

January 2, 2003

Mr. J. A. Price
Site Vice President - Millstone
Dominion Nuclear Connecticut, Inc.
c/o Mr. David A. Smith
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Waterford, CT 06385

SUBJECT: MILLSTONE POWER STATION, UNIT NOS. 2 AND 3 - ISSUANCE OF
AMENDMENT RE: RELOCATE SELECTED MILLSTONE UNIT 2 AND 3
TECHNICAL SPECIFICATION RELATED TO THE REACTOR COOLANT
SYSTEM AND PLANT SYSTEMS TO THE TECHNICAL REQUIREMENTS
MANUAL (TAC NOS. MB4066 AND MB4067)

Dear Mr. Price:

The Commission has issued the enclosed Amendment Nos. 272 and 214 to Facility Operating License Nos. DPR-65 and NPF-49 for the Millstone Power Station, Unit Nos. 2 (MP2) and 3 (MP3), in response to your application dated February 14, 2002, as supplemented on September 9, 2002.

The amendment will relocate selected MP2 and MP3 Technical Specifications (TSs) related to the Reactor Coolant System and Plant Systems to the Technical Requirements Manual. The TSs for Unit 2 include 3/4.4.9.1, "Pressure/Temperature Limits," 3/4.7.2, "Steam Generator Pressure/Temperature Limitation," 3/4.7.5, "Flood Level," 3/4.7.7, "Sealed Source Contamination," and related tables, figures, and Bases sections. The TSs for Unit 3 include 3/4.4.9.1, "Pressure/Temperature Limits," 3/4.7.2, "Steam Generator Pressure/Temperature Limitation," 3/4.7.6, "Flood Protection," 3/4.7.11, "Sealed Source Contamination," and corresponding tables, figures, and Bases sections.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Victor Nerses, Sr. Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-336 and 50-423

Enclosures: 1. Amendment No. 272 to DPR-65
2. Amendment No. 214 to NPF-49
3. Safety Evaluation

cc w/encls: See next page

Millstone Power Station
Units 2 and 3

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Units 2 and 3

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The amendment will relocate selected MP2 and MP3 Technical Specifications (TSs) related to the Reactor Coolant System and Plant Systems to the Technical Requirements Manual. The TSs for Unit 2 include 3/4.4.9.1, "Pressure/Temperature Limits," 3/4.7.2, "Steam Generator Pressure/Temperature Limitation," 3/4.7.5, "Flood Level," 3/4.7.7, "Sealed Source Contamination," and related tables, figures, and Bases sections. The TSs for Unit 3 include 3/4.4.9.1, "Pressure/Temperature Limits," 3/4.7.2, "Steam Generator Pressure/Temperature Limitation," 3/4.7.6, "Flood Protection," 3/4.7.11, "Sealed Source Contamination," and corresponding tables, figures, and Bases sections.

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Sincerely,

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Victor Nerses, Sr. Project Manager, Section 2
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Docket Nos. 50-336 and 50-423

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3. Safety Evaluation

cc w/encls: See next page

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ACCESSION NUMBER: ML030030636 *SE provided 10/21/02, no major changes made

** see previous concurrence

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DOMINION NUCLEAR CONNECTICUT, INC.

DOCKET NO. 50-336

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 272
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the applicant dated February 14, 2002, as supplemented on September 9, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 272 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, and shall be implemented within 90 days of issuance. The implementation of this amendment shall include the relocation of certain technical specification requirements to the Millstone Power Station, Unit No. 2 Technical Requirements Manual as described in the licensee's application dated February 14, 2002, as supplemented on September 9, 2002, and evaluated in the staff's Safety Evaluation attached to this amendment.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: January 2, 2003

ATTACHMENT TO LICENSE AMENDMENT NO. 272

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following pages of the Appendix A Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

VIII
XIII
3/4 4-18
3/4 4-20
3/4 7-10
3/4 7-13
3/4 7-14
3/4 7-15
3/4 7-19
3/4 7-20
B3/4 4-7
B3/4 7-3a
B3/4 7-4a
B3/4 7-5

Insert

VIII
XIII
3/4 4-18
3/4 4-20
3/4 7-10
3/4 7-13
3/4 7-14
3/4 7-15
3/4 7-19
3/4 7-20
B3/4 4-7
B3/4 7-3a
B3/4 7-4a
B3/4 7-5

DOMINION NUCLEAR CONNECTICUT, INC., ET AL.

DOCKET NO. 50-423

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 214
License No. NPF-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the applicant dated February 14, 2002, as supplemented on September 9, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-49 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 214, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated in the license. Dominion Nuclear Connecticut, Inc. shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance, and shall be implemented within 90 days of issuance. The implementation of this amendment shall include the relocation of certain technical specification requirements to the Millstone Power Station, Unit No. 3 Technical Requirements Manual as described in the licensee's application dated February 14, 2002, as supplemented on September 9, 2002, and evaluated in the staff's Safety Evaluation attached to this amendment.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA by SRichards for/

James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: January 2, 2003

ATTACHMENT TO LICENSE AMENDMENT NO. 214

FACILITY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace the following pages of the Appendix A Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

viii
x
xiv
xv
3/4 4-33
3/4 4-36
3/4 7-10
3/4 7-14
3/4 7-30
3/4 7-31
B3/4 4-12
B3/4 7-7
B3/4 7-10
B3/4 7-25

Insert

viii
x
xiv
xv
3/4 4-33
3/4 4-36
3/4 7-10
3/4 7-14
3/4 7-30
3/4 7-31
B3/4 4-12
B3/4 7-7
B3/4 7-10
B3/4 7-25

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 272
TO FACILITY OPERATING LICENSE NO. DPR-65 AND
AMENDMENT NO. 214
TO FACILITY OPERATING LICENSE NO. NPF-49
DOMINION NUCLEAR CONNECTICUT, INC.
MILLSTONE POWER STATION, UNIT NOS. 2 AND 3
DOCKET NO. 50-336 AND 50-423

1.0 INTRODUCTION

By letter dated February 14, 2002, as supplemented on September 9, 2002, the Dominion Nuclear Connecticut, Inc. (licensee or DNC), submitted a request for changes to the Millstone Power Station, Unit Nos. 2 (MP2) and 3 (MP3) Technical Specifications (TSs). The requested changes would relocate selected MP2 and MP3 TSs related to the Reactor Coolant System and Plant Systems to the Technical Requirements Manual (TRM). The MP2 TSs proposed for relocation include 3/4.4.9.1, "Pressure/Temperature Limits," 3/4.7.2, "Steam Generator Pressure/Temperature Limitation," 3/4.7.5, "Flood Level," 3/4.7.7, "Sealed Source Contamination," and related tables, figures, and Bases sections. The MP3 TSs proposed for relocation include 3/4.4.9.1, "Pressure/Temperature Limits," 3/4.7.2, "Steam Generator Pressure/Temperature Limitation," 3/4.7.6, "Flood Protection," 3/4.7.11, "Sealed Source Contamination," and corresponding tables, figures and Bases section.

The September 9, 2002, letter provided clarifying information that did not change the initial proposed no significant hazards consideration determination or expand the amendment beyond the scope of the initial notice.

2.0 BACKGROUND

Section 182a of the Atomic Energy Act of 1954 as amended, requires applicants for nuclear power plant operating licenses to include TSs as part of the license. The Commission's regulatory requirements related to the content of TSs are set forth in 10 CFR 50.36. The regulation requires that TSs include items in five specific categories, including (1) safety limits, limiting safety system settings and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. The regulation does not specify the particular requirements to be included in the TSs.

Additionally, 10 CFR 50.36(c)(2)(ii) sets forth four criteria to be used in determining whether a limiting condition for operation (LCO) is required to be included in TSs. The four criteria defined by 10 CFR 50.36 for determining whether a particular matter is required to be included in the TS LCOs, are as follows:

- (1) Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary,
- (2) a process variable, design feature, or operating restriction that is an initial condition of a design-basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier,
- (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design-basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier, and
- (4) a structure, system, or component which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

Existing TS requirements which fall within or satisfy any of the above criteria must be retained in the TSs; those requirements which do not fall within or satisfy these criteria may be relocated to other licensee-controlled documents.

3.0 EVALUATION

3.1 Pressure/Temperature Limits

TS LCO 3.4.9.1 provides the requirements for the reactor coolant system pressure and temperature limits in all MODES. Specifically, LCO 3.4.9.1 requires that the reactor coolant system (except the pressurizer) temperature, pressure, and heatup and cooldown rate be limited in accordance with limits specified in Table 4.4-2 and shown on Figures 3.4-2a and 3.4-2b for MP2, and Figures 3.4-2 and 3.4-3 for MP3. These requirements ensure that the reactor coolant system structural integrity remains within the design criteria. The licensee has proposed to relocate Surveillance Requirement (SR) 4.4.9.1b and Table 4.4-3 of MP2 and SR 4.4.9.1.2 and Table 4.4-5 of MP3 to the respective facility's TRM. SR 4.4.9.1b currently states:

The reactor vessel material irradiation surveillance specimens shall be removed and examined, to determine changes in material properties, at the intervals shown in Table 4.4-3. The results of these examinations shall be used to update Table 3.4-2 and Figures 3.4-2a and 3.4-2b.

SR 4.4.9.1.2 for MP3 has similar language to that discussed above. Table 4.4-3 of MP2 and Table 4.4-5 of MP3 provide the reactor vessel material irradiation surveillance schedule for each capsule.

Appendix H to Part 50 requires the implementation of a material surveillance program to monitor changes in the fracture toughness properties of ferritic materials in the reactor vessel

beltline region of light water nuclear power reactors which result from exposure of these materials to neutron irradiation and the thermal environment. Under the program, the fracture toughness test data are obtained from material specimens exposed in surveillance capsules, which are withdrawn periodically from the reactor vessel. These data will be used as described in Section IV of Appendix G to Part 50 to adjust pressure and temperature limits in the TS. SR 4.4.9.1b and SR 4.4.9.1.2, and the associated Tables 4.4-2 of MP2 and 4.4-5 of MP3, duplicate the requirements of Appendix H of Part 50. Therefore, neither SR is needed to assure that, pursuant to 10 CFR 50.36(c)(3) pertaining to surveillance requirements, the necessary quality of systems and components is maintained. In addition, the SRs do not assure that facility operation will be within safety limits or that LCOs will be met. Accordingly, SR 4.4.9.1b and SR 4.4.9.1.2, and the associated tables, are not required to be in the TS. Therefore, the Nuclear Regulatory Commission (the staff) finds the relocation of SR 4.4.9.1b and Table 4.4-3 of MP2 and SR 4.4.9.1.2 and Table 4.4-5 of MP3 to the respective facility's TRM to be acceptable.

3.2 Steam Generator Pressure/Temperature Limitation

The licensee has proposed to relocate existing conditions, actions, and surveillance requirements for the steam generator pressure/temperature limits in TS 3/4.7.2.1 for MP2 and TS 3/4.7.2 for MP3 to the TRM. These requirements define the limitations on steam generator pressure and temperature which ensure that the pressure induced stresses in the steam generators do not exceed the maximum allowed fracture toughness stress. As discussed in Section 5.2.3.3.1 of the MP3 Final Safety Analysis Report (FSAR), the fracture toughness properties of the reactor coolant pressure boundary components (which includes the steam generators), meet the requirements of the American Society of Mechanical Engineers Section III, paragraphs NB-2000, NC-2300 and ND-2300, as appropriate.

The staff evaluated the existing TS against the four criteria set forth in 10 CFR 50.36(c)(2)(ii). Steam generator pressure and temperature limits are not a form of instrumentation nor a structure, system or component, and therefore, do not meet criteria 1, 3 or 4. The steam generator pressure and temperature limits are operating restrictions. However, these operating restrictions are not an initial condition for a design basis accident or transient analysis. Therefore, steam generator pressure and temperature limits do not meet criterion 2 for inclusion in the TSs. Since TS 3/4.7.2.1 and TS 3/4.7.2, for MP2 and MP3, respectively, do not satisfy these criteria, TS 3/4.7.2.1 and TS 3/4.7.2 may be relocated to other licensee-controlled documents.

3.3 Flood Level/Flood Protection

The existing conditions, actions, and surveillance requirements for flood level/flood protection in TS 3/4.7.5.1 for MP2 and TS 3/4.7.6 for MP3 will be relocated to the TRM. TS 3/4.7.5.1 ensures that one service water pump motor will be protected against flooding to a minimum elevation of 28 feet Mean Sea Level. TS 3/4.7.6 ensures that the service water pump cubicle watertight doors will be closed and the pump cubicle sump drain valves will be closed before the water level reaches the critical elevation of 14.5 feet Mean Sea Level.

The staff evaluated the existing TS against the four criteria set forth in 10 CFR 50.36(c)(2)(ii). Flood level/flood protection requirements are not a form of instrumentation nor a structure, system or component, and therefore, do not meet criteria 1, 3 or 4. The flood level/flood

protection requirements are operating restrictions. However, these operating restrictions are not an initial condition for a design basis accident or transient analysis. Therefore, flood level/flood protection requirements do not meet criterion 2 for inclusion in the TSs. Since TS 3/4.7.5.1 and TS 3/4.7.6, for MP2 and MP3 respectively, do not satisfy these criteria, TS 3/4.7.5.1 and TS 3/4.7.6 may be relocated to other licensee-controlled documents.

3.4 Sealed Source Contamination

The existing conditions, actions, and surveillance requirements for sealed source contamination in TS 3/4.7.7 for MP2 and TS 3/4.7.11 for MP3 will be relocated to the TRM. The existing LCO requires that each sealed source containing radioactive material either in excess of 100 microcuries of beta and/or gamma emitting material or 5 microcuries of alpha emitting material shall be free of greater than or equal to 0.005 microcuries of removable contamination. This limitation ensures that the total body or individual organ irradiation does not exceed allowable limits in the event of ingestion or inhalation of the source material.

The staff evaluated the existing TS against the four criteria set forth in 10 CFR 50.36(c)(2)(ii). Sealed Source Contamination requirements are not a form of instrumentation nor a structure, system or component, and therefore, do not meet criteria 1, 3 or 4. The sealed source contamination requirements are not process variables, design features, or operating restrictions that are an initial condition for a design basis accident or transient analysis. Therefore, sealed source contamination requirements do not meet criterion 2 for inclusion in the TSs. Since TS 3/4.7.7 and TS 3/4.7.11, for MP2 and MP3, respectively, do not satisfy these criteria, TS 3/4.7.7 and TS 3/4.7.11 may be relocated to other licensee-controlled documents.

The licensee has referenced the TRM in both MP2 and MP3 FSARs. As such, changes to the TRM would be controlled in accordance with approved station procedures and the requirements of 10 CFR 50.59. Therefore, the staff has determined that sufficient regulatory control over changes to the TRM exists and concludes that selected TSs, discussed above, may be relocated from the TSs to the licensee's TRM.

3.5 SUMMARY

The staff has reviewed the licensee's submittal and supporting documentation. Based on our review discussed above, the staff finds the proposed relocation of the selected TSs to the MP2 and MP3 TRMs to be acceptable. Additionally, the staff concludes that there is reasonable assurance that plant operation in this manner poses no undue risk to the health and safety of the public.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendments involve no

significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (67 FR 18645). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

7.0 REFERENCES

1. Price, J. A., Dominion Nuclear Connecticut, to USNRC, "Millstone Nuclear Power Station, Units 2 and 3 Technical Specifications Change Requests 2-21-01 and 3-18-01 Relocation of Selected Technical Specifications Related to the Reactor Coolant System and Plant Systems," February, 14, 2002.
2. Price, J. A., Dominion Nuclear Connecticut, to USNRC, "Millstone Nuclear Power Station, Units 2 and 3 Technical Specifications Change Requests 2-21-01 and 3-18-01 Relocation of Selected Technical Specifications Related to the Reactor Coolant System and Plant Systems," September 9, 2002.

Principal Contributor: K. Kavanagh

Date: January 2, 2003