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DUKE POWER Catawba Nuclear Station 4800 Concord Rd. York, SC 29745

803 831 3000

January 7, 2003

RE: Catawba Nuclear Station Selected Licensee Commitments Manual Revision Date 12/16/02

Attached are revisions to the Catawba Nuclear Station Selected Licensee Commitments Manual. Please remove and replace the following pages:

REMOVE

INSERT

LIST OF EFFECTIVE PAGES

Pages 1 through 4 Revision 1 Pages 1 through 4 Revision 2

TAB 16.7

Chapter 16.7-5, remove pages 16.7-5-1, 16.7-5-2, 16.7-5-3 Revision 0 Chapter 16.7-5, insert pages 16.7-5-1, 16.7-5-2, 16.7-5-3 Revision 1

TAB 16.9

Chapter 16.9-19, remove pages 16.9-19-1 & 16.9-19-2 Revision 0

Chapter 16.9-19, insert pages 16-9-19-1 & 16.9-19-2 Revision 1

If you have any questions concerning the contents of this package update, contact Jill Ferguson at (803) 831-3938.

Gary D. Gilbert Regulatory Compliance Manager

Attachments

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16.7 INSTRUMENTATION

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16.7-5 Turbine Overspeed Protection

COMMITMENT At least one Turbine Overspeed Protection System shall be OPERABLE.

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APPLICABILITY: MODES 1, 2, and 3.

REMEDIAL ACTIONS

_	CONDITION		REQUIRED ACTION	COMPLETION TIME
Α.	One stop valve or one control valve per high pressure turbine steam line inoperable.	A.1	Restore inoperable valve(s) to OPERABLE status.	72 hours
	·	<u>OR</u>		
		A.2	Close at least one valve in affected steam line(s).	78 hours
		OR		
		A.3	Isolate the turbine from the steam supply.	78 hours
В.	One intermediate stop valve or one intercept valve per low pressure	B.1	Restore inoperable valve(s) to OPERABLE status.	72 hours
	inoperable.	<u>OR</u>		
		B.2	Close at least one valve in affected steam line(s).	78 hours
		<u>OR</u>		
-	-	В.3	Isolate the turbine from the steam supply.	78 hours

(continued)

REMEDIAL ACTIONS (continued)

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	CONDITION		REQUIRED ACTION	COMPLETION TIME
C.	Turbine Overspeed Protection System inoperable for reasons other than Condition A or B.	C.1	Isolate the turbine from the steam supply.	6 hours

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TESTING REQUIREMENTS

	TEST	FREQUENCY
TR 16.7-5-1	 This TR shall be performed in MODE 1 or in MODE 2 with the turbine operating. 	
	2. Not required to be performed until 24 hours after each valve is opened.	
	Cycle four high pressure turbine stop valves, six low pressure turbine intermediate stop valves, and six low pressure turbine intercept valves through at least one complete cycle from the running position.	31 days
TR 16.7-5-2	 This TR shall be performed in MODE 1 or in MODE 2 with the turbine operating. Not required to be performed until 24 hours after 	
	each valve is opened. Perform direct observation of the movement of four high pressure turbine stop valves, six low pressure turbine intermediate stop valves, six low pressure turbine intercept valves, and four high pressure turbine control	92 days
<u></u>	valves, through one complete cycle from the funning position.	

(continued)

TESTING REQUIREMENTS (continued)

	TEST	FREQUENCY
TR 16.7-5-3	Perform CHANNEL CALIBRATION.	18 months
TR 16.7-5-4	Disassemble at least one each of the four high pressure turbine stop valves, six low pressure turbine intermediate stop valves, six low pressure turbine intercept valves, and four high pressure turbine control valves, and perform a visual and surface inspection of valve seats, disks, and stems, and verify no unacceptable flaws or corrosion.	40 months

BASES This COMMITMENT is provided to ensure that the Turbine Overspeed Protection instrumentation and the turbine speed control valves are OPERABLE and will protect the turbine from excessive overspeed. Protection from turbine excessive overspeed is required since excessive overspeed of the turbine could generate potentially damaging missiles which could impact and damage safety related components, equipment, or structures.

> The term "Isolate the turbine from the steam supply" used in Required Actions A.3, B.3, and C.1 can be met in several ways. These include: Maintaining the turbine in a tripped condition; or Hydraulically gagging all four main stop valves closed; or Hydraulically gagging all four main control valves closed; or Closing the main steam isolation valves and main steam isolation valve bypass valves.

REFERENCES None

16.9 AUXILIARY SYSTÊMS

16.9-19 Refueling Operations - Manipulator Crane

COMMITMENT The reactor building manipulator crane and an auxiliary hoist shall be used for movement of fuel assemblies or control rods and shall be OPERABLE with:

- a. The manipulator crane used for movement of fuel assemblies having:
 - 1) A minimum capacity of 3250 pounds, and
 - 2) An overload cutoff limit \leq 2900 pounds.
- b. Auxiliary hoists used for latching, unlatching, and drag load testing of control rods having:
 - 1) A minimum capacity of 1000 pounds, and
 - A load indicator which shall be used to prevent applying a lifting force > 600 pounds on the core internals.

APPLICABILITY: During movement of fuel assemblies and control rods within the reactor vessel.

REMEDIAL ACTIONS

	CONDITION		REQUIRED ACTION	COMPLETION TIME
Α.	Manipulator crane or auxiliary hoist inoperable.	A.1	Suspend use of any inoperable manipulator crane or auxiliary hoist from operations involving the movement of fuel assemblies and control rods within the reactor vessel.	Immediately

TESTING REQUIREMENTS

	TEST	FREQUENCY
TR 16.9-19-1	Perform a load test of \geq 3250 pounds for each manipulator crane used for movement of fuel assemblies within the reactor vessel and demonstrate an automatic load cutoff when crane load is > 2850 pounds.	Within 14 days prior to movement of fuel assemblies within the reactor vessel
TR 16.9-19-2	Perform a load test of \geq 1000 pounds for each auxiliary hoist and associated load indicator used for movement of control rods or control rod drag load testing within the reactor vessel.	Within 14 days prior to movement of control rods or control rod drag load testing within the reactor vessel

- BASES The OPERABILITY requirements for the manipulator cranes ensure that: (1) manipulator cranes will be used for movement of control rods and fuel assemblies, (2) each crane has sufficient load capacity to lift a control rod or fuel assembly, and (3) the core internals and reactor vessel are protected from excessive lifting force in the event they are inadvertently engaged during lifting operations.
- REFERENCES 1. Letter from NRC to Gary R. Peterson, Duke, Issuance of Improved Technical Specifications Amendments for Catawba, September 30, 1998.