

December 13, 2002

MEMORANDUM TO: William Ruland, Director  
Project Directorate IV  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

FROM: Girija Shukla, Project Manager, Section 2 */RA/*  
Project Directorate IV  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

SUBJECT: FORTHCOMING **CLOSED** MEETING WITH WESTINGHOUSE  
ELECTRIC COMPANY

DATE & TIME: Wednesday, December 18, 2002  
1:00 p.m. - 4:00 p.m.

LOCATION: U.S. Nuclear Regulatory Commission  
One White Flint North  
11555 Rockville Pike, Room O-9B2  
Rockville, Maryland

PURPOSE: To discuss implementation of ZrB<sub>2</sub> Integral Fuel Bundle Absorber (IFBA)  
for Combustion Engineering (CE) fuel designs.

PARTICIPANTS: Participants from the NRC include members of the Office of Nuclear  
Reactor Regulation (NRR)

|                       |                          |
|-----------------------|--------------------------|
| <u>NRC</u>            | <u>WESTINGHOUSE</u>      |
| Girija Shukla         | Chuck Molnar             |
| Ralph Caruso          | Zeses Karoutas           |
| Undine Shoop          | Ivan Fiero               |
| Shih-Liang Wu, et al. | Charles Brinkman, et al. |

Project No. 700

Attachment: Agenda (Accession No. ML023470476)

cc w/att: See next page

MEETING CONTACT: Girija Shukla, (301) 415-8439  
Gss@nrc.gov

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**ADAMS Accession No.: ML023470476**

NRC-001

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Westinghouse Electric Company

Project No. 700

cc:

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Mr. H. A. Sepp, Manager  
Regulatory and Licensing Engineering  
Westinghouse Electric Company  
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Pittsburgh, PA 15230-0355

## AGENDA

### IMPLEMENTATION OF ZrB2 (IFBA) FOR CE FUEL DESIGNS

DECEMBER 18, 2002

1. Background on Westinghouse ZrB2 IFBA Experience
  - a. Purpose and Scope of Meeting
  - b. Benefits of ZrB2 IFBA
  - c. Westinghouse/CE Integration Plans
  - d. Westinghouse Experience
2. Impact of ZrB2 IFBA on Irradiation Performance
  - a. Neutronics
  - b. Fuel Performance
3. ZrB2 IFBA Implementation in CE Design and Licensing
  - a. Models and Properties
  - b. Code Modifications
  - c. Benchmarking and Verification
  - d. Approved Topical Reports Affected
  - e. Design and Licensing Impact
    - i. Fuel Performance
    - ii. Fuel Mechanical Design
    - iii. Emergency Core Cooling System Evaluations
    - iv. Non-Loss-of-Coolant Accidents
    - v. Nuclear Design
4. Conclusions
5. Topical Report Summary
6. Schedule
7. NRC Comments/Feedback