



Portland General Electric Company

April 23, 1990

Trojan Nuclear Plant  
Docket 50-344  
License NPF-1

Director, Office of Nuclear Material  
Safety and Safeguards  
Attn: Mr. Charles E. MacDonald, Chief  
Transportation Branch  
Division of Safeguards and Transportation  
U.S. Nuclear Regulatory Commission  
Washington DC 20555

Dear Sir:

Packaging and Transportation of Radiation Material

In accordance with Chapter 10, Code of Federal Regulations, Part 71.12 (10 CFR 71.12) Portland General Electric Company (PGE) requests the addition of Trojan Nuclear Plant, License NPF-1, to the list of users for NRC Certificate of Compliance for Radioactive Materials Packages Number 6206, Package Identification Number USA/6206/AF. This Certificate was issued to the Babcock and Wilcox (B&W) Fuel Company under Docket Number 71-6206.

It is our understanding that 10 CFR 71.12 grants a general license to all NRC licensees to use a NRC licensed container providing the requirements of 10 CFR 71 Subpart H, Quality Assurance, are satisfied (i.e., the general licensee has a previously approved Quality Assurance Program and the NRC is notified prior to initial use). The NRC has previously approved Topical

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Mr. Charles E. MacDonald  
April 23, 1990  
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Report PGE-8010, Trojan Nuclear Plant "Nuclear Quality Assurance Program" for Radioactive Material Packages (No. 0327 dated August 1, 1989). A copy of the NRC Certificate of Compliance for the B&W container and the NRC approval of the PGE Quality Assurance Program is attached. PGE has not used the B&W shipping containers at the present time but may have to use them for a return shipment of new fuel in 1991.

Sincerely,



T. D. Walt  
Acting Vice President, Nuclear

Attachment

c: U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington DC 20555

Mr. John B. Martin  
Regional Administrator, Region V  
U.S. Nuclear Regulatory Commission

Mr. David Stewart-Smith  
State of Oregon  
Department of Energy

Mr. R. C. Barr  
NRC Resident Inspector  
Trojan Nuclear Plant

CERTIFICATE OF COMPLIANCE  
FOR RADIOACTIVE MATERIALS PACKAGES

1. a. CERTIFICATE NUMBER 6206	b. REVISION NUMBER 11	c. PACKAGE IDENTIFICATION NUMBER USA/6206/AF	d. PAGE NUMBER 1	e. TOTAL NUMBER PAGES 2
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2. PREAMBLE

- a. This certificate is issued to certify that the packaging and contents described in Item 5 below, meets the applicable safety standards set forth in Title 10, Code of Federal Regulations, Part 71, "Packaging and Transportation of Radioactive Material."
- b. This certificate does not relieve the consignor from compliance with any requirement of the regulations of the U.S. Department of Transportation or other applicable regulatory agencies, including the government of any country through or into which the package will be transported.

3. THIS CERTIFICATE IS ISSUED ON THE BASIS OF A SAFETY ANALYSIS REPORT OF THE PACKAGE DESIGN OR APPLICATION

a. ISSUED TO (Name and Address) B & W Fuel Company P.O. Box 11646 Lynchburg, VA	b. TITLE AND IDENTIFICATION OF REPORT OR APPLICATION: Babcock & Wilcox Company application dated April 10, 1985, as supplemented.
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c. DOCKET NUMBER 71-6206

4. CONDITIONS

This certificate is conditional upon fulfilling the requirements of 10 CFR Part 71, as applicable, and the conditions specified below.

5.

(a) Packaging

- (1) Model No.: Model B
- (2) Description

A fuel assembly shipping container consisting of a steel strongback clamping assembly, shock mounted to a steel outer container. Two, 3/16-inch thick, 8-5/8-inch high and full-length stainless steel plates containing 1.5% minimum boron are positioned between adjacent fuel assemblies. The outer container is approximately 40 inches in diameter by 200 inches long. Gross weight of loaded container not to exceed 7,600 pounds.

(3) Drawings

The container is constructed in accordance with Babcock and Wilcox Company Drawing Nos. PE-52F, Revision 3; PE-53F, Revision 3; and PE-54F, Revision 2.

(b) Contents

(1) Type and form of material

UO<sub>2</sub> sintered pellet diameters ranging from 0.30 inch to 0.60 inch. Uranium may be enriched to a maximum 4.05 w/o in the U-235 isotope. The pellets are clad in minimum 0.020-inch thick zircalloy or minimum 0.016-inch thick stainless steel rods. The rods are assembled into fuel assemblies with a maximum cross section of 8.6 inches by 8.6 inches. The void volume within the fuel assembly must not exceed two times the volume of UO<sub>2</sub> in the fuel assembly. The fuel assemblies may contain inserted control rod assemblies.

(2) Maximum quantity of material per package

Two fuel assemblies containing not more than 18.8 kg U-235 per assembly.

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|----|---|------------|
| 5. | (c) Fissile Class   | II and III |
|    | (1) Minimum transport index<br>to be shown on label for<br>Class II | 6.3        |
|    | (2) Maximum number of packages<br>per shipment for Class III        | 20         |
6. Each fuel assembly must be unsheathed or must be enclosed in an unsealed, polyethylene sheath which will not extend beyond the ends of the fuel assembly. The ends of the sheath must not be folded or taped in any manner that would prevent the flow of liquids into or out of the sheathed fuel assembly.
  7. There must be a clamp bow to restrain each spacer grid and end fitting. The ratio of assembly weight to the number of clamp bows must not exceed 168 pounds per clamp.
  8. The weight of the contents (fuel assemblies, control rods, spacers, etc.) must not exceed 3,360 pounds.
  9. Fabrication of additional packagings is not authorized.
  10. The package authorized by this certificate is hereby approved for use under the general license provisions of 10 CFR 71.12.
  11. Expiration date: May 31, 1990.

REFERENCES

Babcock & Wilcox Company application dated April 10, 1985.

Supplements dated: September 22, 1986; February 18, April 15, May 17, and July 15, 1988; and July 13, 1989.

FOR THE U.S. NUCLEAR REGULATORY COMMISSION

*Charles E. MacDonald*  
 Charles E. MacDonald, Chief  
 Transportation Branch  
 Division of Safeguards  
 and Transportation, NMSS

Date: OCT 05 1989



**ATTACHMENT 2**  
UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUKE COCKFIELD

AUG 7 1989

AUG 01 1989

SGTB:LLG  
71-0327

Portland General Electric Company  
ATTN: Mr. David W. Cockfield, Vice President  
121 SW Salmon Street  
Portland, OR 97204

Gentlemen:

Enclosed is Quality Assurance Program Approval for Radioactive Material Packages  
No. 0327, Revision No. 3.

Please note the conditions included in the approval.

Sincerely,

*Charles E. MacDonald*  
Charles E. MacDonald, Chief  
Transportation Branch  
Division of Safeguards  
and Transportation, NMSS

Enclosure:  
As stated

Copies to: Seaman, Meek, T. Price, TNP:GOV REL F:NRC CHRONO, NRC TO PGE,  
TNP:POW ST OP 7-5:Radioactive Transportation

R.M. NELSON

AUG 29 '89 ①

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QUALITY ASSURANCE PROGRAM APPROVAL  
FOR RADIOACTIVE MATERIAL PACKAGES

0327

REVISION NUMBER

3

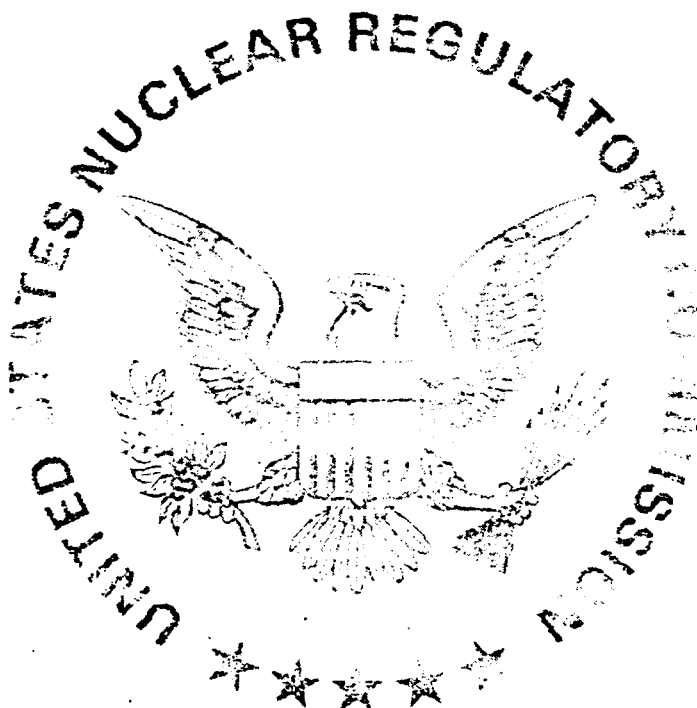
Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974, as amended, and Title 10, Code of Federal Regulations, Chapter 1, Part 71, and in reliance on statements and representations heretofore made in Item 5 by the person named in Item 2, the Quality Assurance Program identified in Item 5 is hereby approved. This approval is issued to satisfy the requirements of Section 71.101 of 10 CFR Part 71. This approval is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

2. NAME <b>Portland General Electric Company</b>			3. EXPIRATION DATE <b>August 31, 1994</b>	
STREET ADDRESS <b>121 SW Salmon Street</b>			4. DOCKET NUMBER <b>71-0327</b>	
CITY <b>Portland</b>	STATE <b>OR</b>	ZIP CODE <b>97204</b>		

5. QUALITY ASSURANCE PROGRAM APPLICATION DATE(S)  
**May 10, 1979 and July 14, 1989**

6. CONDITIONS

Activities conducted under applicable criteria of Appendix B to 10 CFR Part 50 for operations at the Trojan Nuclear Power Plant (NRC Docket No. 50-344) to be executed in accordance with Quality Assurance Program PGE-8010, as amended, and included as Chapter 17.2 of the FSAR.



*Charles E. MacDonald*  
FOR THE U.S. NUCLEAR REGULATORY COMMISSION  
Charles E. MacDonald

AUG 01 1989

CHIEF, TRANSPORTATION BRANCH  
DIVISION OF SAFEGUARDS AND TRANSPORTATION  
OFFICE OF NUCLEAR MATERIAL SAFETY AND SAFEGUARDS

DATE

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