

ENERGY NORTHWEST

P.O. Box 968 ■ Richland, Washington 99352-0968

November 27, 2002
GO2-02-192

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Subject: **COLUMBIA GENERATING STATION, DOCKET NO. 50-397
REQUEST FOR AMENDMENT TO TECHNICAL SPECIFICATION 4.2.1
AND 5.6.5.b, CORRECTIONS TO ORIGINAL SUBMITTAL**

Reference Letter GO2-02-138, dated September 3, 2002, RL Webring (Energy Northwest) to NRC, "Request for Amendment to Technical Specification 4.2.1 and 5.6.5.b"

Dear Sir or Madam:

The referenced letter requested an amendment to the Columbia Generating Station Technical Specifications. Subsequent to the transmittal, Energy Northwest has identified corrections that should be made to the submittal. These corrections are administrative and do not affect any of the technical aspects of the amendment request.

Accordingly, you are requested to replace the first insert page in the markup of current Technical Specification pages (Reference letter, Attachment 1) and two of the revised (typed) pages (Reference letter, Attachment 2) in accordance with the instructions provided in Attachment 1 to this letter.

The Energy Northwest staff regrets any inconvenience these discrepancies may have caused.

In accordance with 10 CFR 50.91, a copy of this application, with attachments, is being provided to the designated Washington State Official.

A001

**REQUEST FOR AMENDMENT TO TECHNICAL SPECIFICATION 4.2.1 AND 5.6.5.B,
CORRECTIONS TO ORIGINAL SUBMITTAL**

Page 2

If you have any questions or require additional information regarding this matter, please contact Ms. CL Perino, Licensing Manager at (509) 377-2075.

Respectfully,



DW Coleman, Manager
Performance Assessment and Regulatory Programs
Mail Drop PE20

Enclosure:

1. Notarized Affidavit

Attachments:

1. Revision Instructions
2. Revised Markup of first insert page for Technical Specification 5.6.5.b
3. Revised Markup of Typed Pages
4. Typed Revised Technical Specification Pages

cc: EW Merschoff – NRC RIV
BJ Benney – NRC NRR
NRC Resident Inspector – 988C
DL Williams – BPA/1399
TC Poindexter – Winston & Strawn
JO Luce – EFSEC

ENCLOSURE 1, Notarized Affidavit

STATE OF WASHINGTON)
)
COUNTY OF BENTON)

Subject: Amendment to Technical
Specification 4.2.1 and
5.6.5.b, Corrections to
Original Submittal

I, DW COLEMAN, being duly sworn, subscribe to and say that I am the Manager, Performance Assessment and Regulatory Programs for ENERGY NORTHWEST, the applicant herein; that I have the full authority to execute this oath; that I have reviewed the foregoing; and that to the best of my knowledge, information, and belief the statements made in it are true.

DATE November 27, 2002

DW Coleman
DW Coleman, Manager
Performance Assessment and Regulatory Programs

On this date personally appeared before me DW Coleman, to me known to be the individual who executed the foregoing instrument, and acknowledged that he signed the same as his free act and deed for the uses and purposes herein mentioned.

GIVEN under my hand and seal this 27th day of November 2002.

Lori A. Walli
Notary Public in and for the
STATE OF WASHINGTON



Residing at Benton County
My Commission Expires 3-29-05

REQUEST FOR AMENDMENT TO TECHNICAL SPECIFICATION 4.2.1 AND 5.6.5.b, CORRECTIONS TO ORIGINAL SUBMITTAL

Attachment 1

Instructions for revisions to referenced request for amendment to Technical Specifications, dated September 3, 2002

1. Action:

For Attachment 1 (Markup of Current Technical Specifications pages), replace the first insert page for the markup of TS 5.6.5.b, page 5.6-3 with enclosed revised page.

Description of Changes:

- a. Insert 1, Added a closing parenthesis to the end of the paragraph.
- b. Insert 3, Reference 11, the document date, August 1990, was deleted to make reference consistent with implementation of TSTF-363 as proposed in the amendment request.

2. Action:

For Attachment 2 (Revised (Typed) TS Pages), replace typed pages 5.6-3 and 5.6-4 with enclosed revised pages.

Description of Changes:

The following changes were made to the previously submitted 'typed' Technical Specification pages:

- a. Page 5.6-3, TS 5.6.5.b, second paragraph. Added a closing parenthesis to make the typed pages consistent with the change described in item 1.a above.
- b. Page 5.6-4, TS 5.6.5.b, Reference 10. Revised typed pages to match the information on the marked up pages provided in original Attachment 1 and corrected document number by deleting 'NF' from the number.
- c. Page 5.6-4, TS 5.6.5.b, Reference 11. Deleted the date, August 1990 to make the reference on the typed pages consistent with the change described in item 1.b above.
- d. Page 5.6-4, TS 5.6.5.b, Reference 18. Revised typed pages to match the information on the marked up pages provided in original Attachment 1.
- e. Page 5.6-4, TS 5.6.5.b, Reference 19. Revised typed pages to match the information on the marked up pages provided in Original Attachment 1.

**REQUEST FOR AMENDMENT TO TECHNICAL SPECIFICATION 4.2.1 AND
5.6.5.b, CORRECTIONS TO ORIGINAL SUBMITTAL**
Attachment 2

Revised Markup of first insert page for Technical Specification 5.6.5.b

TS 5.6.5(b)

INSERT 1

Identify the Topical Reports(s) by number and title or identify the staff Safety Evaluation Report for a plant specific methodology by NRC letter and date. The COLR will contain the complete identification for each of the TS referenced topical reports used to prepare the COLR (i.e., report number, title, revision, date and any supplements).

INSERT 2

1. XN-NF-81-58(P)(A), "RODEX2 Fuel Rod Thermal-Mechanical Response Evaluation Model", Exxon Nuclear Company
2. XN-NF-85-67(P)(A), "Generic Mechanical Design for Exxon Nuclear Jet Pump BWR Reload Fuel", Exxon Nuclear Company
3. EMF-85-74(P) Supplement 1(P)(A) and Supplement 2(P)(A), "RODEX2A (BWR) Fuel Rod Thermal-Mechanical Evaluation Model", Siemens Power Corporation
4. ANF-89-98(P)(A), "Generic Mechanical Design Criteria for BWR Fuel Designs", Advanced Nuclear Fuels Corporation
5. XN-NF-80-19(P)(A) Volume 1, "Exxon Nuclear Methodology for Boiling Water Reactors - Neutronic Methods for Design and Analysis", Exxon Nuclear Company
6. XN-NF-80-19(P)(A) Volume 4, "Exxon Nuclear Methodology for Boiling Water Reactors: Application of the ENC Methodology to BWR Reloads", Exxon Nuclear Company
7. EMF-2158(P)(A), "Siemens Power Corporation Methodology for Boiling Water Reactors: Evaluation and Validation of CASMO-4/MICROBURN-B2", Siemens Power Corporation
8. XN-NF-80-19(P)(A) Volume 3, "Exxon Nuclear Methodology for Boiling Water Reactors, THERMEX: Thermal Limits Methodology Summary Description", Exxon Nuclear Company
9. XN-NF-84-105(P)(A) Volume 1, "XCOBRA-T: A Computer Code for BWR Transient Thermal-Hydraulic Core Analysis", Exxon Nuclear Company

INSERT 3

11. ANF-913(P)(A) Volume 1 "COTRANSA2: A Computer Program for Boiling Water Reactor Transient Analysis", Advanced Nuclear Fuels Corporation, August 1990
12. ANF-1358(P)(A) "The Loss of Feedwater Heating Transient in Boiling Water Reactors", Advanced Nuclear Fuels Corporation
13. EMF-2209(P)(A), "SPCB Critical Power Correlation", Siemens Power Corporation
14. EMF-2245(P)(A), "Application of Siemens Power Corporation's Critical Power Correlations to Co-Resident Fuel", Siemens Power Corporation
15. EMF-2361(P)(A), "EXEM BWR-2000 ECCS Evaluation Model", Framatome ANP Richland

**REQUEST FOR AMENDMENT TO TECHNICAL SPECIFICATION 4.2.1 AND
5.6.5.b, CORRECTIONS TO ORIGINAL SUBMITTAL**
Attachment 3

Revised Markup of Typed Pages

5.6 Reporting Requirements

5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

- b. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents:

Identify the Topical Reports(s) by number and title or identify the staff Safety Evaluation Report for a plant specific methodology by NRC letter and date. The COLR will contain the complete identification for each of the TS referenced topical reports used to prepare the COLR (i.e., report number, title, revision, date and any supplements).

1. XN-NF-81-58(P)(A), "RODEX2 Fuel Rod Thermal-Mechanical Response Evaluation Model", Exxon Nuclear Company
2. XN-NF-85-67(P)(A), "Generic Mechanical Design for Exxon Nuclear Jet Pump BWR Reload Fuel", Exxon Nuclear Company
3. EMF-85-74(P) Supplement 1(P)(A) and Supplement 2(P)(A), "RODEX2A (BWR) Fuel Rod Thermal-Mechanical Evaluation Model", Siemens Power Corporation
4. ANF-89-98(P)(A), "Generic Mechanical Design Criteria for BWR Fuel Designs", Advanced Nuclear Fuels Corporation
5. XN-NF-80-19(P)(A) Volume 1, "Exxon Nuclear Methodology for Boiling Water Reactors - Neutronic Methods for Design and Analysis", Exxon Nuclear Company
6. XN-NF-80-19(P)(A) Volume 4, "Exxon Nuclear Methodology for Boiling Water Reactors: Application of the ENC Methodology to BWR Reloads", Exxon Nuclear Company
7. EMF-2158(P)(A), "Siemens Power Corporation Methodology for Boiling Water Reactors: Evaluation and Validation of CASMO-4/MICROBURN-B2", Siemens Power Corporation
8. XN-NF-80-19(P)(A) Volume 3, "Exxon Nuclear Methodology for Boiling Water Reactors, THERMEX: Thermal Limits Methodology Summary Description", Exxon Nuclear Company

(continued)

5.6 Reporting Requirements

5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

9. XN-NF-84-105(P)(A) Volume 1, "XCOBRA-T: A Computer Code for BWR Transient Thermal-Hydraulic Core Analysis", Exxon Nuclear Company

10. ANF-~~NF-524~~(P)(A), Revision 2 and Supplements 1 and 2; Advanced Nuclear Fuels Corporation Critical Power Methodology for Boiling Water Reactors, November 1990;

MOVE

11. ANF-913(P)(A) Volume 1 "COTRANSA2: A Computer Program for Boiling Water Reactor Transient Analysis", Advanced Nuclear Fuels Corporation, August 1990

12. ANF-1358(P)(A) "The Loss of Feedwater Heating Transient in Boiling Water Reactors", Advanced Nuclear Fuels Corporation

13. EMF-2209(P)(A), "SPCB Critical Power Correlation", Siemens Power Corporation

14. EMF-2245(P)(A), "Application of Siemens Power Corporation's Critical Power Correlations to Co-Resident Fuel", Siemens Power Corporation

15. EMF-2361(P)(A), "EXEM BWR-2000 ECCS Evaluation Model", Framatome ANP Richland

16. EMF-2292(P)(A), "ATRIUM™ -10: Appendix K Spray Heat Transfer Coefficients", Siemens Power Corporation

17. EMF-CC-074(P)(A) Volume 4, "BWR Stability Analysis-Assessment of STAIF with Input from MICROBURN-B2", Siemens Power Corporation

18. CENPD-300-P-A, "Reference Safety Report for Boiling Water Reactor Reload Fuel," JULY 1996;

19. ^{INSERT} NEDO-32465-A, "BWR Owners' Group Reactor Stability Detect and Suppress Solutions Licensing Basis Methodology and Reload Applications," August 1996.

ABB COMBUSTION
ENGINEERING
NUCLEAR OPERATIONS

(continued)

**REQUEST FOR AMENDMENT TO TECHNICAL SPECIFICATION 4.2.1 AND
5.6.5.b, CORRECTIONS TO ORIGINAL SUBMITTAL**
Attachment 4

Typed Revised Technical Specifications pages

5.6 Reporting Requirements

5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

- b. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents:

Identify the Topical Reports(s) by number and title or identify the staff Safety Evaluation Report for a plant specific methodology by NRC letter and date. The COLR will contain the complete identification for each of the TS referenced topical reports used to prepare the COLR (i.e., report number, title, revision, date and any supplements).

1. XN-NF-81-58(P)(A), "RODEX2 Fuel Rod Thermal-Mechanical Response Evaluation Model," Exxon Nuclear Company
2. XN-NF-85-67(P)(A), "Generic Mechanical Design for Exxon Nuclear Jet Pump BWR Reload Fuel," Exxon Nuclear Company
3. EMF-85-74(P) Supplement 1(P)(A) and Supplement 2(P)(A), "RODEX2A (BWR) Fuel Rod Thermal-Mechanical Evaluation Model," Siemens Power Corporation
4. ANF-89-98(P)(A), "Generic Mechanical Design Criteria for BWR Fuel Designs," Advanced Nuclear Fuels Corporation
5. XN-NF-80-19(P)(A) Volume 1, "Exxon Nuclear Methodology for Boiling Water Reactors - Neutronic Methods for Design and Analysis," Exxon Nuclear Company
6. XN-NF-80-19(P)(A) Volume 4, "Exxon Nuclear Methodology for Boiling Water Reactors: Application of the ENC Methodology to BWR Reloads," Exxon Nuclear Company
7. EMF-2158(P)(A), "Siemens Power Corporation Methodology for Boiling Water Reactors: Evaluation and Validation of CASMO-4/MICROBURN-B2," Siemens Power Corporation
8. XN-NF-80-19(P)(A) Volume 3, "Exxon Nuclear Methodology for Boiling Water Reactors, THERMEX: Thermal Limits Methodology Summary Description," Exxon Nuclear Company

(continued)

5.6 Reporting Requirements

5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

9. XN-NF-84-105(P)(A) Volume 1, "XCOBRA-T: A Computer Code for BWR Transient Thermal-Hydraulic Core Analysis," Exxon Nuclear Company
10. ANF-524(P)(A), "ANF Critical Power Methodology for Boiling Water Reactors," Advanced Nuclear Fuels Corporation
11. ANF-913(P)(A) Volume 1 "COTRANSA2: A Computer Program for Boiling Water Reactor Transient Analysis," Advanced Nuclear Fuels Corporation
12. ANF-1358(P)(A) "The Loss of Feedwater Heating Transient in Boiling Water Reactors," Advanced Nuclear Fuels Corporation
13. EMF-2209(P)(A), "SPCB Critical Power Correlation," Siemens Power Corporation
14. EMF-2245(P)(A), "Application of Siemens Power Corporation's Critical Power Correlations to Co-Resident Fuel," Siemens Power Corporation
15. EMF-2361(P)(A), "EXEM BWR-2000 ECCS Evaluation Model," Framatome ANP Richland
16. EMF-2292(P)(A), "ATRIUM™ -10: Appendix K Spray Heat Transfer Coefficients," Siemens Power Corporation
17. EMF-CC-074(P)(A) Volume 4, "BWR Stability Analysis-Assessment of STAIF with Input from MICROBURN-B2," Siemens Power Corporation
18. CENPD-300-P-A, "Reference Safety Report for Boiling Water Reactor Reload Fuel," ABB Combustion Engineering Nuclear Operations
19. NEDO-32465-A, "BWR Owners' Group Reactor Stability Detect and Suppress Solutions Licensing Basis Methodology and Reload Applications"

(continued)
