Time Critical: _____ Yes <u> X </u> No

Prepared By:	Date:
Reviewed By:	Date:
Shift Supv/Shift Mgr Review By:	Date:
Approved By:	Date:

3MONTICELLO NUCLEAR GENERATING PLANT		JPM-3139-001
TITLE:	CONTROL ROOM PANEL WALKDOWN	Revision 0
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JPM SET-UP

Simulator Setup:

Place simulator in "RUN" until core thermal power updates.
 IOS the "green" light "off" for #11 Core Spray Pump.
 Place the RCIC Flow Control in "manual".
 Place the Master FW controller in "manual".
 Place the RFP High Level Trip "Normal/Bypass" switch in the "Bypass" position.
 EPR and MPR both in control (married).

Initial Conditions:

The time is 1800. The plant is operating at 100% power with all equipment operating as expected.

Initiating Cues:

As the BOP operator, you are to walkdown the control room panels C03 through C08 and complete page #1 of the CONTROL ROOM SHIFT TURNOVER CHECKLIST, Form 3139, and identify any discrepancies discovered during the panel walkdown. The lined out equipment status items on Form 3139 are not available in the simulator. (Provide Candidate a copy of Form 3139 (CONTROL ROOM SHIFT TURNOVER CHECKLIST))

3MONTICELLO NUCLEAR GENERATING PLANT		JPM-3139-001
TITLE:	CONTROL ROOM PANEL WALKDOWN	Revision 0
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PERFORMANCE INFORMATION

Performance Objective	Standard
<p>STEP 1 (CHECKLIST 3139)</p> <p>Operator completes "parameter logging" on Form 3139.</p>	<p>Standard:</p> <p>Operator completes "parameter logging", no discrepancies identified.</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 2 (CHECKLIST 3139)</p> <p>Operator completes INSTRUMENTATION section of Form 3139.</p>	<p>Standard:</p> <p>Operator completes INSTRUMENTATION section, no discrepancies identified.</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 3 "C" (CHECKLIST 3139)</p> <p>Operator completes CONTROLLER MODES section of Form 3139.</p>	<p>Standard:</p> <p>Operator Completes CONTROLLER MODES section, and identifies the following discrepancy:</p> <p>1) RFP level Trip in BYPASS</p> <p>Cue: As the CRS, when notified of any of the above discrepancy, inform candidate that another operator will correct the condition.</p> <p>Comments: Candidate must identify the above listed discrepancy.</p>

3MONTICELLO NUCLEAR GENERATING PLANT		JPM-3139-001
TITLE:	CONTROL ROOM PANEL WALKDOWN	Revision 0
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Performance Objective	Standard
<p>STEP 4 "C" (CHECKLIST 3139)</p> <p>Operator completes EMERGENCY SYSTEMS section of Form 3139.</p>	<p>Standard:</p> <p>Operator completes EMERGENCY SYSTEMS section, and identifies the following discrepancies.</p> <ol style="list-style-type: none"> 1. #11 Core Spray pump light indication for 152-505 is out. Notifies CRS . 2. RCIC Flow Controller in manual and notifies CRS <p>Cue: When notified for each of the above discrepancies, Inform the candidate that the applicable system is INOP and the problem will be investigated.</p> <p>Comment: Candidate must identify both discrepancies.</p>
<p>STEP 9</p> <p>INFORM EVALUATOR THAT THE TASK HAS BEEN COMPLETED.</p>	<p>Standard:</p> <p>Operator informs evaluator that the task is completed.</p> <p>Cue: None</p> <p>Comments: DO NOT PROMPT!</p>

JOB PERFORMANCE MEASURE

SRO: X SRO/RO: SRO/RO/NLO: NLO:

Plant Reference: 4 AWI-02.02.05, Rev 9; 0255-04-IA-1, Rev 56

Task Standard: Evaluate Temporary Change to Procedure and Determine Acceptability

K/A Reference: 2.2 Task No: 2.2.11 Rating RO/SRO: 2.5/3.4

Recent Events: None

Probabilistic Risk Assessment Human Error: None

Monticello Specific Task List Reference: SS007.002 CR299.120

METHOD OF TESTING

TIME FOR COMPLETION

Simulator: X

Normal:

Estimated Time to Complete: 5 Min

In-Plant: X

Alternate
Path: X

Maximum Time to Complete: 10 Min

Time Critical: Yes X No

Prepared By:

Date:

Reviewed By:

Date:

Shift Supv/Shift Mgr Review By:

Date:

Approved By:

(Supt Ops Trng) Date:

JPM SET-UP

Plant Setup:

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Fill in Test 0255-04-IA-1 as follows:

- D Sign Shift Supv approval to commence.
- D Reason for Performing: No. 1 – Quarterly
- D Complete in conjunction with:
 - a. No
 - b. No
 - c. No
- D Fill in prerequisites: Avg Torus Water temp 80/80°F A&B
RWCU RWP-22
East HCU RWP – 51

Initial prerequisite regarding status of Core Spray valve position
- D Fill in numbers and initials for STEPS 1 through 56a.
- D Cross off STEP 56b, initial and date for system engineer supervisor and initial for system engineer.
- D Write in Comments:

Delete STEP 56b to allow completion of the operability test and return 11 RHR pump to service. The RHR pump minimum flow valve is NOT required to open to establish the required pump flow. Also, RHR pump minimum flow valve is not required to open for the RHR LPCI mode to perform its intended function.

Initial Conditions:

The Reactor is operating at 100% power with all systems normal. The RHR System Operability Test (No. 0255-04-IA-1) for Loop A is being performed to prove RHR system operability. No. 11 RHR Pump minimum flow valve failed to open after the pump was started. The RHR System Engineer is in the Control Room.

Initiating Cues:

The Control Room Operator brings you, the Duty Control Room Supv, test procedure No. 0255-04-IA-1 with a request for a temporary change to the test procedure. The temporary change has been requested by the RHR System Engineer. You are to review the change and either approve or disapprove the change as appropriate.

MONTICELLO NUCLEAR GENERATING PLANT PERFORMANCE INFORMATION		JPM-4AWI-02.02.05-
Performance Objective	Standard	
STEP 1 (4 AWI-02.02.05, STEP 4.1.4)	Standard:	Page 3 of 5
Determines that a permanent procedure change is not desired. Temporary changes SHALL NOT change the scope or intent of an approved procedure.	Evaluates the change request against the requirements delineated in 4AWI-02.02.05. Cue: None Comments: It is not necessary for the candidate to refer directly to 4 AWI-02.02.05. If knowledge level is sufficient, the candidate may immediately disapprove the request based upon the reasons in STEPS 4 and 5.	
STEP 2 (4 AWI-02.02.05, STEP 4.1.5) Determines that the procedure being changed is not a modification procedure.	Standard: Evaluates the change request against the requirements delineated in 4 AWI-02.02.05. Cue: None Comments: None	
STEP 3 (4 AWI-02.02.05, STEP 4.1.7) Determines that the requested change is not of an emergency nature.	Standard: Evaluates the change request against the requirements delineated in 4 AWI-02.02.05. Cue: None Comments: None	
STEP 4 (B.03.04-05 PRECAUTION) Determines that the requested change would place the No. 11 RHR pump in an operational condition that could result in pump damage due to prolonged operation against a shutoff head following LPCI initiation with the reactor vessel at high pressure.	Standard: Makes reference to possible pump damage due to operation without minimum flow when giving reason for disapproval of the Temporary Procedure Change Request. Cue: None Comments: None	

MONTICELLO NUCLEAR GENERATING PLANT

JPM-4AWI-02.02.05-

Intentionally Blank

TITLE: TEMPORARY CHANGE REQUEST

Revision 7a

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MONTICELLO NUCLEAR GENERATING PLANT		JPM-0000-B-001
TITLE:	DAILY JET PUMP OPERABILITY	Revision 0
		Page 1 of 8

JOB PERFORMANCE MEASURE

SRO: _____ SRO/RO: X SRO/RO/NLO: _____ NLO: _____

Plant Reference: 0000-B, Test 0133, Rev. 84

Task Standard: Perform the Daily Jet Pump Operability test.

K/A Reference: Generic Task No.: 2.2.12 Rating RO/SRO: 3.0/3.4

Recent Events: None

Probabilistic Risk Assessment Human Error: None

Monticello Specific Task List Reference: CR202.101

METHOD OF TESTING		TIME FOR COMPLETION
Simulator: <u> X </u> In-Plant: _____	Normal: _____ Alternate Path: <u> X </u> _____	Estimated Time to Complete: <u> 15 min. </u> Maximum Time to Complete: <u> 25 min. </u> Time Critical: _____ Yes <u> X </u> No

Prepared By:	Date:
Reviewed By:	Date:
Shift Supv/Shift Mgr Review By:	Date:
Approved By:	Date:

MONTICELLO NUCLEAR GENERATING PLANT		JPM-0000-B-001
TITLE:	DAILY JET PUMP OPERABILITY	Revision 0
		Page 2 of 8

JPM SET-UP	
<p>Simulator Setup:</p> <p>100% power.</p> <p>IOS Meter 2-184-23A to indicate 3200 volts.</p> <p>IOS Meter 2-184-23B to indicate 3400 volts.</p> <p>Verify A and B Recirc Pump speeds are at 82.0% and 83.0% respectively.</p> <p>Place a clean set of Operations Daily Log, parts: A, B, D, E, H, G, J, on a clip board.</p> <p>A Copy of 0000-B, Step 6 with the values for Raw dp filled in for both Loop A and B.</p> <p>Place simulator in 'RUN" for 15 minutes prior to exam to allow SPDS to update Core Thermal Power.</p>	
<p>Initial Conditions:</p> <p>The plant is operating at 100% power with all systems operating as expected.</p>	
<p>Initiating Cues:</p> <p>You have been directed to perform the Daily Jet Pump Operability Test per 0000-B, "Operations Daily Log-Part B", Test 0133.</p>	

MONTICELLO NUCLEAR GENERATING PLANT		JPM-0000-B-001
TITLE:	DAILY JET PUMP OPERABILITY	Revision 0
		Page 3 of 8

PERFORMANCE INFORMATION

Performance Objective	Standard
<p>STEP 1</p> <p>Locates procedure 0000-B, "Operations Daily Log-Part B", Test 0133.</p>	<p>Standard: Locates appropriate procedure.</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 2 "C" (Procedure Step 4)</p> <p><u>IF</u> the Reactor is in Run, <u>AND</u> there is recirculation flow, <u>THEN</u> obtain and record the values for the item described in the matrices below and perform the following:</p>	<p>Standard:</p> <p>Records the following values in the matrices</p> <ul style="list-style-type: none"> • RVP501 99.7% • SI-2-184-16A 82.2% • SI-2-184-16B 82.0% • FI-2-159A 25.5 gpm • FI-2-159B 25.7 gpm • FI-2-3-92B 25.5 Mlbm/hr • FI-2-3-92A 25.5 Mlbm/hr • (25.5/28.8) X 100 = 88.5% • (25.5/28.8) X 100 = 88.5% • 88.5/82.2 = 1.076 • 88.5/82.2 = 1.076 <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 3 "C" (Procedure Step 4.a)</p> <p>On Figure 17, Recirculation Pump Flow vs Speed, of Ops Man B.01.04-06 (REACTOR RECIRCULATION SYSTEM – FIGURES) plot points for values of variables A vs C and variables B vs D.</p> <p>1) Does point for A vs C fall in ACCEPTABLE RANGE? Yes No</p> <p>2) Does point for B vs D fall in ACCEPTABLE RANGE? Yes No</p>	<p>Standard:</p> <p>Checks "YES" to Answer Questions 1 and 2</p> <p>Cue: None</p> <p>Comments: None</p>

MONTICELLO NUCLEAR GENERATING PLANT		JPM-0000-B-001
TITLE:	DAILY JET PUMP OPERABILITY	Revision 0
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Performance Objective	Standard
<p>STEP 4 "C" (Procedure Step 4.b)</p> <p>On Figure 15 Loop A: Jet Pump Loop Flow/Pump Speed, of Ops Man B.01.04-06 plot point for value of variables J vs A.</p> <p>1) Does point for J vs A fall in ACCEPTABLE RANGE? Yes No</p>	<p>Standard:</p> <p>Checks "YES" to Answer Question</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 5 "C" (Procedure Step 4.c)</p> <p>On Figure 16 Loop B: Jet Pump Loop Flow/Pump Speed, of Ops Man B.01.04-06 plot point for value of variables K vs B.</p> <p>2) Does point for K vs B fall in ACCEPTABLE RANGE? Yes No</p>	<p>Standard:</p> <p>Checks "YES" to Answer Question</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 6 (Procedure Step 4.d)</p> <p><u>IF</u> any question is checked No, <u>THEN</u> notify Shift Supervisor, <u>AND</u> perform Procedure 0442 (SPECIAL JET PUMP OPERABILITY TEST).</p>	<p>Standard: None</p> <p>Cue: None</p> <p>Comments: All questions were checked Yes.</p>
<p>STEP 7 "C" (Procedure Step 5.a)</p> <p><u>IF</u> the Reactor is in Run, <u>AND</u> there is recirculation flow, <u>THEN</u> perform the following:</p> <p>a. From FPR-2-3-95 (C-04), Core dP Red Pen, record dP: _____</p>	<p>Standard:</p> <p>Records the following:</p> <ul style="list-style-type: none"> • 14.1 psid <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 8 "C" (Procedure Step 5.b)</p> <p>b. Record the square root of Core dP (from a. above): = _____</p>	<p>Standard:</p> <p>Records square root of dP.</p> <ul style="list-style-type: none"> • 3.75 <p>Cue: None</p> <p>Comments: None</p>

MONTICELLO NUCLEAR GENERATING PLANT		JPM-0000-B-001
TITLE:	DAILY JET PUMP OPERABILITY	Revision 0
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Performance Objective	Standard
<p>STEP 9 "C" (Procedure Step 5.c)</p> <p>c. From CRT mimic obtain and record Core Flow: _____ x 10⁶ lb/hr</p>	<p>Standard:</p> <p>Records Core Flow</p> <ul style="list-style-type: none"> • 51.7 <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 10 "C" (Procedure Step 5.d)</p> <p>Calculate and record 100 x (c.÷57.6x10⁶ lb/hr): = _____ % rated Core Flow</p>	<p>Standard:</p> <p>Calculates and records % rated Core Flow</p> <ul style="list-style-type: none"> • 89.756% <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 11 "C" (Procedure Step 5.e)</p> <p>On Figure 20 Core Plate dP vs Core Flow, of Ops Man B.01.04-06 plot point for value of b. vs d. above.</p> <p>1) Does point for b. vs d. fall in ACCEPTABLE RANGE? Yes No</p> <p>2) <u>IF</u> No, <u>THEN</u> notify Shift Supervision to determine actions and notifications.</p>	<p>Standard:</p> <p>Checks "YES" to Answer Question</p> <p>Cue: None</p> <p>Comments: None</p>

MONTICELLO NUCLEAR GENERATING PLANT		JPM-0000-B-001
TITLE:	DAILY JET PUMP OPERABILITY	Revision 0
		Page 6 of 8

Performance Objective	Standard
<p>STEP 12 "C" (Procedure Step 6.a)</p> <p>NOTE: STEP 6 collects operational data on Jet Pump dP to establish the expected normal range of deviation.</p> <p><u>IF</u> the Reactor is in Run, <u>AND</u> there is recirculation flow, <u>THEN</u> From FR-7288, JET PUMP FLOW RECORDER (Panel C-38), record individual Jet Pump and loop average dPs in the matrix below and perform the following:</p> <p>a. For each loop divide each individual Jet Pump raw dP by its Loop Average dP and record in matrix.</p>	<p>Standard:</p> <p>Records individual Jet Pump and loop average dPs, and performs action in step 6a</p> <p>Cue:</p> <p>Here are the current Jet Pump Raw dp readings.</p> <p>Comments:</p> <p>The data for Jet Pump Raw dp is not available on the simulator. When candidate explains to evaluator that these readings are in Cable Spreading, <u>THEN</u> provide copy of the jet pump raw dp values.</p>
<p>STEP 13 (Procedure Step 6.b)</p> <p>c. <u>IF</u> the Reactor is in RUN, <u>AND</u> recirc pump speed \leq 60%, <u>THEN</u> the jet pump dP SHALL be monitored and evaluated every 24 hours until such time as evaluation at the higher pump speed is made. Complete test 0442 (SPECIAL JET PUMP OPERABILITY TEST) and notify System Engineer for evaluation of this condition.</p>	<p>Standard</p> <p>Candidate determines step 6.b is not applicable.</p> <p>Cue: None</p> <p>Comments: Recirc pump speed is > 60%.</p>

MONTICELLO NUCLEAR GENERATING PLANT		JPM-0000-B-001
TITLE:	DAILY JET PUMP OPERABILITY	Revision 0
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Performance Objective	Standard
<p>STEP 14 "C" (Procedure Step 7.a & b)</p> <p><u>IF</u> the Reactor is in Run, <u>AND</u> Recirc Pump speed (SI-2-184-16A&B on C-04) >50%, <u>THEN</u> perform the following:</p> <p>a. Record 2-184-23A (C-04), A Pump Volts, indication: _____ Volts</p> <p>b. Record 2-184-23B (C-04), B Pump Volts, indication: _____ Volts</p>	<p>Standard:</p> <p>Records Pump Volts for both pumps as follows:</p> <ul style="list-style-type: none"> • 2-184-23A 3200 • 2-184-23B 3400 <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 15 "C" (Procedure Step 7.c & d)</p> <p>b. Record SI-2-184-16A, Recirc Pump 11 % Speed, indication: _____ %</p> <p>d. Record SI-2-184-16B, Recirc Pump 12 % Speed, indication: _____ %</p>	<p>Standard:</p> <p>Records Recirc Pump speed as follows:</p> <ul style="list-style-type: none"> • SI-2-184-16A 82.0 to 83.0 • SI-2-184-16B 82.0 to 83.0 <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 16 "C" (Procedure Step 7.e)</p> <p>e. Record result of a. ÷ c.: = _____ Volts/% Speed for 11 (35.5 – 40.5)</p>	<p>Standard:</p> <p>Records result of a. ÷ c.</p> <ul style="list-style-type: none"> • 38.5 to 39.02 <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 17 "C" (Procedure Step 7.f)</p> <p>f. Record result of b. ÷ d.: = _____ Volts/% Speed for 12 (35.5 – 40.5)</p>	<p>Standard:</p> <p>Records result of b. ÷ d.</p> <ul style="list-style-type: none"> • 40.96 to 41.46 <p>Cue: None</p> <p>Comments: None</p>

MONTICELLO NUCLEAR GENERATING PLANT		JPM-0000-B-001
TITLE:	DAILY JET PUMP OPERABILITY	Revision 0
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Performance Objective	Standard
<p>STEP 18 "C" (Procedure Step 7.g)</p> <p>g. <u>IF</u> either acceptance criteria is not met, <u>THEN</u> on Panel C-21 from TR-2-184-26, RECIRC MG SET WINDING TEMP, and TR-2-2-31, RECIRC PUMP AND MTR TEMP, verify MG Set generator and Recirc pump motor winding temperatures are normal and not unexpectedly trending up, <u>AND</u> notify Shift Supervision to determine actions and notifications.</p>	<p>Standard:</p> <p>Determines the following:</p> <ul style="list-style-type: none"> • Acceptance criteria for #11 Recirc pump is met. • Acceptance criteria for #12 Recirc pump is <u>not</u> met. <p>And then monitors Panel C-21 from TR-2-184-26, RECIRC MG SET WINDING TEMP, and TR-2-2-31, RECIRC PUMP AND MTR TEMP, to verify #12 MG Set generator and #12 Recirc pump motor winding temperatures are normal and not unexpectedly trending up, and notifies Shift Supervision.</p> <p>Cue: Acknowledge Shift Supervision notification.</p> <p>Comments: Temperatures are normal</p>
<p>STEP 19</p> <p>INFORM EVALUATOR THAT THE TASK HAS BEEN COMPLETED.</p>	<p>Standard:</p> <p>Operator informs evaluator that the task is completed.</p> <p>Cue: None</p> <p>Comments: DO NOT PROMPT!</p>

MONTICELLO NUCLEAR GENERATING PLANT		JPM-4AWI-04.04.03-
TITLE:	INITIATE JUMPER BYPASS FORM	Revision 6a
		Page 1 of 7

JOB PERFORMANCE MEASURE		
SRO: <u> X </u>	SRO/RO: <u> </u>	SRO/RO/NLO: <u> </u> NLO: <u> </u>
Plant Reference: 4 AWI-04.04.03, Rev 17; B.03.02-05, Rev 17; Jumper Bypass Form 3034, Rev 20		
Task Standard: Initiate Jumper Bypass Form		
K/A Reference: Generic		Task No: 2.2.11 Rating RO/SRO: 3.7/3.8
Recent Events: None		
Probabilistic Risk Assessment Human Error: None		
Monticello Specific Task List Reference: CR299.128 SS003.004		
METHOD OF TESTING		TIME FOR COMPLETION
Simulator: <u> X </u>	Normal: <u> X </u>	Estimated Time to Complete: <u> 10 Min </u>
In-Plant: <u> X </u>	Alternate Path: <u> </u>	Maximum Time to Complete: <u> 20 Min </u>
		Time Critical: <u> </u> Yes <u> X </u> No

Prepared By:	Date:
Reviewed By:	Date:
Shift Supv/Shift Mgr Review By:	Date:
Approved By:	(Supt Ops Trng) Date:
JPM SET-UP	
Simulator/Plant Setup:	

MONTICELLO NUCLEAR GENERATING PLANT		JPM-4AWI-04 04 03-
TITLE:	INITIATE JUMPER BYPASS FORM	is required to complete Revision 6a
The operator is not required to perform the steps in order, but is required to complete all steps.		Page 2 of 7

Provide blank Form 3034 to SRO per STEP 1, if requested.

Initial Conditions:

You are the Duty Control Room Supervisor. A packing leak has developed on HPCI Valve MO-2034 (HPCI INBOARD STEAM LINE ISOLATION). It has been decided that MO-2034 will be electrically backseated in an attempt to stop the steam leak per procedure B.03.02-05.H.2. A bypass control is needed to complete the procedure.

Initiating Cues:

Properly document the hanging of the jumper as required in procedure B.03.02-05.H.2.

MONTICELLO NUCLEAR GENERATING PLANT PERFORMANCE INFORMATION		JPM-4AWI-04.04.03-
Performance Objective	Standard	
STEP 1		Page 3 of 7
<p>Locate the following procedures:</p> <ul style="list-style-type: none"> a. 4 AWI-04.04.03 (BYPASS CONTROL) section 4.2, "OC Reviewed Procedures". b. B.03.02-05.H.2, (Backseating HPCI Steam Supply Motor-Operated Valves). 	<p>Locates appropriate procedures.</p> <p>Cue: None</p> <p>Comments: None</p>	
<p>STEP 2 (Proc STEP 4.2.1)</p> <p>OC reviewed procedures are those procedures for which OC revision review is required.</p>	<p>Standard:</p> <p>Determines that B.03.02-05 is an OC reviewed procedure and proceeds to the next step.</p> <p>Cue: None</p> <p>Comments: None</p>	
<p>STEP 3 (Proc STEP 4.2.2)</p> <p>Reviews Step 4.2.2.</p>	<p>Standard:</p> <p>Determines that a 10CFR50.59 screening is not required for B.03.02-05.</p> <p>Cue: None</p> <p>Comments: None</p>	
<p>STEP 4 "C" (Proc STEP 4.2.3)</p> <p>A Form 3034 is not required for OC reviewed procedures provided:</p> <ul style="list-style-type: none"> A. The procedure contains documented independent verification of installation and removal of the bypass device. <p>For Operations Manual Procedures, documentation may be provided by a log entry as noted in Section 4.3.7 of 4 AWI-04.04.02 (EQUIPMENT POSITIONING, WITNESS CHECK, AND INDEPENDENT VERIFICATION METHODS).</p> <ul style="list-style-type: none"> B. The procedure contains a Shift 	<p><u>Form 3034 method used</u></p> <p>Standard:</p> <p>Operator uses form 3034 to document Jumper/Bypass. If this method is used, then proceed to STEP 6.</p> <p>Cue: None</p> <p>Comments:</p> <p>Provide individual a copy of Form 3034</p> <p><u>Log entry method used</u></p> <p>Standard:</p> <p>Documentation may be provided by a log entry. If this method is used, then operator completes log entry per 4 AWI-04.04.02, Section 4.3.7.E.</p> <p>Cue:</p> <p>Provide paper for operator to perform log entry and</p>	

MONTICELLO NUCLEAR GENERATING PLANT		IPM-4AWI-04.04.03- Revision 6a
TITLE:	INITIATE JUMPER BYPASS FORM	Page 4 of 7
<p>Specialist sign-off for approval to initiate the procedure. Shift Supervisor sign-off is not required for for Operations Manual Procedures.</p> <p>C. The bypass is not installed for more than 7 days.</p>	<p>Inform the operator that the evaluator will perform independent verification.</p> <p>Comments: None</p>	
<p>STEP 5 "C"</p> <p>If log entry method is used, inform evaluator that the task is completed.</p>	<p>Standard:</p> <p>Log entry must include a Performed By: _____ and Independent Verification By: _____ sign-off.</p> <p>Operator informs evaluator that the task is completed.</p> <p>Cue: None</p> <p>Comments: DO NOT PROMPT!</p>	
<p>STEP 6 (Proc STEP 4.3.1.A)</p> <p>Check SECURITY RELATED if the jumper or bypass is to be installed in security systems identified in Operations Manual Sections B.08.12.01 through B.08.12.07 or has Component Master List system codes of SCM, SEL, SHV, or SIN, except for security doors that provide fire barriers.</p>	<p>Standard:</p> <p>Determines that the bypasses are <u>NOT</u> SECURITY RELATED and leaves SECURITY RELATED [] blank.</p> <p>Cue:</p> <p>If requested, next jumper bypass number is 02-100</p> <p>Comments:</p> <p>This is not a critical step because if sent to security, it would be returned prior to authorization.</p>	
<p>STEP 7 "C" (Proc STEP 4.3.1.B)</p> <p>Reason For Installation – Identify the work to be completed or the purpose of the bypass.</p> <p>1. Reference the approved procedure number or work order if applicable.</p>	<p>Standard:</p> <p>Enters "B.03.02–05.H.2 (Backseating HPCI Steam Supply Motor–Operated Valves) in the appropriate space on Form No. 3034.</p> <p>Cue: None</p> <p>Comments:</p> <p>Similar wording may be used. The procedure title is not required to be included.</p>	
<p>STEP 8 "C" (Proc STEP 4.3.1.C)</p> <p>Installed For – Identify the responsible individual, work group, or work document</p>	<p>Standard:</p> <p>Enters the Duty Shift Supv in the "Installed For" space.</p>	

MONTICELLO NUCLEAR GENERATING PLANT		JPM-4AWI-04 04 03-
INITIATE JUMPER BYPASS FORM		Revision 6a
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<p>number requiring the bypass installation. Bypasses installed for Operations may identify the Duty Shift Supervisor as the responsible individual.</p>	<p>Cue: None</p> <p>Comments:</p> <p>The SRO may also enter "Control Room Ops" or the controlling work group.</p>	
<p>STEP 9 (Proc STEP 4.3.1.D)</p> <p>If the bypass is installed for an OC reviewed procedure, the 10 CFR 50.59 review requirements have been satisfied. For all other bypasses, a System Engineer review is required to determine the need for a 10 CFR 50.59 evaluation. The System Engineer SHALL complete the Jumper Bypass Form per section 4.5. Note that for an Emergency condition, the bypass may be used and the review performed later.</p>	<p>Standard:</p> <p>Enter NO for System Engineer required (Section 3 of Form 3034).</p> <p>Cue: None</p> <p>Comments:</p> <ol style="list-style-type: none"> 1. This is an OC reviewed procedure thus, System Engineer review not required. 2. This is not a critical step because if the Shift Supv sent it for System Engineer review, this would be more conservative, although not needed. 	
<p>STEP 10 (Proc STEP 4.3.1.E)</p> <p>Installation anticipated greater than 7 days – Indicate YES or NO, and if YES, specify affected critical drawings. The preparer SHALL identify the Bypass Form number on the affected critical drawings by affixing a removable adhesive label (available from the Shift Supv) on the drawing preferably in the lower right-hand corner.</p>	<p>Standard:</p> <p>Enters NO.</p> <p>Cue: None</p> <p>Comments:</p> <p>The bypass will be cleared as soon as MO-2034 is backseated.</p>	
<p>STEP 11 (Proc STEP 4.3.1.F)</p> <p>Function Bypassed/Affected Equipment – Identify the function bypassed and/or the affected equipment.</p>	<p>Standard:</p> <p>Enters "NA" in the "Function Bypassed/Affected Equipment" space.</p> <p>Cue: None</p> <p>Comments:</p> <p>NA should be entered due to involvement of a procedure requiring OC revision review.</p>	
<p>STEP 12 (Proc STEP 4.3.1.G)</p> <p>If temporarily modifying electrical circuits, perform configuration verification of the proposed installation. If not modifying electrical circuits, the preparer may NA the CONFIGURATION VERIFICATION</p>	<p>Standard:</p> <p>Enters NA in the "Configuration Verified By" block.</p> <p>Cue: None</p> <p>Comments:</p>	

MONTICELLO NUCLEAR GENERATING PLANT		JPM-4AWI-04.04.03-
blank on the Jumper Bypass Form. See Section 4.4 for details.		NA should be entered due to involvement of a procedure requiring OC revision review.
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STEP 13 (Proc STEP 4.3.1.H) Prepared By: Provide the name of the respective work group representative, System Engineer or other individual requesting the jumper bypass.	Standard: Enters own name. Cue: None Comments: None
STEP 14 INFORM EVALUATOR THAT THE TASK HAS BEEN COMPLETED.	Standard: Operator informs evaluator that the task is completed. Cue: None Comments: DO NOT PROMPT!

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MONTICELLO NUCLEAR GENERATING PLANT		JPM-AWI-08.04.01-002
TITLE:	EXPECTED DOSE DETERMINATION TO INSPECT EQUIPMENT	Revision 0
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JPM SET-UP	
Simulator Setup:	<p style="text-align: center;">Calculator</p>
Initial Conditions:	<p>The plant is operating at 100% power.</p>
Initiating Cues:	<p>An inadvertent Group III caused a water hammer requiring the pipe supports and hangers to be inspected for possible damage. You have been requested to inspect the piping for #12 RWCU Pump (highlighted in red) that runs along the shield wall below RWCU MO-2398. Some climbing is anticipated and the estimated time to complete this task is 15 minutes.</p> <p>Determine the expected dose you would receive while completing this task, class of protective clothing to wear, and what additional actions must be taken to gain entry to the area. Vocalize your thought process and report when your task is complete.</p>

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TITLE:	EXPECTED DOSE DETERMINATION TO INSPECT EQUIPMENT	Revision 0
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PERFORMANCE INFORMATION

Performance Objective	Standard
<p>STEP 1</p> <p>Locates a copy of the survey map and the Radiation Work Permit (RWP 22).</p>	<p>Standard:</p> <p>Describes location of survey map and Radiation Work Permit (RWP 22)</p> <p>Cue:</p> <p>When operator describes where survey maps and RWPs are located then provide him with a copy of the appropriate survey map and RWP.</p> <p>Comments:</p> <p>Survey maps and RWPs are located on the basement level of the administration building in the same area where personnel obtain their ALANOR.</p>
<p>STEP 2</p> <p>Determine the Dose Rate near the component.</p>	<p>Standard:</p> <p>Dose Rate determined to be 440 mrem/hr.</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 3 "C"</p> <p>Calculates expected dose.</p>	<p>Standard:</p> <p>Expected dose calculated to be 110 mrem</p> <p>Cue: None</p> <p>Comments: None</p>

MONTICELLO NUCLEAR GENERATING PLANT		JPM-AWI-08.04.01-002
TITLE:	EXPECTED DOSE DETERMINATION TO INSPECT EQUIPMENT	Revision 0
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Performance Objective	Standard
<p>STEP 4 "C"</p> <p>Determines the requirements for entering a "Locked High Radiation Area (LHRA).</p>	<p>Standard:</p> <p>Determines that:</p> <ul style="list-style-type: none"> • Required to notify Rad Prot Coordinator • Requires a Dose Rate Meter • Requires Class 2 protective clothing <p>Cue: None</p> <p>Comments: May determine that a key is required for LHRA.</p>
<p>STEP 5</p> <p>INFORM EVALUATOR THAT THE TASK HAS BEEN COMPLETED.</p>	<p>Standard:</p> <p>Operator informs evaluator that the task is completed.</p> <p>Cue: None</p> <p>Comments: DO NOT PROMPT!</p>

MONTICELLO NUCLEAR GENERATING PLANT		JPM-AWI-08.04.01-001
TITLE:	EXPECTED DOSE DETERMINATION TO INSPECT EQUIPMENT	Revision 0
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JOB PERFORMANCE MEASURE

SRO: X SRO/RO: SRO/RO/NLO: NLO:

Plant Reference: 4 AWI-08.04.01, Rev. 12

Task Standard: Calculate Expected Dose to Inspect Equipment

K/A Reference: Generic Task No.: 2.3.4 Rating RO/SRO: 2.5/3.1

Recent Events: None

Probabilistic Risk Assessment Human Error: None

Monticello Specific Task List Reference: SS299.248

METHOD OF TESTING		TIME FOR COMPLETION
Simulator: <u> X </u>	Normal: <u> X </u>	Estimated Time to Complete: <u> 15 min. </u>
In-Plant: <u> </u>	Alternate Path: <u> </u>	Maximum Time to Complete: <u> 25 min. </u>
		Time Critical: <u> </u> Yes <u> X </u> No

Prepared By:	Date:
Reviewed By:	Date:
Shift Supv/Shift Mgr Review By:	Date:
Approved By:	Date:

MONTICELLO NUCLEAR GENERATING PLANT		JPM-AWI-08.04.01-001
TITLE:	EXPECTED DOSE DETERMINATION TO INSPECT EQUIPMENT	Revision 0
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JPM SET-UP											
<p>Simulator Setup:</p> <p style="padding-left: 40px;">Calculator</p>											
<p>Initial Conditions:</p> <p style="padding-left: 40px;">The plant is operating at 100% power.</p>											
<p>Initiating Cues:</p> <p style="padding-left: 40px;">An inadvertent Group III caused a water hammer requiring the pipe supports and hangers to be inspected for possible damage. You have been requested to inspect the piping for #12 RWCU Pump (highlighted in red) that runs along the shield wall below RWCU MO-2398. Some climbing is anticipated and the estimated time to complete this task is 15 minutes.</p> <p style="padding-left: 40px;">Your task is to determine the expected dose that an operator would receive, class of protective clothing required, and what actions must be taken to gain entry to the area while completing this task. Then determine which of the following operators can be assigned the task. Vocalize your thought process and report when your task is complete.</p>											
<table border="1"> <thead> <tr> <th>NAME</th> <th>YEAR TO DATE DOSE</th> </tr> </thead> <tbody> <tr> <td>John</td> <td>1950 mr</td> </tr> <tr> <td>Paul</td> <td>1800 mr</td> </tr> <tr> <td>George</td> <td>1750 mr</td> </tr> <tr> <td>Fred</td> <td>1930 mr</td> </tr> </tbody> </table>		NAME	YEAR TO DATE DOSE	John	1950 mr	Paul	1800 mr	George	1750 mr	Fred	1930 mr
NAME	YEAR TO DATE DOSE										
John	1950 mr										
Paul	1800 mr										
George	1750 mr										
Fred	1930 mr										

MONTICELLO NUCLEAR GENERATING PLANT		JPM-AWI-08.04.01-001
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PERFORMANCE INFORMATION

Performance Objective	Standard
<p>STEP 1</p> <p>Locates a copy of the survey map and the Radiation Work Permit (RWP 22).</p>	<p>Standard:</p> <p>Describes location of survey map and Radiation Work Permit (RWP22).</p> <p>Cue:</p> <p>When operator describes where survey maps and RWPs are located then provide him with a copy of the appropriate survey map and RWP.</p> <p>Comments:</p> <p>Survey maps and RWPs are located on the basement level of the administration building in the same area where personnel obtain their ALANOR.</p>
<p>STEP 2</p> <p>Determine the Dose Rate near the component.</p>	<p>Standard:</p> <p>Dose Rate determined to be 440 mrem/hr</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 3 "C"</p> <p>Calculates expected dose.</p>	<p>Standard:</p> <p>Expected dose calculated to be 110 mrem.</p> <p>Cue: None</p> <p>Comments: None</p>

MONTICELLO NUCLEAR GENERATING PLANT		JPM-AWI-08.04.01-001
TITLE:	EXPECTED DOSE DETERMINATION TO INSPECT EQUIPMENT	Revision 0
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Performance Objective	Standard
<p>STEP 4 "C"</p> <p>Determines the requirements for entering a "Locked High Radiation Area (LHRA).</p>	<p>Standard:</p> <p>Determines that:</p> <ul style="list-style-type: none"> • Required to notify Rad Prot Coordinator. • Requires a Dose Rate Meter • Requires Class 2 protective clothing <p>Cue: None</p> <p>Comments:</p> <p>May determine that a key is required for LHRA.</p>
<p>STEP 5 "C"</p> <p>Determines which of the four operators could perform the task.</p>	<p>Standard:</p> <p>Determines that Paul or George can perform the task.</p> <p>Cue: None</p> <p>Comments:</p> <p>John and Fred would exceed the 2000 mrem/yr. Administrative limit at Monticello. John 1950 + 110 = 2060 mrem Fred 1930 + 110 = 2040 mrem</p> <p>Paul and George will not exceed the 2000 mrem/yr administrative limit if assigned the task Paul 1800 + 110 = 1910 mrem George 1750 + 110 = 1860 mrem</p>
<p>STEP 6</p> <p>INFORM EVALUATOR THAT THE TASK HAS BEEN COMPLETED.</p>	<p>Standard:</p> <p>Operator informs evaluator that the task is completed.</p> <p>Cue: None</p> <p>Comments: DO NOT PROMPT!</p>

MONTICELLO NUCLEAR GENERATING PLANT		JPM-A.3 - 001
TITLE:	DON FIRE FIGHTING PROTECTION GEAR	Revision 0
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JOB PERFORMANCE MEASURE

SRO: _____ SRO/RO: X SRO/RO/NLO: _____ NLO: _____

Plant Reference: R7637L-003

Task Standard: Don Fire Fighting Protection Gear

K/A Reference: Generic Task No.: 2.4.26 Rating RO/SRO: 2.9/3.3

Recent Events: None

Probabilistic Risk Assessment Human Error: None

Monticello Specific Task List Reference: FB006.001

METHOD OF TESTING		TIME FOR COMPLETION
Simulator: _____ X In-Plant: _____	Normal: <u> X </u> Alternate Path: _____	Estimated Time to Complete: <u> 8 min. </u> Maximum Time to Complete: <u> 15 min. </u> Time Critical: _____ Yes <u> X </u> No

Prepared By:	Date:
Reviewed By:	Date:
Shift Supv/Shift Mgr Review By:	Date:
Approved By:	Date:

MONTICELLO NUCLEAR GENERATING PLANT		JPM-A.3 - 001
TITLE:	DON FIRE FIGHTING PROTECTION GEAR	Revision 0
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JPM SET-UP	
Simulator Setup:	Not Applicable
Initial Conditions:	The following announcement has just been given over the plant page: “Attention all plant personnel, a fire has been reported in the Hot Machine Shop Lube Oil Storage Tank area, this is a Class B fire, Fire Brigade members report to the fire brigade room”.
Initiating Cues:	You are to report to the fire brigade room and don the appropriate fire fighting protection gear.

MONTICELLO NUCLEAR GENERATING PLANT		JPM-A.3 - 001
TITLE:	DON FIRE FIGHTING PROTECTION GEAR	Revision 0
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PERFORMANCE INFORMATION

Performance Objective	Standard
<p>STEP 1</p> <p>Locates fire brigade room</p>	<p>Standard:</p> <p>Proceeds to the fire brigade room.</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 2</p> <p>Selects appropriate clothing:</p> <ul style="list-style-type: none"> • Boots • Bunker Pants • Coat • Gloves • NOMAX Hood • Helmet 	<p>Standard:</p> <p>Locates and selects appropriate clothing</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 3 "C"</p> <p>Dons the following protective clothing:</p> <ul style="list-style-type: none"> • Bunker Pants • Boots • Coat • NOMAX Hood 	<p>Standard:</p> <p>Dons the following protective clothing:</p> <ol style="list-style-type: none"> 1. Bunker Pants 2. Boots 2. Coat 2. NOMAX Hood <p>Cue: None</p> <p>Comments: Bunker pants must be put on prior to the coat.</p>

MONTICELLO NUCLEAR GENERATING PLANT		JPM-A.3 - 001
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Performance Objective	Standard
<p>STEP 4 "C"</p> <p>Don Scott Air Pack</p>	<p>Standard:</p> <ol style="list-style-type: none"> 1. Removes protective cover from Scott Air Pack 2. Inspects Scott Air Pack and hoses for damage and proper air pressure (2100 psig) 3. Places arms through harness 4. Buckles waist belt 5. Tightens shoulder and waist belt straps 6. Dons face mask and tightens/adjusts straps 7. Performs negative pressure test to prove adequate mask seal. <p>Cue: Do not valve in air or hook up the mask to the regulator on the Scott Air Pack</p> <p>Comments: None</p>
Performance Objective	Standard
<p>STEP 5 "C"</p> <p>Dons the rest of fire fighting gear</p> <ul style="list-style-type: none"> • Gloves • Helmet 	<p>Standard</p> <p>Dons helmet and gloves</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 6</p> <p>INFORM EVALUATOR THAT THE TASK HAS BEEN COMPLETED.</p>	<p>Standard:</p> <p>Operator informs evaluator that the task is completed.</p> <p>Cue: None</p> <p>Comments: DO NOT PROMPT!</p>

MONTICELLO NUCLEAR GENERATING PLANT		JPM-A.2-101-001
TITLE:	E-PLAN CLASSIFICATION	Revision 7a
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JPM SET-UP	
<p>Simulator Setup: Form 5790-102-02 incorrectly filled out Form 5790-102-02 correctly filled out MET Tower Data Sheet.</p>	
<p>Initial Conditions: The plant is operating at 100% power with the MPR providing pressure control due to the EPR being out of service.</p> <p>There is a sudden pressure increase of 10 psig followed by a rapid pressure decrease to 820 psig. A group 1 Isolation and Reactor scram occur causing all rods to fully insert. In verifying the Group 1 Isolation, it is discovered that MSIVs AO-2-80A and AO-2-86A failed to close.</p> <p>ARM readings in the Turbine Building (Turbine floor, front standard, etc.) increase above their alarm setpoints. As attempts are made to close the failed MSIVs, radiation levels increase in both the Reactor Building and Turbine Building until Turbine Building ARMs indicate full scale and the Reactor Building vent WRGMs have increased to 7200 μci/sec. Fuel cladding degradation is confirmed due to Main Steam Line and Offgas Monitor radiation level increases. All applicable abnormal and EOP procedures have been entered.</p> <p>Reactor water level is being maintained +9 to +48 inches with feedwater level control. Reactor Building vent monitors are at 20 mR/HR and slowly increasing. Fuel Pool process radiation monitors are at 10 and 15 mR/HR and stable. Standby Gas Treatment/Secondary Containment isolation initiation setpoints have NOT been reached yet.</p> <p>The shift chemist has just informed you there is an unplanned release and it is currently ten (10) times greater than normal.</p>	
<p>Initiating Cues: Determine the appropriate Emergency Classification which indicates the highest classification. Notify SEC to make Protective Action Recommendations if appropriate.</p>	

Time To Classify

START TIME: _____

I/cmb

MONTICELLO NUCLEAR GENERATING PLANT		JPM-A.2-101-001
TITLE:	E-PLAN CLASSIFICATION	Revision 7a
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PERFORMANCE INFORMATION

Performance Objective	Standard
<p>STEP 1</p> <p>Locates procedure A.2-101, CLASSIFICATION OF EMERGENCIES.</p>	<p>Standard:</p> <p>Locates appropriate procedure.</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 2 (Procedure Step 6.1.2.A.2)</p> <p>Refer to FIGURE 7.1 (LIST OF INITIATING CONDITIONS) and identify any guidelines applicable to the initiating condition.</p>	<p>Standard:</p> <p>Locates appropriate initiating condition(s). (1, 2, 5, 6, 28, 29)</p> <p>Cue: None</p> <p>Comments:</p> <p>Operator may locate appropriate initiating condition(s) by searching through FIGURE 7.2 (EMERGENCY CLASSIFICATION GUIDELINES) directly without use of FIGURE 7.1.</p>
<p>STEP 3 (Procedure STEP 6.1.2.A.3))</p> <p>Locate the applicable guideline in FIGURE 7.2 (EMERGENCY CLASSIFICATION GUIDELINES).</p>	<p>Standard:</p> <p>Locates appropriate guideline(s). (1, 2, 5, 6, 28, 29)</p> <p>Cue: None</p> <p>Comments:</p> <p>Guideline 28 is the most applicable.</p>
<p>STEP 4 (Procedure STEP 6.1.2.A.4)</p> <p>If multiple events and/or indications are involved, classify the emergency based on the event (or indication) that results in the highest (most conservative) emergency classification.</p>	<p>Standard:</p> <p>Classifies event as a General Emergency IAW Guideline 28</p> <p>Cue: None</p> <p>Comments: Classification must be completed within 15 minutes.</p>

Time To Classify

STOP TIME: _____

Time to Make Notification

START TIME _____

I/cmb

MONTICELLO NUCLEAR GENERATING PLANT		JPM-A.2-101-001
TITLE:	E-PLAN CLASSIFICATION	Revision 7a
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Performance Objective	Standard
<p>STEP 5 (Proc A.2-204, STEP 6.1.1.A)</p> <p>Initiate Form 5790-102-02 (MONTICELLO EMERGENCY NOTIFICATION REPORT FORM)</p> <p>A. Complete Section 1.4 recommending an evacuation of a 2 mile radius and 5 miles downwind and advise the remainder of the plume EPZ to go indoors to monitor EAS broadcasts.</p>	<p>Standard:</p> <p>Notifies SEC to make Protective Action Recommendations if appropriate</p> <p>Cue: As SEC, provide candidate with completed copy of Form 5790-102-02 (with incorrect sectors N, P, Q) for review and approval. (Include MET Tower data)</p> <p>Comments: None</p>
<p>STEP 6 (Proc A.2-204, STEP 6.1.1.B)</p> <p>Determine which geopolitical sub-areas are affected by referring to the Sector Sub-area Conversion Table on Form 5790-102-02.</p>	<p>Standard:</p> <p>Identifies the geopolitical sub-areas on Form 5790-102-02 should be 5E and 5S.</p> <p>Instructs SEC to fill out a new form with correct Geo-political sub-areas identified (downwind sectors E, F,G and sub-areas 5E & 5S).</p> <p>Cue: Provide candidate with Form 5790-102-02 correctly completed (with sectors E, F, G).</p> <p>Comments: None</p>
<p>STEP 7 (Proc A.2-204, STEP 6.1.1.C)</p> <p>Ensure completion of Parts 1.0 and 2.0 of Form 5790-102-02 and submit the completed form to the ED for approval.</p>	<p>Standard:</p> <p>Reviews new Form 5790-102-02 and approves.</p> <p>Cue: None</p> <p>Comments: None</p>
<p>STEP 8</p> <p>INFORM EVALUATOR THAT THE TASK HAS BEEN COMPLETED.</p> <p>Time to Make Notification</p> <p>STOP TIME _____</p>	<p>Standard:</p> <p>Operator informs evaluator that the task is completed.</p> <p>Cue: None</p> <p>Comments: Form 5790-102-02 must be completed within 10 minutes from classification. DO NOT -PROMPT!</p>

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Meteorological Data	
Channel 04	17
Channel 05	297
Channel 11	-1.47
Channel 13	54°F
Channel 16	Y