



ACR PHYSICS TESTS IN ZED-2

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Presentation Outline

- **Preliminary comments about the ZED-2 experimental program in support of ACR**
- **List experiments to be performed plus physics phenomena addressed by activity**
- **As each experiment is listed a brief description of the basic procedure plus analysis will be included**



Preliminary Comments

- **ZED-2 (Zero Energy Deuterium reactor)**
- **Maximum power is 200 Watts (nominal)**
- **This corresponds to an average neutron flux of about 10^9 neutrons $\text{cm}^{-2} \text{s}^{-1}$**
- **Standard fuel charge for the ACR program is 300 CANFLEX bundles containing SEU**
- **Data obtained are to validate physics codes and associated nuclear data libraries used for design and licensing ACR**



Flux Map Experiments

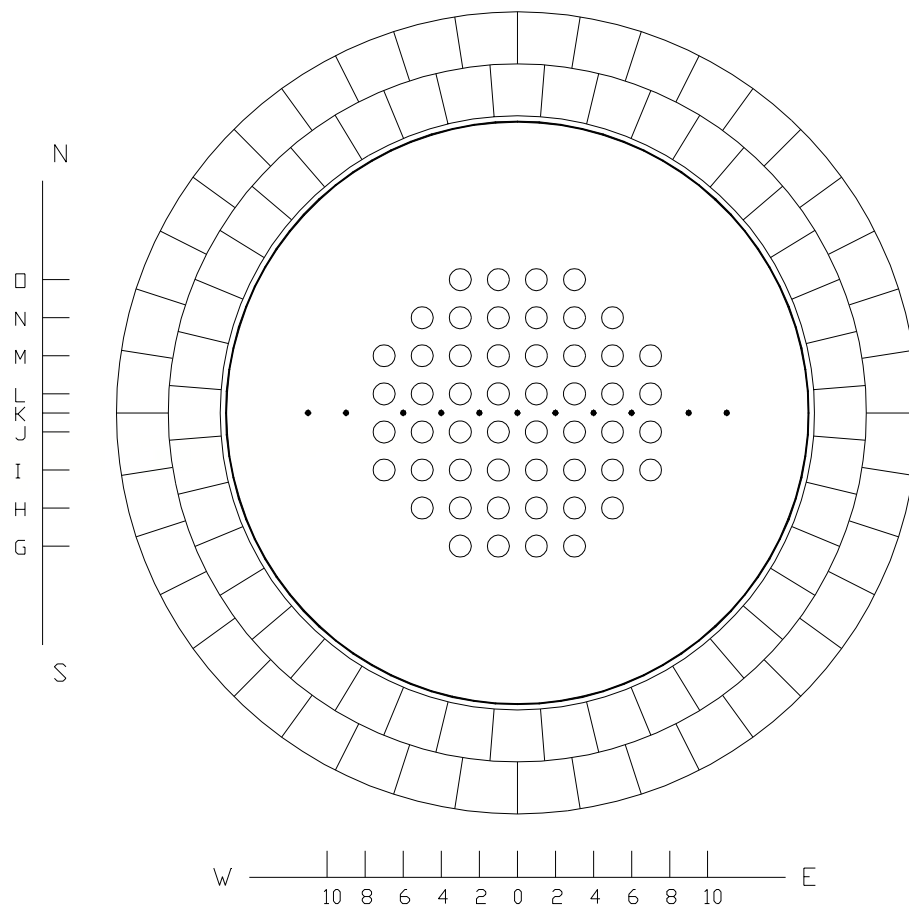
- **These experiments provide validation data related to lattice reactivity and coolant void reactivity (CVR)**
- **Physics Phenomena (PH0 PH1)**
- **Measurements are performed by positioning activation foils across a lattice of SEU**
- **The reactor is operated to activate the foils and the foils are counted to determine the global flux distribution across the lattice**
- **Measurements to be performed at various lattice spacings and at two coolant conditions—H₂O and air**



Flux-Map Set Up

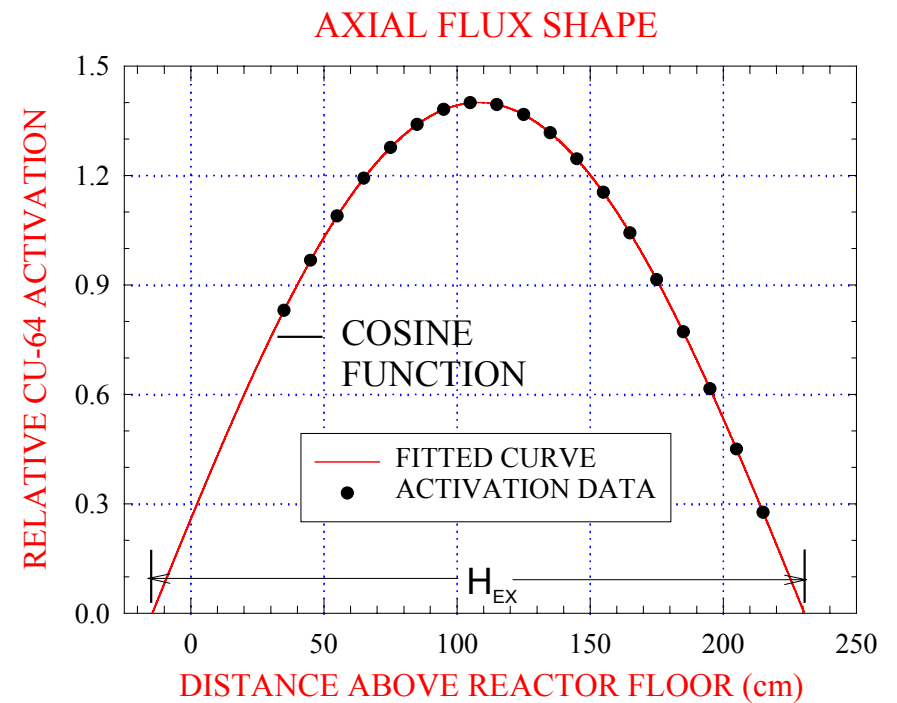
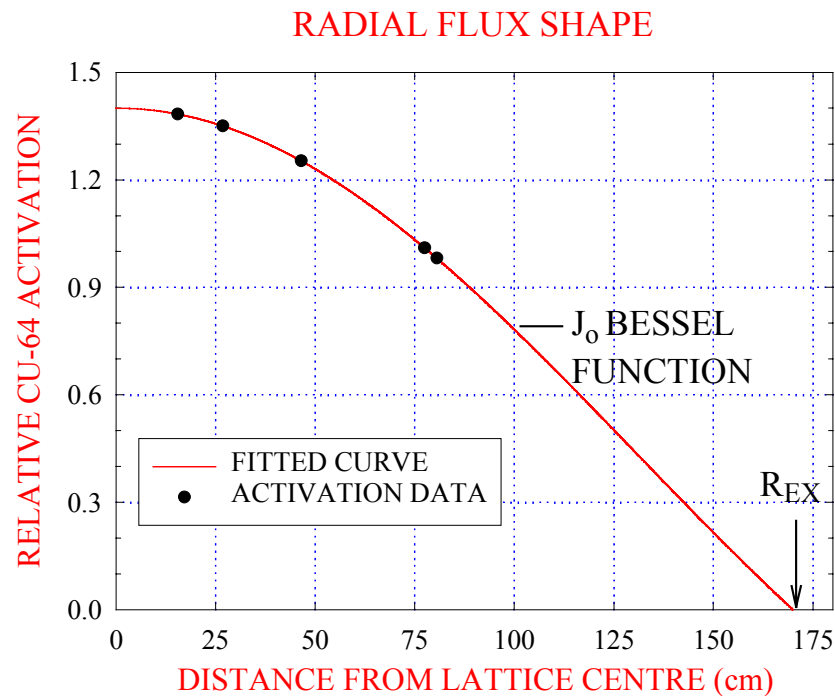
○ SEU Fuel

• Copper Foil Positions





ANALYSIS



$$\text{Buckling} = (2.405/R_{EX})^2 + (\pi/H_{EX})^2$$

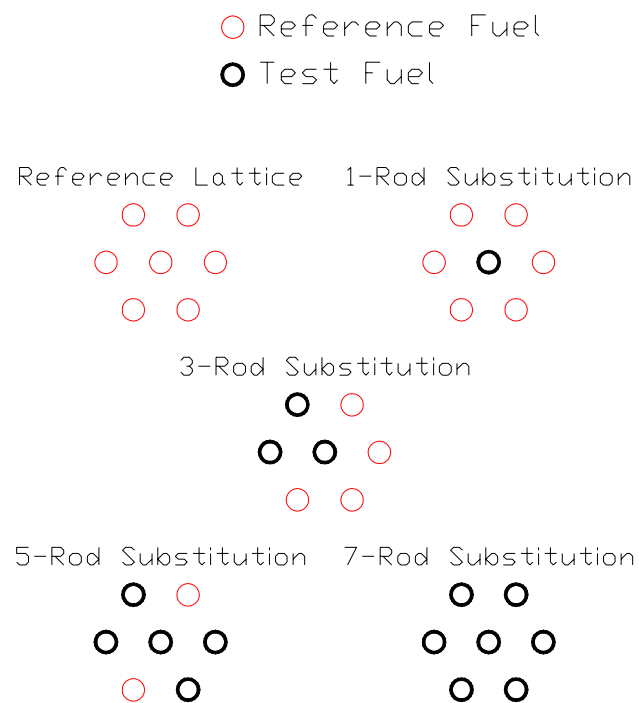
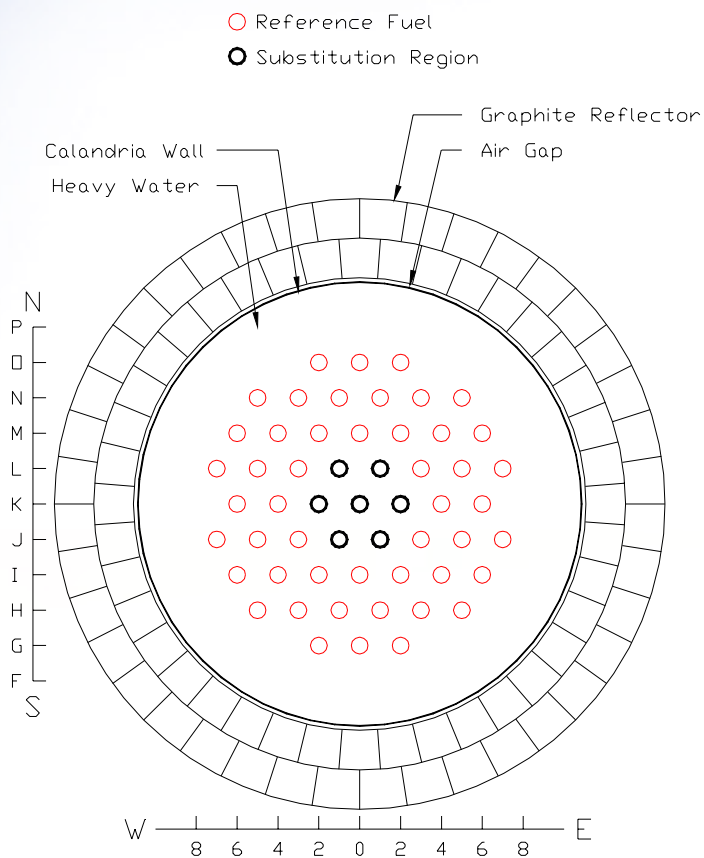


SUBSTITUTION EXPERIMENTS

- **These experiments provide validation data related to lattice reactivity and CVR for simulated irradiated fuel**
- **Physics Phenomena (PH0 PH1 PH8)**
- **The simulated irradiated fuel will be a mixed-oxide (MOX) containing Pu and depleted U**
- **Measurements are performed by systematically replacing reference fuel with test fuel while observing the resulting change in moderator critical height**
- **Measurements will be performed using two test fuels, SEU and MOX (representing fresh fuel and irradiated fuel) with the test fuel at two coolant conditions**

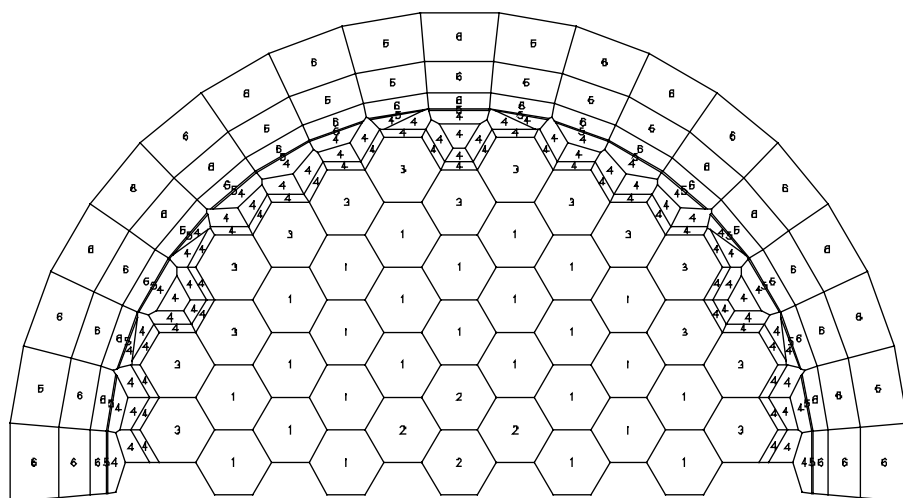


SUBSTITUTION SET UP





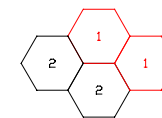
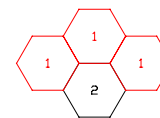
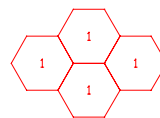
ANALYSIS



Legend

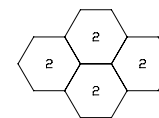
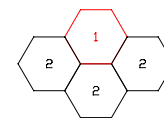
- Region 1 - Reference Channels
- Region 2 - Test Fuel Channels
- Region 3 - Heavy Water Reflector
- Region 4 - Heavy Water Reflector
- Region 5 - Calandria Wall
- Region 6 - Graphite Reflector

Reference Lattice 1-Rod Substitution 3-Rod Substitution



5-Rod Substitution

7-Rod Substitution





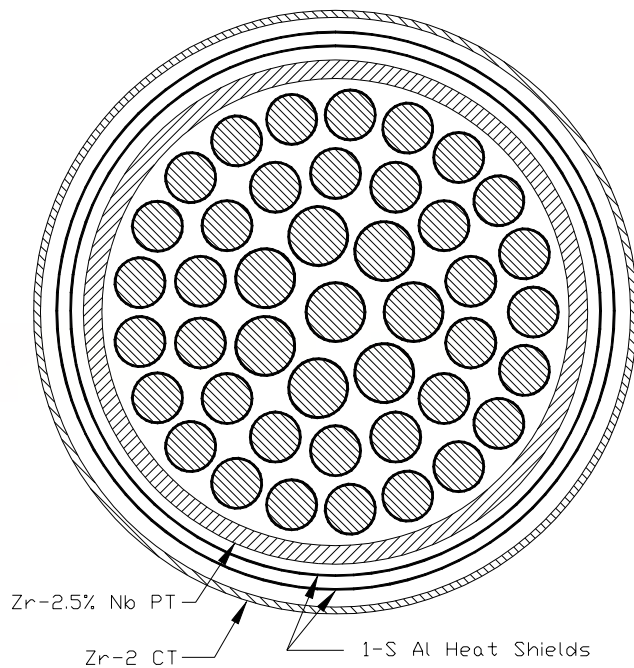
FUEL/COOLANT TEMPERATURE COEFF.

- **These experiments provide validation data for fuel-temperature coefficient and the effect on CVR vs channel temperature**
- **Physics Phenomena (PH0 PH1 PH2 PH7 PH8)**
- **Measurements are performed in a 7-rod substitution lattice with the test fuel contained in specially modified channels**
- **Measurements to be performed using both SEU and MOX (fresh fuel and irradiated fuel) and at two coolant conditions**

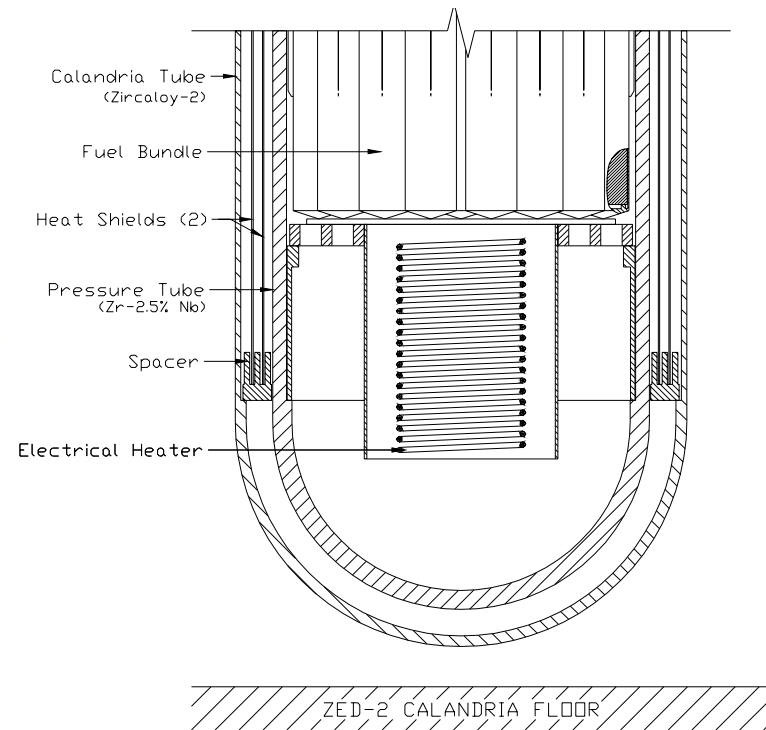


H.T. EXPERIMENTAL SET-UP

PLAN VIEW



BOTTOM SECTION





PROCEDURE AND ANALYSIS

- Channel contents are heated from room temperature to 300°C (572°F)
- As pre-selected temperatures are achieved the reactor is stabilized and the critical height and core conditions are recorded
- For the water-coolant tests a He cover gas is used to suppress boiling
- High pressure CO₂ gas (~500 psi) is used as the convective medium for the fuel-temperature coefficient tests
- A substitution-type analysis is performed using the moderator critical-height data obtained as the channel contents are heated to derive bucklings for the various temperatures and coolant conditions



MODERATOR TEMPERATURE COEFFICIENT

- **These experiments provide validation data for the effects on lattice reactivity and (CVR) with moderator temperature**
- **Physics Phenomena (PH0 PH1 PH3 PH4)**
- **Flux-map experiments will be performed using SEU lattices at moderator temperatures between room temperature and $\sim 40^{\circ}\text{C}$ (104°F)**

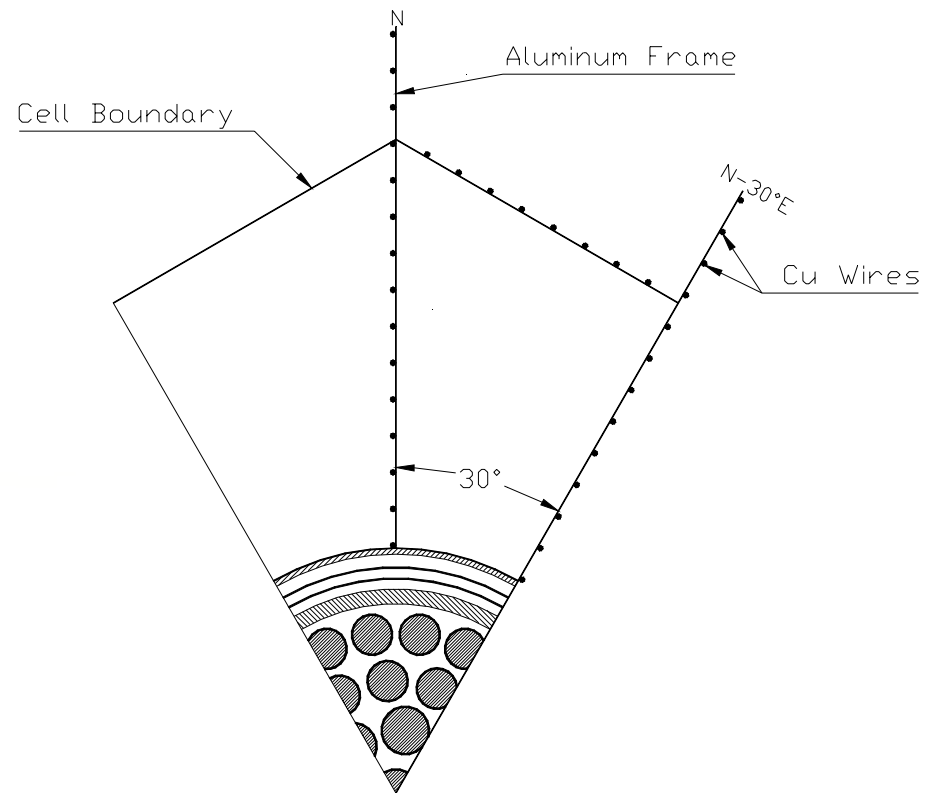
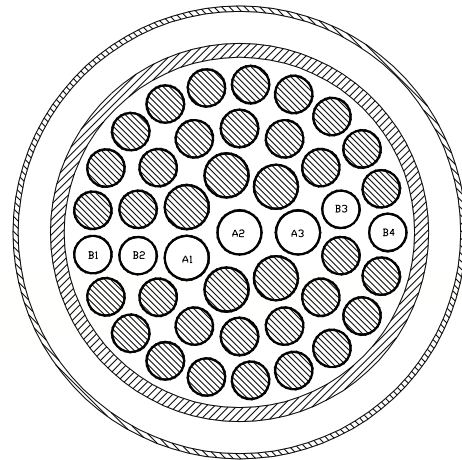
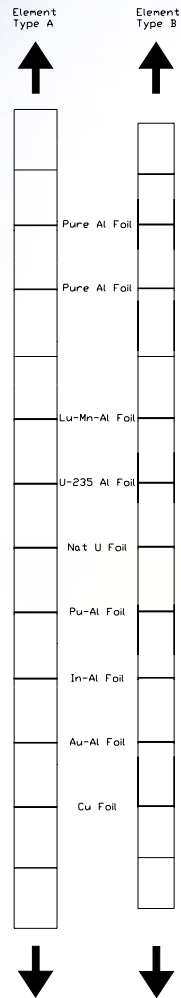


FINE-STRUCTURE REACTION RATES

- **These experiments will provide information about detailed neutron distributions in space and energy plus end-flux peaking effects**
- **Physics Phenomena (PH0 PH1 PH2 PH7 PH14)**
- **Activation foils are placed between the fuel pellets of a special demountable bundle**
- **The bundle is positioned in the center location of a seven-rod lattice and the reactor is operated to activate the foils**
- **Measurements will be done using SEU (fresh fuel) and MOX (simulated irradiated fuel) and at two coolant conditions**



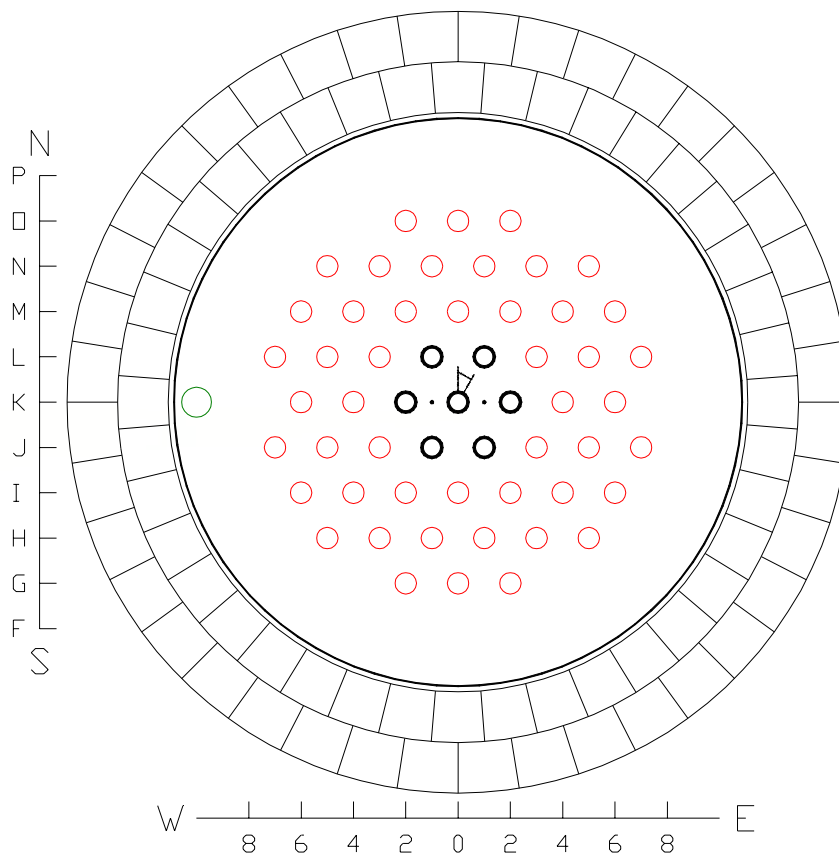
FINE STRUCTURE SET UP





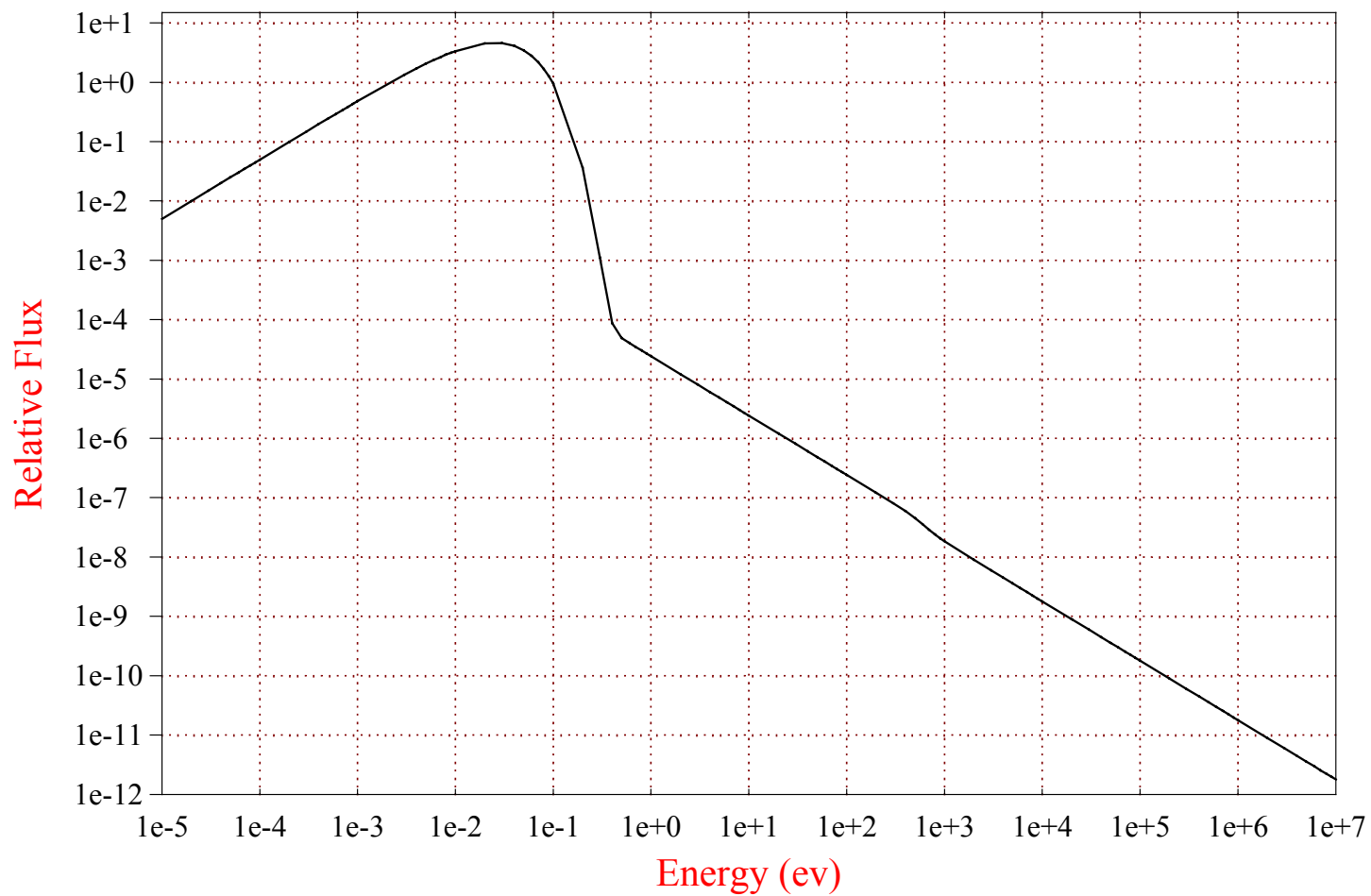
FINE STRUCTURE LATTICE

- Driver Channel
- Test Channels
- Reference Wheels (2)
- Al Frame and Stringers





REFERENCE WHEEL SPECTRUM





ANALYSIS

- **U-235, Cu-63, and Mn-55 are approximately 1/V absorbers having only small resonance integrals**
- **Au-197 and In-115 have large resonances in the epithermal region at 4.916 eV & 1.457 eV, respectively**
- **Pu-239, Lu-176 have large thermal resonances at 0.296 eV & 0.141 eV, respectively**
- **Reaction-rate ratios (normalized to the reference-wheel spectrum) are sensitive indicators of the energy spectra across the test cell (i.e. Pu-239/U-235 fission, Lu-176/Mn-55 capture, In-115/Cu-63 capture etc.)**
- **Activation data will be compared directly to lattice cell predictions**



MODERATOR POISON EXPERIMENTS

- These experiments are to provide validation data for the lattice reactivity and void reactivity effect of poisoning the moderator
- Physics phenomena (PH0 PH1 PH3 PH4)
- Flux map experiments will be performed using a SEU lattice with and without poison (e.g. Gd, B) added to the moderator and reflector
- The flux maps will be performed at two moderator temperatures and two coolant conditions



CHECKERBOARD LATTICE MEASUREMENTS

- **These measurements are to provide validation data for detailed radial cell-to-cell leakage in an ACR-type lattice representing bi-directional fueling**
- **Physics phenomena (PH0, PH8, PH14)**
- **Checkerboard region in the lattice will contain alternating SEU (fresh fuel) and MOX (irradiated fuel) channels**
- **Measurements will use various coolant conditions in the checkerboard region (e.g. all channels cooled, SEU cooled MOX voided, SEU voided MOX cooled, etc.)**

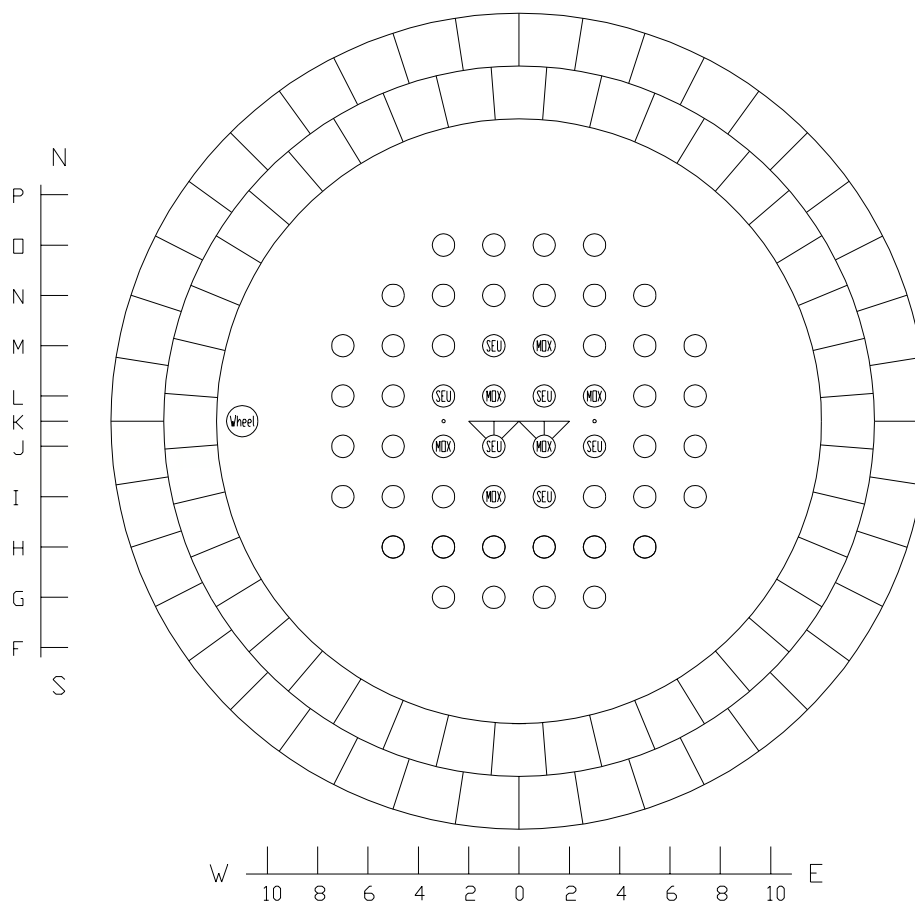


CHECKERBOARD LATTICE

○ SEU Driver Fuel

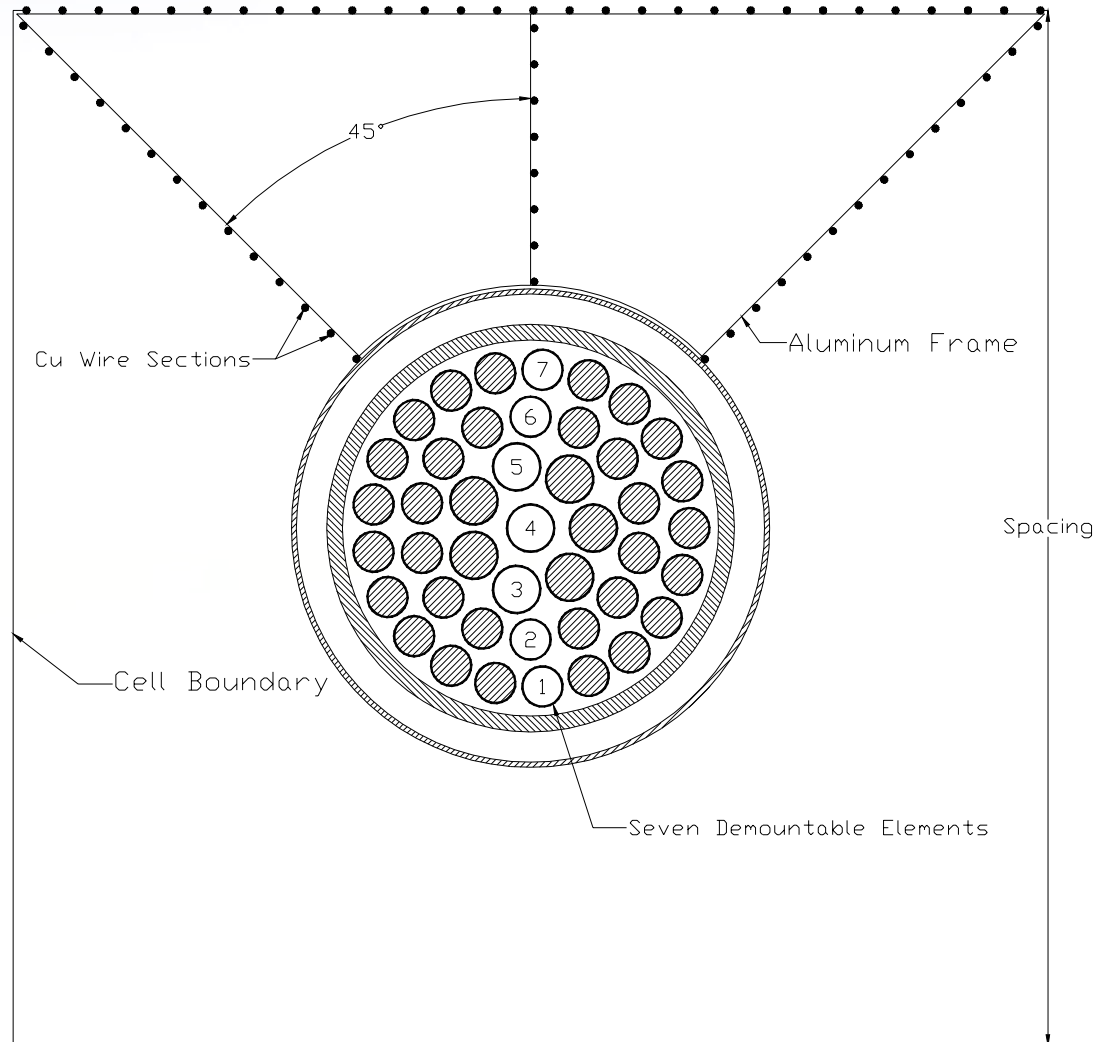
• Stringer

▽ Frame





TEST CELLS





ANALYSIS

- **Activation data will be compared directly to lattice-cell calculations and diffusion calculations to demonstrate the ability of the codes to predict detailed cell-to-cell neutron distributions**



CONTROL DEVICE MEASUREMENTS

- **These measurements are to demonstrate the ability of our physics toolset codes to calculate reactivity-device cell parameters**
- **Physics phenomena (PH0 PH11)**
- **The device will be inserted into the test region of the checkerboard lattice and fine-structure plus flux-map measurements will be performed**
- **The device will be inserted into a uniform lattice of SEU to derive the reactivity worth using period measurements**



ROD DROP EXPERIMENT

- This test is to determine the component of the delayed neutron fraction due to photo neutrons
- Physics phenomena (PH12)
- An SEU lattice will be set up in ZED-2 and after operating for a pre-determined period an absorber rod will be inserted into the lattice
- The resulting negative transient will be analyzed to determine the contribution of delayed photo neutrons to Beta through the reaction $D(\gamma, n)$ $E_{\gamma} \sim 2.225 \text{ MeV}$



ANALYSIS

Requirements for the analysis are:

- Neutron yield per fission of ^{235}U
- An effective delayed fraction and relative group yields and half-lives for ^{235}U direct delayed neutrons
- Neutron yield per fission of ^{238}U
- Relative numbers of fissions of ^{235}U and ^{238}U in the lattice
- An effective delayed fraction and relative group yields and half-lives for ^{238}U direct delayed neutrons
- Relative group yields and half-lives for delayed photo-neutrons
- Relative yields of delayed photo-neutrons for ^{235}U and ^{238}U
- The effective yield of delayed photo-neutrons for ^{235}U and ^{238}U (allowed to vary in the analysis)



SUMMARY

- **ZED-2 tests are to provide validation data for the physics codes and associated nuclear data libraries used in the design and licensing of ACR**
- **The presentation has listed the experiments planned to address various physics phenomena identified in the Physics Code Qualification Plan for ACR**
- **Experimental techniques to be employed are flux maps, substitution experiments, fine-structure experiments and kinetics experiments**



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