

December 9, 2002

Mr. Bryce L. Shriver
Senior Vice President
and Chief Nuclear Officer
PPL Susquehanna, LLC
769 Salem Boulevard
Berwick, PA 18603-0467

SUBJECT: SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 2 - REQUEST FOR
ADDITIONAL INFORMATION (RAI) - MINIMUM CRITICAL POWER RATIO
SAFETY LIMITS AND REFERENCE CHANGES (TAC NO. MB5610)

Dear Mr. Shriver:

In reviewing your application of July 17, 2002, the U.S. Nuclear Regulatory Commission staff has determined that additional information contained in the enclosure to this letter is needed to complete its review. We discussed this issue with your staff during a conference call on November 20, 2002. As we indicated during our conversation, we are enclosing a formal RAI. The enclosed questions have been revised from the draft RAI to exclude the questions that were answered during the conference call. As agreed to by your staff, we request you respond within 30 days of the date of this letter.

If you have any questions, please contact me at 301-415-1402.

Sincerely,

/RA/

Timothy G. Colburn, Sr. Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-388

Enclosure: RAI

cc w/encl: See next page

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DATE	12/5/02	12/6/02	12/5/02	12/9/02

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REQUEST FOR ADDITIONAL INFORMATION
RELATING TO MINIMUM CRITICAL POWER RATIO (MCPR)
SAFETY LIMITS AND REFERENCE CHANGES
SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 2
DOCKET NO. 50-388

The Nuclear Regulatory Commission (NRC) staff is reviewing the proposed license amendment request to Facility Operating License No. NPF-22, relating to MCPR Safety Limits and reference changes dated July 17, 2002, and has found further clarification is needed as follows:

1. Identify any differences between the reference core loading pattern for the analysis and the real core loading pattern. Provide the procedure of how to deal with this difference if it does occur.
2. On page 1, attachment 1 of your submittal, you state that NRC approval of the previously used computer code ANFB-10 critical power correlation required a factor of 2 to be applied to the number of pins calculated to be in boiling transition for the Safety Limit calculation. Provide a detailed basis of this statement and identify the impact on the Safety Limit calculation.

Enclosure

Susquehanna Steam Electric Station,
Units 1 &2

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