

January 21, 2003

Mr. J. A. Stall
Senior Vice President, Nuclear and
Chief Nuclear Officer
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

SUBJECT: TURKEY POINT PLANT, UNIT 4 - REQUEST FOR ADDITIONAL
INFORMATION REGARDING REDUCTION OF DECAY TIME FOR CORE
OFF-LOAD (TAC NOS. MB6549 AND MB6550)

Dear Mr. Stall:

By letter dated October 21, 2002, Florida Power and Light Company requested an amendment to reduce the minimum time required for reactor subcriticality prior to removing irradiated fuel from the reactor pressure vessel from 100 to 72 hours, and relocate the 72 hour decay time requirement from the Technical Specifications (TSs) to the TS Bases. Based on our review of your submittal, the U.S. Nuclear Regulatory Commission staff finds that a response to the enclosed request for additional information is needed before we can complete the review.

This request was discussed with Olga Hanek of your staff on January 14, 2003, and it was agreed that a response would be provided within 30 days of the issuance of this letter.

If you have any questions, please contact me at (301) 415-2315.

Sincerely,

/RA/

Eva A. Brown, Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-250 and 50-251

Enclosure: Request for Additional Information

cc w/encl: See next page

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Mr. J. A. Stall
Florida Power and Light Company

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REQUEST FOR ADDITIONAL INFORMATION

RELOCATION OF DECAY TIME REQUIREMENT AND DEFINITION OF RECENTLY

IRRADIATED FUEL

TURKEY POINT PLANT UNITS 3 AND 4

DOCKET NOS. 50-250 AND 50-251

1. Provide the total fission product activities in the fuel rod gap that are available for release to the spent fuel pool water surrounding the failed fuel assembly consistent with a 72-hour decay.
2. Provide the amounts of fission product activities (in curies) released to the environment (noble gases and iodine) following the postulated fuel handling accident.
3. The submittal requests to use a calculation methodology for the licensing basis to determine an acceptable off-load rate and decay time (72 hours or greater) given the heat load and cooling water temperature. Submit a sample calculation demonstrating the methodology and benchmark data for the calculation. Provide inputs and outputs for other examples.
4. The submittal states that administrative procedures are in place to halt fuel movement to ensure pool temperature is maintained below 150 degrees Fahrenheit (°F). Peak pool temperatures will not occur for many hours after spent fuel movement is halted. This administrative control was established when the peak temperature was slightly over 150 °F.
 - a) Demonstrate how your administrative procedures ensure the pool temperature will remain below 150 °F.
 - b) Explain the criteria for resuming off-loading the spent fuel.
5. In Section 2.1.3, the submittal references American Concrete Institute (ACI)-349 and states that the code indicates temperatures of up to 350 °F can be tolerated for short periods of time with no appreciable impact on concrete strength. However, this section of the code is referring to accident conditions. ACI-349 states that, for normal operation, the temperature shall not exceed 150 °F. The code does not support long-term steady state temperatures of 180 °F. However, it appears that the submittal does not request an increase in the design basis temperature above 150 °F. Please verify that this is accurate.
6. Explain how the availability of the alternate spent fuel pool pump and make-up water sources are controlled to ensure availability during off-loads.

Enclosure