

December 24, 2002

Mr. John L. Skolds, President
Exelon Nuclear
Exelon Generation Company, LLC
4300 Winfield Road
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SUBJECT: APPROVAL OF PROPOSED REVISION 70 OF QUALITY ASSURANCE
TOPICAL REPORT EGC-1A, REV. 70, IN ACCORDANCE WITH
10 CFR 50.54(a) REQUIREMENTS FOR EXELON/AMERGEN PLANTS
(TAC NOS. MB4893, MB4894, MB4895, MB4896, MB4897, MB4898, MB4899,
MB4900, MB4901, MB4902, MB4903, MB4904, MB4905, MB4906, MB4907,
MB4908, MB4909, MB4910, MB4911, MB4912 AND MB4913)

Dear Mr. Skolds:

On April 9, 2002, Exelon Generation Company, LLC (Exelon), and AmerGen Energy Company, LLC (AmerGen) submitted proposed Topical Report (QATR) EGC-1A, Revision 70 for Nuclear Regulatory Commission (NRC) review and approval in accordance with 10 CFR 50.54(a), for Braidwood Station, Units 1 and 2; Byron Station, Units 1 and 2; Clinton Power Station, Unit 1; Dresden Nuclear Power Station, Units 1, 2, and 3; LaSalle County Station, Units 1 and 2; Limerick Generating Station, Units 1 and 2; Oyster Creek Generating Station; Peach Bottom Atomic Power Station, Units 1, 2, and 3; Quad Cities Nuclear Power Station, Units 1 and 2; Three Mile Island Nuclear Station, Unit 1; and Zion Nuclear Power Station, Units 1 and 2. The submittal was supplemented on November 27, 2002.

A public meeting was held with representatives of Exelon/Amergen on November 21, 2002, in Rockville, Maryland, to discuss the NRC staff questions and comments associated with QATR EGC-1A, Revision 70. As a result of this, Exelon/AmerGen provided additional information in the November 27, 2002, submittal addressing the staff's concerns at the meeting. The enclosed safety evaluation documents the bases for our conclusion and approval of this report.

Sincerely,

/RA/

Mahesh Chawla, Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-456, STN 50-457, STN 50-454, STN 50-455, 50-461, 50-10, 50-237,
50-249, 72-37, 50-373, 50-374, 50-352, 50-353, 50-219, 72-15, 50-171, 50-277,
50-278, 72-29, 50-254, 50-265, 50-289, 50-295, and 50-304

cc: See next page

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50-278, 72-29, 50-254, 50-265, 50-289, 50-295, and 50-304

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LaSalle, Units 1 and 2

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Zion Nuclear Power Station

- 8 -

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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

PROPOSED CHANGE TO THE QUALITY ASSURANCE PROGRAM

QUALITY ASSURANCE PROGRAM CONSOLIDATION

EXELON GENERATION COMPANY, LLC

AND

AMERGEN ENERGY COMPANY, LLC

BRAIDWOOD STATION, UNITS 1 AND 2 (DOCKET NOS. STN 50-456, STN 50-457)

BYRON STATION, UNITS 1 AND 2 (DOCKET NOS. STN 50-454, STN 50-455)

CLINTON POWER STATION, UNIT 1 (DOCKET NO. 50-461)

DRESDEN NUCLEAR POWER STATION, UNITS 1, 2, AND 3 (DOCKET NOS. 50-10, 50-237,

50-249)

LASALLE COUNTY STATION, UNITS 1 AND 2 (DOCKET NOS. 50-373, 50-374)

LIMERICK GENERATING STATION, UNITS 1 AND 2 (DOCKET NOS. 50-352, 50-353)

OYSTER CREEK GENERATING STATION (DOCKET NO. 50-219)

PEACH BOTTOM ATOMIC POWER STATION, UNITS 1, 2, AND 3 (DOCKET NOS. 50-171,

50-277, 50-278)

QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2 (DOCKET NOS. 50-254,

50-265)

THREE MILE ISLAND, UNIT 1 (DOCKET NO. 50-289)

ZION NUCLEAR POWER STATION, UNITS 1 AND 2 (DOCKET NOS. 50-295, 50-304)

1.0 INTRODUCTION

By letter dated April 9, 2002 (Ref. 1), Exelon Nuclear submitted Revision 70 of Quality Assurance Topical Report (QATR) EGC-1A for the Nuclear Regulatory Commission's (NRC) review in accordance with the provisions of 10 CFR 50.54(a)(4). The submittal was supplemented by letter dated November 27, 2002 (Ref. 2). The topical report describes the Appendix B quality assurance (QA) program for the nuclear fleet of 17 operating reactors owned by Exelon Generation Company, LLC (Exelon) and AmerGen Energy Company, LLC (AmerGen).

The Exelon/AmerGen QA program was previously applicable to the Braidwood, Byron, Dresden, LaSalle, and Quad Cities nuclear power stations. Revision 70 extends the applicability of the QATR to the Clinton, Limerick, Oyster Creek, Peach Bottom, and Three Mile Island, Unit 1 nuclear power stations. The additional plants would adopt the QA program described in QATR, EGC-1A, Revision 70 in lieu of their currently docketed site-specific QA programs (Refs. 3, 4, 5, 6, and 7).

The licensee states that formation of the Exelon and AmerGen companies, which are responsible for operation of multiple sites, makes it prudent to have a common and consistent QA program. The common QATR will allow efficiencies and flexibility in implementing QA requirements, but will maintain sufficient control of activities affecting quality through administrative and programmatic controls.

2.0 BACKGROUND

The QA program described in Revision 69 of the QATR follows the guidance of the 1989 editions of American Society of Mechanical Engineers (ASME) Nuclear Quality Assurance (NQA) standards NQA-1 and NQA-2, "Quality Assurance Program Requirements for Nuclear Facilities." In addition, implementation of the QA program is consistent with the guidance provided by ANSI/ANS 3.2-1988, "Administrative Controls and Quality Assurance for the Operational Phase of Nuclear Power Plants." These standards were adopted by the licensee in 1993 (Ref. 8), when the plant owner (then Commonwealth Edison) held ASME Nuclear Certificates, authorizing the company to conduct certain construction activities in accordance with the ASME Code. The ASME Nuclear Certificate program was terminated in 1997.

In September 2000, Commonwealth Edison submitted Revision 67a of the QATR, which removed information pertaining to the ASME construction QA program. Revision 67a was approved by the staff as the basis for the licensee's operational QA program (Ref. 9). Subsequent revisions through Revision 69 were made in accordance with the provisions of 10 CFR Part 50, Section 54(a)(3) and did not reduce program commitments.

For Revision 70, the licensee proposes to update its commitment to quality standards by adopting the guidance of NQA-1-1994. The 1994 edition differs from the 1989 edition primarily in consolidating NQA-1 and NQA-2 into a single multi-part document and restructuring its format to facilitate use of various parts of the standards. The basic requirements of the 1989 editions, as defined therein, are substantially unchanged.

3.0 EVALUATION

This evaluation is documented in two parts. First, changes incorporated into Revision 70 for the plants covered under Revision 69 are evaluated. Second, the implementation process, for those plants that are adopting the QATR in lieu of site-specific QA programs (Refs. 3-7), is evaluated.

3.1 Plants Covered under QATR, Revision 69

For the sites covered by the QA program described in Revision 69 of the QATR, only one change requires NRC approval, which is the adoption of NQA-1-1994 as the basis for the QA program. With respect to the licensee's program, the transition from the 1989 to the 1994 edition of NQA-1 has minimal impact because the 1994 edition restructures and consolidates the guidance of the 1989 edition rather than revising requirements. The QATR continues to conform to the regulatory positions of Regulatory Guide (RG) 1.28, Revision 3 and RG 1.33, Revision 2. Based on the above, the licensee's adoption of NQA-1-1994 is found to be acceptable.

With respect to changes to Revision 69 that are allowed under 10 CFR 50.54(a)(3), the licensee has invoked the provisions of § 50.54(a)(3)(ii) to adopt a previously approved alternative for independent review (Ref. 10). In addition, text related to the previously held ASME Nuclear Certificates has been removed while the level of detail in describing the operational program is retained.

For the sites covered under Revision 69, the QA program described in Revision 70 of the QATR continues to satisfy the acceptance criteria set forth in the NUREG-0800 Standard Review Plan (SRP) 17.2, "Quality Assurance During the Operations Phase," and, therefore, continues to meet the requirements of 10 CFR 50.36(c)(5) and Appendix B to 10 CFR Part 50.

3.2 Plants Previously Covered by Site-Specific QA Programs

Revision 70 extends the applicability of the QATR to the Clinton, Limerick, Oyster Creek, Peach Bottom, and Three Mile Island (Unit 1) nuclear power stations. In consolidating the QA programs for these plants, the licensee followed the guidance of SRP 17.1 and SRP 17.2 in performing reviews that compared positions and requirements contained in the docketed QA programs to those described in the QATR. The approach taken by the licensee in reconciling site-specific differences was to (1) add detail to the QATR; (2) make changes allowed under § 50.54(a)(3), (3) incorporate site-specific clarifications and exceptions into the QATR, and (4) identify changes as reductions in commitment under § 50.54(a)(4) to be submitted and approved by the staff. The licensee states that its intent is to remove the clarifications and exceptions as commonality among the site quality programs is achieved.

3.2.1 Acceptability of NQA-1-1994 as the Basis for the Common QATR

RG 1.28, Revision 3 endorses NQA-1-1983 as a method acceptable to the NRC staff for complying with the provisions of Appendix B with regard to establishing and implementing the requisite QA program for the design and construction of nuclear power plants.

The provisions of § 50.54(a)(3)(ii) allow the use of QA alternatives or exceptions previously approved by an NRC safety evaluation. Prior to 1997, NRC approval letters did not generally provide a rationale for approving a change to a QA program. The applicable NRC approval letter, dated June 7, 1993 (Ref. 8), approved the use of NQA-1-1989 as the basis for the licensee's operational QA program (QATR CE-1-A, Revision 63), but provided no rationale for approving the change. Docketed information, applicable to the submittal, was insufficient to reconstitute the basis for approval of NQA-1-1989.

Accordingly, the licensee has included, in Attachment 5 to the submittal (Ref. 1), a side-by-side analysis of requirement equivalency between NQA-1-1983 and NQA-1-1994. Based on the comparison, the licensee concludes that the requirements of NQA-1-1983 are equivalent to those contained within NQA-1-1994. The licensee further concludes that NQA-1-1989, which serves as the basis for QATR, Revision 69, is contained within NQA-1-1994. The staff examined the side-by-side comparison of NQA-1-1983 and NQA-1-1994 equivalence, and concurs with the licensee's conclusions in finding NQA-1-1994, as implemented through the common QATR, to provide an acceptable basis for the licensee's operational QA program.

3.2.2 Acceptability of the Process for Consolidating Site-Specific QA Programs

The licensee's implementation process for consolidating the site-specific QA programs employs the four principal methods described above. The staff finds that the detail added to the QATR does not reduce commitments of the QA program previously approved by the staff. The staff reviewed the licensee's use of previously approved alternatives and exceptions and finds them to be acceptable in that they were applied in a manner applicable to the licensee's facilities. The staff also reviewed the site-specific clarifications and exceptions that have been incorporated into Appendix C of the QATR and finds them to be acceptable in that they maintain existing, NRC-approved commitments of the site-specific programs.

With respect to the quality standards listed in Appendix C of the QATR, Revision 70, the quality program described in Revision 69 followed the guidance of ANS 3.1 (1981), "Selection, Qualification, and Training of Personnel for Nuclear Power Plants," and ANS/ANS 3.2 (1988), "Administrative Controls and Quality Assurance for the Operational Phase of Nuclear Power Plants." The sites covered by Revision 69 continue to follow the guidance of these standards. In adopting the common QATR, sites which have adopted the QATR in lieu of site-specific programs continue to follow the editions of these standards, previously approved by the NRC.

With respect to ANS 3.1, Clinton, Limerick, Oyster Creek, and Three Mile Island follow the guidance of ANS 3.1 (1978); Peach Bottom follows the guidance of ANSI N18.1 (1971). Information specific to implementation of these standards has been retained in the Updated Final Safety Analysis Reports for these sites.

With respect to ANS/ANS 3.2 (1988), Clinton, Limerick, Oyster Creek, and Three Mile Island follow the guidance of ANSI/ANS N18.7 (1976); Peach Bottom follows the guidance of ANSI N18.7 (1972). Information specific to implementation of these standards is included in Appendix C of the QATR.

In adopting the common QA program, the licensee submitted changes to site-specific QA programs for three sites (Oyster Creek, Peach Bottom, and Three Mile Island) as reductions in commitments. The bases for these reductions in commitments, provided as Attachment 2

(Oyster Creek), Attachment 3 (Peach Bottom), and Attachment 4 (Three Mile Island, Unit 1) of the submittal, were reviewed by the staff and found to be acceptable in that they do not reduce the effectiveness of the licensee's QA program and continue to meet the requirements of 10 CFR Part 50, Appendix B.

4.0 CONCLUSION

Based on review of Revision 70 of the QATR, the staff concludes that the licensee's common QA program satisfies the requirements of Appendix B to 10 CFR Part 50 in accordance with the review criteria contained in Standard Review Plan 17.2 and, therefore, is acceptable.

5.0 REFERENCES

1. Letter, Exelon Nuclear to the U.S. NRC, "Request for Approval of Quality Assurance Program Changes for Exelon Generation Company, LLC, and AmerGen Energy Company, LLC, Nuclear Power Plants," April 9, 2002 (Accession No. ML021070541).
2. Letter, Exelon Nuclear to the U.S. NRC, "Information to Provide Clarification of April 9, 2002 Exelon/AmerGen Submittal for NRC Approval of Conversion to Common Quality Assurance Topical Report for Exelon Generation Company, LLC, and AmerGen Energy Company, LLC, Nuclear Power Plants," November 27, 2002.
3. Clinton Quality Assurance Program, as described in the Clinton Power Station Quality Assurance Manual, Revision 29a, October 31, 2001 (Accession No. ML020230122).
4. Limerick Quality Assurance Program, Revision 11, as described in the Limerick Generating Station Updated Final Safety Analysis Report, Section 17.2, Revision 11, September 19, 2002 (Accession No. ML02284040443).
5. Oyster Creek Quality Assurance Program, as described in the Oyster Creek Operational Quality Assurance Plan 2000-PLN-7200.01, Revision 3, October 1, 2001 (Accession No. ML013510434).
6. Peach Bottom Quality Assurance Program, as described in Peach Bottom Atomic Power Station Updated Final Safety Analysis Report, Appendix D.11, Revision 18, April 2001 (Accession No. ML011010109).
7. Three Mile Island, Unit 1, Quality Assurance Program, as described in the Three Mile Island Operational Quality Assurance Program 1000-PLN-7200.01, Revision 25, May 13, 2002 (Accession No. ML0227700280).
8. Quality Assurance Topical Report CE-1-A, Revision 63, approved by NRC letter dated June 7, 1993.

9. Quality Assurance Program for Nuclear Generating Stations, Topical Report CE-1-A, Revision 67a, approved by NRC letter dated March 13, 2001 (Accession No. ML0107200581).
10. Safety Evaluation, Nuclear Quality Assurance Plan TVA-NQA-PLN89-A, Elimination of Independent Engineering Group, Tennessee Valley Authority - Browns Ferry, Watts Bar, and Sequoyah Nuclear Plants, August 26, 1999.

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Date: December 24, 2002