

AmerGen Energy Company, LLC
Oyster Creek
US Route 9 South
PO Box 388
Forked River, NJ 08731-0388

10 CFR 50.73(a)(2)(i)

December 4, 2002
2130-02-20334

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington DC 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Licensee Event Report 02-002
Local Leak Rate Test Results in Excess
of Technical Specification Limits

Enclosed is Licensee Event Report LER 02-002. This event did not affect the health and safety of the public.

If any additional information or assistance is required, please contact Mr. John Rogers of my staff at 609.971.4893.

Very truly yours,



Ron J. DeGregorio
Vice President, Oyster Creek

RJD/JJR
cc: Administrator, Region I
NRC Project Manager
Senior Resident Inspector

JBJ22

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST 500 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Oyster Creek Unit 1

DOCKET NUMBER (2)

05000 - 219

PAGE (3)

1 of 3

TITLE (4)

Local Leak Rate Test Results in Excess of Technical Specification Limits

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
10	11	02	02	-- 02	-- 00					05000
									FACILITY NAME	DOCKET NUMBER
										05000

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)						
		20.2201(b)		20.2203(a)(2)(v)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)
POWER LEVEL (10)	O	20.2203(a)(1)		20.2203(a)(3)(i)		50.73(a)(2)(ii)		50.73(a)(2)(x)
		20.2203(a)(2)(i)		20.2203(a)(3)(ii)		50.73(a)(2)(iii)		73.71
		20.2203(a)(2)(ii)		20.2203(a)(4)		50.73(a)(2)(iv)		OTHER
		20.2203(a)(2)(iii)		50.36(c)(1)		50.73(a)(2)(v)		
		20.2203(a)(2)(iv)		50.36(c)(2)		50.73(a)(2)(vii)		

LICENSEE CONTACT FOR THIS LER (12)

NAML

John Rogers

TELEPHONE NUMBER (Include Area Code)

609.971.4893

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)

X NO

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On October 11, 2002, Local Leak Rate Test results indicated that Main Steam Isolation Valve V-1-0007 exceeded the Technical Specification leak rate limit of .05(.75) L_a at 35 psig (equivalent to 15.98 SCFH). The leak was unable to be quantified and was greater than 50 SCFH at 35 psig.

The cause of this occurrence was attributed to component degradation.

The safety significance of this occurrence is considered minimal as the total penetration leakage would have been limited by Main Steam Isolation Valve V-1-0009 in the same header. The leakage past V-1-009 was quantified at 2.214 SCFH.

V-1-0007 was refurbished and successfully leak rate tested prior to restart from the 1R19 Refueling Outage.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1)	DOCKET (2)	LER NUMBER (6)			PAGE (3)
Oyster Creek, Unit 1	05000	YEAR	SEQUENTIAL NUMBER	REV	2 of 3
	-219	02	-- 02	-- 00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

DATE OF DISCOVERY

The condition being reported was discovered on October 11, 2002.

IDENTIFICATION OF OCCURRENCE

Main Steam Isolation Valve (MSIV) V-1-0007 (EII SB-ISB) exceeded the leak rate criteria specified in Technical Specification 4.5.D.2. This condition is considered to be reportable in accordance with 10 CFR 50.73(a)(2)(i).

CONDITIONS PRIOR TO DISCOVERY

The plant was in a COLD SHUTDOWN condition for refueling outage 1R19 when this condition was discovered.

DESCRIPTION OF OCCURRENCE

On October 11, 2002, Local Leak Rate Testing (LLRT) results indicated that Main Steam Line Isolation Valve V-1-0007 exceeded the Technical Specification limit of .05(.75)L_s at 35 psig (equivalent to 15.98 SCFH). The leak could not be quantified, but was in excess of 50 SCFH at 35 psig.

APPARENT CAUSE OF OCCURRENCE

The apparent cause of this occurrence was component degradation. This MSIV had previously exceeded Technical Specification limits during the 14R refueling outage, but had successfully passed LLRT in 15R, 16R, 17R and 18R.

LICENSEE EVENT REPORT (LER)
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ANALYSIS OF OCCURRENCE AND SAFETY SIGNIFICANCE

The MSIVs are containment isolation valves designed to minimize the coolant loss from the vessel, and the resultant offsite dose, in the event of a main steamline break accident. The design basis loss of coolant accident was evaluated for the primary containment maximum allowable accident leak rate of 1.0% per day at an initial pressure of 35 psig, which decays to 1.0 psig after 2.5 hours. The 1.0 psig is assumed to remain for the next 21.5 hours. The calculated leakage provides adequate margin between projected potential offsite dose and 10 CFR 100 guidelines. This projected dose was not exceeded.

The analysis for the contribution of MSIV leakage to control room habitability was reviewed. The MSIV contribution is approximately 25% (243 SCF) of the total radioactive leakage assumed (1000 SCF). The minor increase in leakage rate from the V-1-007 penetration is well bounded by the existing total radioactive leakage and, therefore, has no impact on control room habitability.

The safety significance of this event is considered minimal. The leakage past the MSIV would have been limited by the leak rate of the other MSIV in the same header which met the leak rate acceptance criteria of Technical Specification 4.5.D.2.

CORRECTIVE ACTIONS

The MSIV components were repaired and retested with satisfactory results prior to restart from the 1R19 outage.

SIMILAR EVENTS

LER 00-0010; Local Leak Rate Test Results in Excess of Technical Specification Limits

Oyster Creek Licensing Correspondence Distribution Sheet

File No. 02082
Reference/Letter No. 2130-02-20334

Letter Date 12/04/2002
Date Sent / Received 12/06/2002

Title Description: LER 2002-002; Test Results in Excess of Tech Spec Limits

LICENSING ENGINEER: John Rogers

SPECIAL NOTES: Distributed without attachment except as noted.

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