

January 31, 2003

Mr. Dhiaa Jamil
Vice President, McGuire Site
Duke Energy Corporation
12700 Hagers Ferry Road
Huntersville, NC 28078-8985

SUBJECT: WILLIAM B. MCGUIRE NUCLEAR STATION, UNIT 2 RE: REQUEST FOR
ADDITIONAL INFORMATION ON STEAM GENERATOR TUBE INSPECTION

Dear Mr. Jamil:

By letters dated March 20 and May 30, 2002, you submitted reports for the McGuire, Unit 2, end of cycle 14 steam generator tube inspection, in accordance with the Technical Specifications and the American Society of Mechanical Engineers Code, Section XI reporting requirements. The Nuclear Regulatory Commission staff has reviewed the information provided and has determined that additional information is required. Our questions are provided in the Enclosure.

Please contact me at (301) 415-1493, if you have any other questions on these issues.

Sincerely,

/RA/

Robert E. Martin, Senior Project Manager, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-370

Enclosure: Request for Additional Information

cc w/encl: See next page

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DATE	1/13/03	1/29/03	1/30/03	1/31/03

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REQUEST FOR ADDITIONAL INFORMATION
ON STEAM GENERATOR TUBE INSPECTION REPORT
DUKE POWER COMPANY
WILLIAM B. MCGUIRE NUCLEAR STATION, UNIT 2
DOCKET NUMBER 50-370

1. Provide definitions for the codes HNI and CBH used in the reports under the "IND" and "LOCATION" columns, respectively. Explain how these codes are used at McGuire, Unit 2.
2. Identify all types of imperfections that are left in service (e.g., wear, manufacturing burnish marks, etc.), the number of each type of imperfection, their location in the steam generator and the basis for leaving them in service (e.g., depth sizing, review of historical data, etc.).
3. The report dated May 30, 2002, indicates that two possible loose parts (PLPs) were confirmed in each steam generator inspected. Describe the actions that were taken upon confirmation of these loose parts, and the basis for leaving them in service, if this was the result.
4. The steam generator tube inspection reports indicate that a PLP was identified in the freespan in Rows 93 and 95 in steam generator B. The freespan location is an unusual location for a PLP. If not already addressed as a result of question 4, above, describe what actions were taken based on the identification of the PLP signal? Describe any theories/conclusions you have regarding the cause of the PLPs.
5. The report dated May 30, 2002, indicates that one tube in steam generator B, which contained a volumetric indication, was left in service. What was the volumetric indication attributed to? If it was attributed to degradation, explain the cause of the degradation, the tube and location within the tube where the degradation was identified, and the basis for leaving the tube in service.
6. The Updated Final Safety Analysis Report (UFSAR) provides some information on the design of the steam generators. Please provide additional details, as follows, which will support the staff's review of the steam generator reports, especially the tables containing the tube-by-tube listing.
 - A. Tube Support Plates - The UFSAR states they are an open flow lattice design. How many tube contact points are there and what is the thickness of the tube support plates?
 - B. What is the tube pitch?

- C. What tube expansion method was used on the tube within the tubesheet? How thick is the tubesheet?
- D. Identify the fabricator of the steam generator tubes and the steam generators.
- E. Please provide a diagram of the steam generator which identifies the numbering scheme of the tube support plates and the fan bars. This will enable the staff to determine the location of the imperfections identified in the tube-by-tube listings enclosed with the steam generator reports.

McGuire Nuclear Station

cc:

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