

December 31, 2002

U.S. Nuclear Regulatory Commission  
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Washington, D.C. 20555-0001

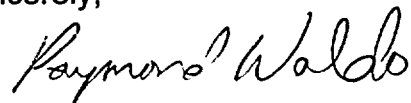
Subject: **Docket No. 50-361**  
**Licensee Event Report No. 2002-006**  
**San Onofre Nuclear Generating Station, Unit 2**

Gentlemen:

This submittal provides a Licensee Event Report (LER) for an occurrence involving the actuation of the Reactor Protection System when Unit 2 experienced a loss of main feedwater to the steam generators.

Any actions listed are intended to ensure continued compliance with existing commitments as discussed in applicable licensing documents; this LER contains no new commitments. If you require any additional information, please so advise.

Sincerely,



LER No. 2002-06

cc: E. W. Merschoff, Regional Administrator, NRC Region IV  
C. C. Osterholtz, NRC Senior Resident Inspector, San Onofre Units 2 & 3

*IE22*

**LICENSEE EVENT REPORT (LER)**

(See reverse for required number of digits/characters for each block)

Estimated burden per response to comply with this mandatory information collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records Management Branch (T-6 E6), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to bjs@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202 (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

<b>1. FACILITY NAME</b> San Onofre Nuclear Generation Station (SONGS) Unit 2	<b>2. DOCKET NUMBER</b> 05000-361	<b>3. PAGE</b> 1 of 2
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**4. TITLE**  
Main Feedwater Controller fault causes loss of Main Feedwater and Reactor Trip on Low Steam Generator Level

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MO	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO	MO	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
11	02	2002	2002-006-00			12	31	2002	None	
									FACILITY NAME	DOCKET NUMBER

<b>9. OPERATING MODE</b>	1	<b>11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §:</b> (Check all that apply)			
<b>10. POWER LEVEL</b>	100	20.2201(b)	20.2203(a)(3)(ii)	50.73(a)(2)(ii)(B)	50.73(a)(2)(ix)(A)
		20.2201(d)	20.2203(a)(4)	50.73(a)(2)(iii)	50.73(a)(2)(x)
		20.2203(a)(1)	50.36(c)(1)(i)(A)	X 50.73(a)(2)(iv)(A)	73.71(a)(4)
		20.2203(a)(2)(i)	50.36(c)(1)(ii)(A)	50.73(a)(2)(v)(A)	73.71(a)(5)
		20.2203(a)(2)(ii)	50.36(c)(2)	50.73(a)(2)(v)(B)	OTHER Specify in Abstract below or in NRC Form 366A
		20.2203(a)(2)(iii)	50.46(a)(3)(ii)	50.73(a)(2)(v)(C)	
		20.2203(a)(2)(iv)	50.73(a)(2)(i)(A)	50.73(a)(2)(v)(D)	
		20.2203(a)(2)(v)	50.73(a)(2)(i)(B)	50.73(a)(2)(vii)	
		20.2203(a)(2)(vi)	50.73(a)(2)(i)(C)	50.73(a)(2)(viii)(A)	
		20.2203(a)(3)(i)	50.73(a)(2)(i)(A)	50.73(a)(2)(viii)(B)	

**12. LICENSEE CONTACT FOR THIS LER**

NAME R. W. Waldo, Station Manager, Nuclear Generation	TELEPHONE NUMBER (Include Area Code) 949-368-8725
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**13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT**

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX
X	JB	LC	F180	N					

<b>14. SUPPLEMENTAL REPORT EXPECTED</b>				<b>15. EXPECTED SUBMISSION DATE</b>		
YES (If yes, complete EXPECTED SUBMISSION DATE)	X	NO		MONTH	DAY	YEAR

**16. ABSTRACT** (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On 11/2/2002 at 0316 PST, with the Unit 2 reactor at 100% power, the Reactor Protection System (RPS) tripped the Unit A main Feedwater (FW) controller failed; the FW regulating valve (2FV1111) began to close; and the water level in steam generator 89 decreased to the low-level trip set point. SCE made a 4 hour telephone notification to the NRC in accordance with 10 CFR 50.72(b) and is providing this follow-up report in accordance with 10 CFR 50.73(a)(2)(iv)(A)

SCE's cause evaluation is ongoing. However, preliminary investigations indicate that the 2FC1111 main FW controller card had a shorted low limit operational amplifier (Motorola, MLM301AG, date code 7432) This may be an age-related failure. This LER will be supplemented if this conclusion changes significantly. SCE replaced both Unit 2 main FW controller cards (2FC1111 and 2FC1121) with new cards (even though 2FC1121 tested satisfactorily) As a precautionary measure, the corresponding cards in Unit 3 have been scheduled for replacement

UFSAR Section 15.2.2.5 and 15.10.2.2.5 evaluate the Loss of Normal Feedwater Flow with the plant at full power. The UFSAR analysis shows that all of the event acceptance criteria (DNBR, offsite doses, peak linear heat rate, peak RCS pressure, and peak secondary pressure) are met. Therefore, there was little to no safety significance to this event

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

1 FACILITY NAME	2 DOCKET NUMBER	6 LER NUMBER			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REV NO	
San Onofre Nuclear Generating Station (SONGS) Unit 2	05000-361	2002	-006 -	00	2 of 2

Reactor Vendor: Combustion Engineering

Description of the Event:

On November 2, 2002 at 0316 PST (event date), with Unit 2 reactor at 100% power, the Reactor Protection System (RPS) {EIS Code JC} actuated causing a Unit 2 reactor trip. A main Feedwater (FW) controller failed {EIS Code JB}; the FW regulating valve (2FV1111) {EIS Component Code FCV} began to close; and the water level in steam generator 89 decreased to the low-level trip set point.

On November 2, 2002, at 0538 PST, Southern California Edison (SCE) made a 4 hour telephone notification to the NRC Operations Center (Log No. 39340) in accordance with 10 CFR 50.72(b)(2)(iv)(B) and 10 CFR 50.72(b)(3)(iv)(A). SCE is providing this follow-up report in accordance with 10 CFR 50.73(a)(2)(iv)(A).

Cause of the Event:

SCE's cause evaluation is ongoing. However, preliminary investigations indicate that the 2FC1111 main FW controller card (serial number 3252836) had a shorted low limit operational amplifier (Motorola, MLM301AG, date code 7432) (Cause Code: X). This may be an age-related failure. This LER will be supplemented if this conclusion changes significantly.

Corrective Actions:

SCE replaced both Unit 2 main FW controller cards (2FC1111 and 2FC1121) with new cards (even though 2FC1121 tested satisfactorily). As a precautionary measure, the corresponding cards in Unit 3 have been scheduled for replacement.

Safety Significance:

UFSAR Section 15.2.2.5 and 15.10.2.2.5 evaluate the Loss of Normal Feedwater Flow with the plant at full power. The UFSAR analysis shows that all of the event acceptance criteria (DNBR, offsite doses, peak linear heat rate, peak RCS pressure, and peak secondary pressure) are met. Therefore, there was little to no safety significance to this event.

An assessment of the conditional core damage probability (CCDP) and the conditional large early release probability (CLERP) for the November 2, 2002 event, determined that the Unit 2 CCDP and CLERP were 5.0E-6 and 2.2E-7, respectively. The assessment was based on the reported actual component unavailability and system alignments at the time of the event.

Additional Information:

On November 21, 1999, the Unit 3 main feedwater controller card 3FC1121 failed due to a manufacturing defect (pinhole oxide defect) in an operational amplifier (Fairchild, date code 7825). That controller card had been in service for many years and when it failed, it failed at the location of the manufacturing defect. Corrective actions for the November 21, 1999 event focused on cards with Fairchild operational amplifiers.