

DCP/NRC1523

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Attachment 3

Draft Modifications to AP1000 Technical Specifications

TABLE OF CONTENTS

<u>Section</u>	<u>Title</u>	<u>Page</u>
CHAPTER 16	TECHNICAL SPECIFICATIONS	
16.1	TECHNICAL SPECIFICATIONS	16.1-1
16.1.1	Introduction to Technical Specifications	16.1-1
1.0	Use and Application	1.1-1
1.1	Definitions.....	1.1-1
1.2	Logical Connectors	1.2-1
1.3	Completion Times.....	1.3-1
1.4	Frequency.....	1.4-1
2.0	Safety Limits (SLs).....	2.0-1
2.1	SLs	2.0-1
2.2	SL Violations	2.0-1
3.0	Limiting Conditions for Operation (LCO) Applicability	3.0-1
3.0	Surveillance Requirement (SR) Applicability	3.0-4
3.1	Reactivity Control Systems.....	3.1-1
3.1.1	Shutdown Margin (SDM)	3.1-1
3.1.2	Core Reactivity.....	3.1-2
3.1.3	Moderator Temperature Coefficient (MTC).....	3.1-4
3.1.4	Rod Group Alignment Limits	3.1-6
3.1.5	Shutdown Bank Insertion Limits.....	3.1-11
3.1.6	Control Bank Insertion Limits.....	3.1-13
3.1.7	Rod Position Indication.....	3.1-16
3.1.8	Physics Tests Exceptions – MODE 2.....	3.1-18
3.1.9	Chemical and Volume Control System (CVS) Demineralized Water Isolation Valves.....	3.1-20
3.2	Power Distribution Limits	3.2-1
3.2.1	Heat Flux Hot Channel Factor ($F_Q(Z)$) (F_Q Methodology)	3.2-1
3.2.2	Nuclear Enthalpy Rise Hot Channel Factor ($F_{\Delta H}^N$)	3.2-5
3.2.3	Axial Flux Difference (AFD) (Relaxed Axial Offset Control (RAOC) Methodology)	3.2-8
3.2.4	Quadrant Power Tilt Ratio (QPTR)	3.2-10
3.2.5	OPDMS-Monitored Power Distribution Parameters	3.2-14
3.3	Instrumentation.....	3.3-1
3.3.1	Reactor Trip System (RTS) Instrumentation	3.3-1
3.3.2	Engineered Safety Feature Actuation System (ESFAS) Instrumentation	3.3-19
3.3.3	Post Accident Monitoring (PAM) Instrumentation	3.3-47
3.3.4	Remote Shutdown Workstation.....	3.3-50

TABLE OF CONTENTS (Cont.)

<u>Section</u>	<u>Title</u>	<u>Page</u>
3.4	Reactor Coolant System (RCS)	3.4-1
3.4.1	RCS Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits.....	3.4-1
3.4.2	RCS Minimum Temperature for Criticality	3.4-3
3.4.3	RCS Pressure and Temperature (P/T) Limits.....	3.4-4
3.4.4	RCS Loops.....	3.4-6
3.4.5	Not Used	3.4-8
3.4.6	Pressurizer.....	3.4-9
3.4.7	Pressurizer Safety Valves.....	3.4-10
3.4.8	RCS Operational Leakage.....	3.4-12
3.4.9	Minimum RCS Flow.....	3.4-14
3.4.10	RCS Leakage Detection Instrumentation	3.4-15
3.4.11	RCS Specific Activity	3.4-18
3.4.12	Automatic Depressurization System (ADS) – Operating	3.4-20
3.4.13	Automatic Depressurization System (ADS) – Shutdown, RCS Intact.....	3.4-22
3.4.14	Automatic Depressurization System (ADS) – Shutdown, RCS Open	3.4-24
3.4.15	Low Temperature Overpressure Protection (LTOP) System.....	3.4-27
3.4.16	RCS Pressure Isolation Valve (PIV) Integrity	3.4-30
3.4.17	Reactor Vessel Head Vent (RVHV).....	3.4-32
3.4.18	Chemical and Volume Control System (CVS) Makeup Isolation Valves.....	3.4-33
3.5	Passive Core Cooling System (PXS)	3.5-1
3.5.1	Accumulators	3.5-1
3.5.2	Core Makeup Tanks (CMTs) – Operating	3.5-4
3.5.3	Core Makeup Tanks (CMTs) – Shutdown, RCS Intact.....	3.5-7
3.5.4	Passive Residual Heat Removal Heat Exchanger (PRHR HX) – Operating.....	3.5-9
3.5.5	Passive Residual Heat Removal Heat Exchanger (PRHR HX) – Shutdown, RCS Intact.....	3.5-12
3.5.6	In-containment Refueling Water Storage Tank (IRWST) – Operating	3.5-14
3.5.7	In-containment Refueling Water Storage Tank (IRWST) – Shutdown, MODE 5	3.5-17
3.5.8	In-containment Refueling Water Storage Tank (IRWST) – Shutdown, MODE 6	3.5-19
3.6	Containment Systems	3.6-1
3.6.1	Containment.....	3.6-1
3.6.2	Containment Air Locks	3.6-2
3.6.3	Containment Isolation Valves	3.6-6

TABLE OF CONTENTS (Cont.)

<u>Section</u>	<u>Title</u>	<u>Page</u>
3.6.4	Containment Pressure.....	3.6-11
3.6.5	Containment Air Temperature	3.6-12
3.6.6	Passive Containment Cooling System (PCS) – Operating	3.6-13
3.6.7	Passive Containment Cooling System (PCS) – Shutdown	3.6-16
3.6.8	Containment Penetrations	3.6-18
3.6.9	pH Adjustment.....	3.6-21
3.7	Plant Systems.....	3.7-1
3.7.1	Main Steam Safety Valves (MSSVs)	3.7-1
3.7.2	Main Steam Isolation Valves (MSIVs)	3.7-5
3.7.3	Main Feedwater Isolation and Control Valves (MFIV and MFCV).....	3.7-9
3.7.4	Secondary Specific Activity.....	3.7-11
3.7.5	Spent Fuel Pool Water Level	3.7-12
3.7.6	Main Control Room Habitability System (VES).....	3.7-13
3.7.7	Startup Feedwater Isolation and Control Valves	3.7-17
3.7.8	Main Steam Line Leakage.....	3.7-19
3.7.9	Fuel Storage Pool Makeup Water Sources	3.7-20
3.7.10	Steam Generator Isolation Valves	3.7-22
3.8	Electrical Power Systems.....	3.8-1
3.8.1	DC Sources – Operating	3.8-1
3.8.2	DC Sources – Shutdown	3.8-5
3.8.3	Inverters – Operating.....	3.8-7
3.8.4	Inverters – Shutdown	3.8-9
3.8.5	Distribution Systems – Operating	3.8-11
3.8.6	Distribution Systems – Shutdown	3.8-14
3.8.7	Battery Parameters	3.8-16
3.9	Refueling Operations.....	3.9-1
3.9.1	Boron Concentration	3.9-1
3.9.2	Unborated Water Source Flow Paths	3.9-2
3.9.3	Nuclear Instrumentation.....	3.9-4
3.9.4	Refueling Cavity Water Level.....	3.9-6
3.9.5	Containment Penetrations	3.9-7
3.9.6	Containment Air Filtration System (VFS).....	3.9-9
4.0	Design Features	4.0-1
4.1	Site	4.0-1
4.1.1	Site and Exclusion Boundaries	4.0-1
4.1.2	Low Population Zone (LPZ).....	4.0-1

TABLE OF CONTENTS (Cont.)

<u>Section</u>	<u>Title</u>	<u>Page</u>
4.2	Reactor Core	4.0-1
4.2.1	Fuel Assemblies.....	4.0-1
4.2.2	Control Rod and Gray Rod Assemblies.....	4.0-1
4.3	Fuel Storage	4.0-2
4.3.1	Criticality.....	4.0-2
4.3.2	Drainage	4.0-2
4.3.3	Capacity.....	4.0-2
5.0	Administrative Controls.....	5.0-1
5.1	Responsibility.....	5.0-1
5.2	Organization.....	5.0-2
5.3	Unit Staff Qualifications	5.0-5
5.4	Procedures.....	5.0-6
5.5	Programs and Manuals.....	5.0-7
5.6	Reporting Requirements.....	5.0-16
5.7	High Radiation Area	5.0-22
B 2.0	Safety Limits (SLs).....	B 2.0-1
B 2.1.1	Reactor Core Safety Limits (SLs).....	B 2.0-1
B 2.1.2	Reactor Coolant System (RCS) Pressure SL	B 2.0-7
B 3.0	Limiting Conditions for Operation (LCO) Applicability.....	B 3.0-1
B 3.0	Surveillance Requirement (SR) Applicability	B 3.0-12
B 3.1	Reactivity Control Systems	B 3.1-1
B 3.1.1	Shutdown Margin (SDM).....	B 3.1-1
B 3.1.2	Core Reactivity.....	B 3.1-7
B 3.1.3	Moderator Temperature Coefficient (MTC).....	B 3.1-13
B 3.1.4	Rod Group Alignment Limits.....	B 3.1-19
B 3.1.5	Shutdown Bank Insertion Limits	B 3.1-30
B 3.1.6	Control Bank Insertion Limits	B 3.1-35
B 3.1.7	Rod Position Indication	B 3.1-41
B 3.1.8	Physics Tests Exceptions – MODE 2	B 3.1-47
B 3.1.9	Chemical and Volume Control System (CVS) Demineralized Water Isolation Valves	B 3.1-54
B 3.2	Power Distribution Limits.....	B 3.2-1
B 3.2.1	Heat Flux Hot Channel Factor ($F_Q(Z)$) (F_Q Methodology).....	B 3.2-1
B 3.2.2	Nuclear Enthalpy Rise Hot Channel Factor ($F_{\Delta H}^N$)	B 3.2-10
B 3.2.3	Axial Flux Difference (AFD) (Relaxed Axial Offset Control (RAOC) Methodology).....	B 3.2-17
B 3.2.4	Quadrant Power Tilt Ratio (QPTR).....	B 3.2-22
B 3.2.5	OPDMS-Monitored Power Distribution Parameters	B 3.2-28

TABLE OF CONTENTS (Cont.)

<u>Section</u>	<u>Title</u>	<u>Page</u>
B 3.3	Instrumentation	B 3.3-1
B 3.3.1	Reactor Trip System (RTS) Instrumentation	B 3.3-1
B 3.3.2	Engineered Safety Feature Actuation System (ESFAS) Instrumentation	B 3.3-57
B 3.3.3	Post Accident Monitoring (PAM) Instrumentation.....	B 3.3-134
B 3.3.4	Remote Shutdown Workstation (RSW).....	B 3.3-144
B 3.4	Reactor Coolant System (RCS)	B 3.4-1
B 3.4.1	RCS Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits ...	B 3.4-1
B 3.4.2	RCS Minimum Temperature for Criticality	B 3.4-6
B 3.4.3	RCS Pressure and Temperature (P/T) Limits	B 3.4-9
B 3.4.4	RCS Loops	B 3.4-17
B 3.4.5	Not Used.....	B 3.4-23
B 3.4.6	Pressurizer	B 3.4-24
B 3.4.7	Pressurizer Safety Valves	B 3.4-27
B 3.4.8	RCS Operational Leakage	B 3.4-31
B 3.4.9	Minimum RCS Flow	B 3.4-37
B 3.4.10	RCS Leakage Detection Instrumentation.....	B 3.4-40
B 3.4.11	RCS Specific Activity	B 3.4-45
B 3.4.12	Automatic Depressurization System (ADS) – Operating.....	B 3.4-49
B 3.4.13	Automatic Depressurization System (ADS) – Shutdown, RCS Intact	B 3.4-53
B 3.4.14	Automatic Depressurization System (ADS) – Shutdown, RCS Open	B 3.4-55
B 3.4.15	Low Temperature Overpressure Protection (LTOP) System.....	B 3.4-58
B 3.4.16	RCS Pressure Isolation Valve (PIV) Integrity	B 3.4-66
B 3.4.17	Reactor Vessel Head Vent (RVHV)	B 3.4-72
B 3.4.18	Chemical and Volume Control System (CVS) Makeup Isolation Valves	B 3.4-75
B 3.5	Passive Core Cooling System (PXS).....	B 3.5-1
B 3.5.1	Accumulators.....	B 3.5-1
B 3.5.2	Core Makeup Tanks (CMTs) – Operating.....	B 3.5-8
B 3.5.3	Core Makeup Tanks (CMTs) – Shutdown, RCS Intact	B 3.5-15
B 3.5.4	Passive Residual Heat Removal Heat Exchanger (PRHR HX) – Operating	B 3.5-18
B 3.5.5	Passive Residual Heat Removal Heat Exchanger (PRHR HX) – Shutdown, RCS Intact.....	B 3.5-25

TABLE OF CONTENTS (Cont.)

<u>Section</u>	<u>Title</u>	<u>Page</u>
B 3.5.6	In-containment Refueling Water Storage Tank (IRWST) – Operating.....	B 3.5-28
B 3.5.7	In-containment Refueling Water Storage Tank (IRWST) – Shutdown, MODE 5.....	B 3.5-35
B 3.5.8	In-containment Refueling Water Storage Tank (IRWST) – Shutdown, MODE 6.....	B 3.5-38
B 3.6	Containment Systems.....	B 3.6-1
B 3.6.1	Containment	B 3.6-1
B 3.6.2	Containment Air Locks	B 3.6-6
B 3.6.3	Containment Isolation Valves.....	B 3.6-14
B 3.6.4	Containment Pressure.....	B 3.6-24
B 3.6.5	Containment Air Temperature.....	B 3.6-27
B 3.6.6	Passive Containment Cooling System (PCS) – Operating	B 3.6-31
B 3.6.7	Passive Containment Cooling System (PCS) – Shutdown.....	B 3.6-39
B 3.6.8	Containment Penetrations.....	B 3.6-42
B 3.6.9	pH Adjustment	B 3.6-50
B 3.7	Plant Systems.....	B 3.7-1
B 3.7.1	Main Steam Safety Valves (MSSVs).....	B 3.7-1
B 3.7.2	Main Steam Isolation Valves (MSIVs).....	B 3.7-7
B 3.7.3	Main Feedwater Isolation and Control Valves (MFIVs and MFCVs).....	B 3.7-15
B 3.7.4	Secondary Specific Activity	B 3.7-20
B 3.7.5	Spent Fuel Pool Water Level.....	B 3.7-23
B 3.7.6	Main Control Room Emergency Habitability System	B 3.7-26
B 3.7.7	Startup Feedwater Isolation and Control Valves.....	B 3.7-33
B 3.7.8	Main Steam Line Leakage.....	B 3.7-37
B 3.7.9	Fuel Storage Pool Makeup Water Sources	B 3.7-40
B 3.7.10	Steam Generator Isolation Valves	B 3.7-45
B 3.8	Electrical Power Systems	B 3.8-1
B 3.8.1	DC Sources – Operating.....	B 3.8-1
B 3.8.2	DC Sources – Shutdown.....	B 3.8-12
B 3.8.3	Inverters – Operating.....	B 3.8-16
B 3.8.4	Inverters – Shutdown.....	B 3.8-21
B 3.8.5	Distribution Systems – Operating.....	B 3.8-25
B 3.8.6	Distribution System – Shutdown.....	B 3.8-38
B 3.8.7	Battery Parameters.....	B 3.8-42

TABLE OF CONTENTS (Cont.)

<u>Section</u>	<u>Title</u>	<u>Page</u>
	B 3.9 Refueling Operations	B 3.9-1
	B 3.9.1 Boron Concentration	B 3.9-1
	B 3.9.2 Unborated Water Source Flow Paths.....	B 3.9-5
	B 3.9.3 Nuclear Instrumentation	B 3.9-8
	B 3.9.4 Refueling Cavity Water Level	B 3.9-11
	B 3.9.5 Containment Penetrations.....	B 3.9-14
	B 3.9.6 Containment Air Filtration System (VFS).....	B 3.9-20
16.2	DESIGN RELIABILITY ASSURANCE PROGRAM	16.2-1
16.3	INVESTMENT PROTECTION.....	16.3-1
16.3.1	Investment Protection Short-Term Availability Controls.....	16.3-1
16.3.2	Combined License Information	16.3-2

LIST OF TABLES

<u>Table No.</u>	<u>Title</u>	<u>Page</u>
16.3-1	List of Investment Protection Short-Term Availability Controls	16.3-4
16.3-2	Investment Protection Short-Term Availability Controls.....	16.3-5

16.1 Technical Specifications**16.1.1 Introduction to Technical Specifications****LCO Selection Criteria**

The NRC Final Policy Statement on Technical Specification Improvement (July 1993) criteria stated below has been used to identify the structures, systems, and parameters for which Limiting Conditions for Operation (LCOs) have been included in the AP1000 Technical Specifications.

1. Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary.
2. A process variable, design feature, or operating restriction that is an initial condition of a Design Basis Accident or Transient Analyses that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.
3. A structure, system or component that is part of the primary success path and which functions or actuates to mitigate a Design Basis Accident or Transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.
4. Structures, systems, and components which operating experience or probabilistic safety assessment has shown to be important to public health and safety.

Technical Specification Content

The content of the AP1000 Technical Specifications meets the 10CFR50.36 requirements and is consistent with the Technical Specification Improvement Program, NUREG 1431, Rev. 2, to the maximum extent possible. The content differs from NUREG 1431 only as necessary to reflect technical differences between the "typical" Westinghouse design and the AP1000 design.

Completion Times and Surveillance Frequencies

The Completion Times and Surveillance Frequencies specified in NUREG 1431 have been applied to similar Actions and Surveillances Requirements in AP1000. Refer to Westinghouse letter DCP/NRC0891 for a discussion regarding selection of Completion Times and Surveillance Frequencies for those AP1000 Tech Specs for which no comparable NUREG 1431 system/function exists and for those AP1000 system design differences which lead to deviations from NUREG 1431 Completion Times and Surveillance Frequencies.

Shutdown Completion Times/Mode Definitions

The AP1000 plant design is different from current Westinghouse designs in that the systems normally used for MODE reduction are non-safety systems; and therefore, are not covered by LCO requirements in Technical Specifications. The passive safety systems, which shut down the

plant require a longer period of time to accomplish mode changes and can not reduce the RCS temperature to below 200°F.

LCO and Bases "TBD" Information

In cases where the detailed design, equipment selection, or other efforts are not sufficiently complete to establish the information required to be specified in Technical Specifications, "[TBD]" (to be determined) has been specified. Additionally, some of the information, such as that established by startup testing, will not be available until a plant is constructed.

Combined License Information

This set of technical specifications is intended to be used as a guide in the development of the plant-specific technical specifications. Combined License applicants referencing the AP1000 will replace preliminary information provided in brackets [] with final plant specific values.

TABLE OF CONTENTS

1.0	USE AND APPLICATION	1.1-1
1.1	Definitions	1.1-1
1.2	Logical Connectors	1.2-1
1.3	Completion Times	1.3-1
1.4	Frequency	1.4-1
2.0	SAFETY LIMITS (SLs)	2.0-1
2.1	SLs	2.0-1
2.2	SL Violations	2.0-1
3.0	LIMITING CONDITIONS FOR OPERATION (LCO) APPLICABILITY . .	3.0-1
3.0	SURVEILLANCE REQUIREMENT (SR) APPLICABILITY	3.0-4
3.1	REACTIVITY CONTROL SYSTEMS	3.1-1
3.1.1	SHUTDOWN MARGIN (SDM)	3.1-1
3.1.2	Core Reactivity	3.1-2
3.1.3	Moderator Temperature Coefficient (MTC)	3.1-4
3.1.4	Rod Group Alignment Limits	3.1-6
3.1.5	Shutdown Bank Insertion Limits	3.1-11
3.1.6	Control Bank Insertion Limits	3.1-13
3.1.7	Rod Position Indication	3.1-16
3.1.8	PHYSICS TESTS Exceptions – MODE 2	3.1-18
3.1.9	Chemical and Volume Control System (CVS) Demineralized Water Isolation Valves .	3.1-20
3.2	POWER DISTRIBUTION LIMITS	3.2-1
3.2.1	Heat Flux Hot Channel Factor ($F_q(Z)$) (F_q Methodology)	3.2-1
3.2.2	Nuclear Enthalpy Rise Hot Channel Factor ($F_{\Delta H}$)	3.2-5
3.2.3	AXIAL FLUX DIFFERENCE (AFD) (Relaxed Axial Offset Control (RAOC) Methodology) . .	3.2-8
3.2.4	QUADRANT POWER TILT RATIO (QPTR)	3.2-10
3.2.5	OPDMS-Monitored Power Distribution Parameters	3.2-14
3.3	INSTRUMENTATION	3.3-1
3.3.1	Reactor Trip System (RTS) Instrumentation . . .	3.3-1
3.3.2	Engineered Safety Feature Actuation System (ESFAS) Instrumentation	3.3-19
3.3.3	Post Accident Monitoring (PAM) Instrumentation	3.3-47
3.3.4	Remote Shutdown Workstation	3.3-50

(continued)

TABLE OF CONTENTS (continued)

3.4	REACTOR COOLANT SYSTEM (RCS)	3.4-1
3.4.1	RCS Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits	3.4-1
3.4.2	RCS Minimum Temperature for Criticality	3.4-3
3.4.3	RCS Pressure and Temperature (P/T) Limits	3.4-4
3.4.4	RCS Loops	3.4-6
3.4.5	Not Used	3.4-8
3.4.6	Pressurizer	3.4-9
3.4.7	Pressurizer Safety Valves	3.4-10
3.4.8	RCS Operational LEAKAGE	3.4-12
3.4.9	Minimum RCS Flow	3.4-14
3.4.10	RCS Leakage Detection Instrumentation	3.4-15
3.4.11	RCS Specific Activity	3.4-18
3.4.12	Automatic Depressurization System (ADS) – Operating	3.4-20
3.4.13	Automatic Depressurization System (ADS) – Shutdown, RCS Intact	3.4-22
3.4.14	Automatic Depressurization System (ADS) – Shutdown, RCS Open	3.4-24
3.4.15	Low Temperature Overpressure Protection (LTOP) System	3.4-27
3.4.16	RCS Pressure Isolation Valve (PIV) Integrity	3.4-30
3.4.17	Reactor Vessel Head Vent (RVHV)	3.4-32
3.4.18	Chemical and Volume Control System (CVS) Makeup Isolation Valves	3.4-33
3.5	PASSIVE CORE COOLING SYSTEM (PXS)	3.5-1
3.5.1	Accumulators	3.5-1
3.5.2	Core Makeup Tanks (CMTs) – Operating	3.5-4
3.5.3	Core Makeup Tanks (CMTs) – Shutdown, RCS Intact	3.5-7
3.5.4	Passive Residual Heat Removal Heat Exchanger (PRHR HX) – Operating	3.5-9
3.5.5	Passive Residual Heat Removal Heat Exchanger (PRHR HX) – Shutdown, RCS Intact	3.5-12
3.5.6	In-containment Refueling Water Storage Tank (IRWST) – Operating	3.5-14
3.5.7	In-containment Refueling Water Storage Tank (IRWST) – Shutdown, MODE 5	3.5-17
3.5.8	In-containment Refueling Water Storage Tank (IRWST) – Shutdown, MODE 6	3.5-19
3.6	CONTAINMENT SYSTEMS	3.6-1
3.6.1	Containment	3.6-1
3.6.2	Containment Air Locks	3.6-2
3.6.3	Containment Isolation Valves	3.6-6
3.6.4	Containment Pressure	3.6-11
3.6.5	Containment Air Temperature	3.6-12

(continued)

TABLE OF CONTENTS

3.6	CONTAINMENT SYSTEMS (continued)	
3.6.6	Passive Containment Cooling System (PCS) –	
	Operating	3.6-13
3.6.7	Passive Containment Cooling System (PCS) –	
	Shutdown	3.6-16
3.6.8	Containment Penetrations	3.6-18
3.6.9	pH Adjustment	3.6-21
3.7	PLANT SYSTEMS	3.7-1
3.7.1	Main Steam Safety Valves (MSSVs)	3.7-1
3.7.2	Main Steam Isolation Valves (MSIVs)	3.7-5
3.7.3	Main Feedwater Isolation and Control Valves	
	(MFIV and MFCV)	3.7-9
3.7.4	Secondary Specific Activity	3.7-11
3.7.5	Spent Fuel Pool Water Level	3.7-12
3.7.6	Main Control Room Habitability System (VES) . .	3.7-13
3.7.7	Startup Feedwater Isolation and Control	
	Valves	3.7-17
3.7.8	Main Steam Line Leakage	3.7-19
3.7.9	Fuel Storage Pool Makeup Water Sources	3.7-20
3.7.10	Steam Generator Isolation Valves	3.7-22
3.8	ELECTRICAL POWER SYSTEMS	3.8-1
3.8.1	DC Sources – Operating	3.8-1
3.8.2	DC Sources – Shutdown	3.8-5
3.8.3	Inverters – Operating	3.8-7
3.8.4	Inverters – Shutdown	3.8-9
3.8.5	Distribution Systems – Operating	3.8-11
3.8.6	Distribution Systems – Shutdown	3.8-14
3.8.7	Battery Parameters	3.8-16
3.9	REFUELING OPERATIONS	3.9-1
3.9.1	Boron Concentration	3.9-1
3.9.2	Unborated Water Source Flow Paths	3.9-2
3.9.3	Nuclear Instrumentation	3.9-4
3.9.4	Refueling Cavity Water Level	3.9-6
3.9.5	Containment Penetrations	3.9-7
3.9.6	Containment Air Filtration System (VFS)	3.9-9
4.0	DESIGN FEATURES	4.0-1
4.1	Site	4.0-1
4.1.1	Site and Exclusion Boundaries	4.0-1
4.1.2	Low Population Zone (LPZ)	4.0-1
4.2	Reactor Core	4.0-1
4.2.1	Fuel Assemblies	4.0-1
4.2.2	Control Rod and Gray Rod Assemblies	4.0-1

(continued)

TABLE OF CONTENTS (continued)

4.3	Fuel Storage	4.0-2
4.3.1	Criticality	4.0-2
4.3.2	Drainage	4.0-2
4.3.3	Capacity	4.0-2
5.0	ADMINISTRATIVE CONTROLS	5.0-1
5.1	Responsibility	5.0-1
5.2	Organization	5.0-2
5.3	Unit Staff Qualifications	5.0-5
5.4	Procedures	5.0-6
5.5	Programs and Manuals	5.0-7
5.6	Reporting Requirements	5.0-16
5.7	High Radiation Area	5.0-22

(continued)

TABLE OF CONTENTS (continued)

B 2.0	SAFETY LIMITS (SLs)	B 2.0-1
B 2.1.1	Reactor Core Safety Limits (SLs)	B 2.0-1
B 2.1.2	Reactor Coolant System (RCS) Pressure SL	B 2.0-7
B 3.0	LIMITING CONDITIONS FOR OPERATION (LCO) APPLICABILITY . .	B 3.0-1
B 3.0	SURVEILLANCE REQUIREMENT (SR) APPLICABILITY	B 3.0-12
B 3.1	REACTIVITY CONTROL SYSTEMS	B 3.1-1
B 3.1.1	SHUTDOWN MARGIN (SDM)	B 3.1-1
B 3.1.2	Core Reactivity	B 3.1-7
B 3.1.3	Moderator Temperature Coefficient (MTC)	B 3.1-13
B 3.1.4	Rod Group Alignment Limits	B 3.1-19
B 3.1.5	Shutdown Bank Insertion Limits	B 3.1-30
B 3.1.6	Control Bank Insertion Limits	B 3.1-35
B 3.1.7	Rod Position Indication	B 3.1-41
B 3.1.8	PHYSICS TESTS Exceptions - MODE 2	B 3.1-47
B 3.1.9	Chemical and Volume Control System (CVS) Demineralized Water Isolation Valves	B 3.1-54
B 3.2	POWER DISTRIBUTION LIMITS	B 3.2-1
B 3.2.1	Heat Flux Hot Channel Factor ($F_Q(Z)$) (F_Q Methodology)	B 3.2-1
B 3.2.2	Nuclear Enthalpy Rise Hot Channel Factor ($F_{\Delta H}^N$)	B 3.2-10
B 3.2.3	AXIAL FLUX DIFFERENCE (AFD) (Relaxed Axial Offset Control (RAOC) Methodology)	B 3.2-17
B 3.2.4	QUADRANT POWER TILT RATIO (QPTR)	B 3.2-22
B 3.2.5	OPDMS-Monitored Power Distribution Parameters	B 3.2-28
B 3.3	INSTRUMENTATION	B 3.3-1
B 3.3.1	Reactor Trip System (RTS) Instrumentation	B 3.3-1
B 3.3.2	Engineered Safety Feature Actuation System (ESFAS) Instrumentation	B 3.3-57
B 3.3.3	Post Accident Monitoring (PAM) Instrumentation	B 3.3-134
B 3.3.4	Remote Shutdown Workstation (RSW)	B 3.3-144
B 3.4	REACTOR COOLANT SYSTEM (RCS)	B 3.4-1
B 3.4.1	RCS Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits	B 3.4-1
B 3.4.2	RCS Minimum Temperature for Criticality	B 3.4-6
B 3.4.3	RCS Pressure and Temperature (P/T) Limits	B 3.4-9
B 3.4.4	RCS Loops	B 3.4-17
B 3.4.5	Not Used	B 3.4-23
B 3.4.6	Pressurizer	B 3.4-24

(continued)

TABLE OF CONTENTS

B 3.4	REACTOR COOLANT SYSTEM (RCS) (continued)	
B 3.4.7	Pressurizer Safety Valves	B 3.4-27
B 3.4.8	RCS Operational LEAKAGE	B 3.4-31
B 3.4.9	Minimum RCS Flow	B 3.4-37
B 3.4.10	RCS Leakage Detection Instrumentation	B 3.4-40
B 3.4.11	RCS Specific Activity	B 3.4-45
B 3.4.12	Automatic Depressurization System (ADS) – Operating	B 3.4-49
B 3.4.13	Automatic Depressurization System (ADS) – Shutdown, RCS Intact	B 3.4-53
B 3.4.14	Automatic Depressurization System (ADS) – Shutdown, RCS Open	B 3.4-55
B 3.4.15	Low Temperature Overpressure Protection (LTOP) System	B 3.4-58
B 3.4.16	RCS Pressure Isolation Valve (PIV) Integrity	B 3.4-66
B 3.4.17	Reactor Vessel Head Vent (RVHV)	B 3.4-72
B 3.4.18	Chemical and Volume Control System (CVS) Makeup Isolation Valves	B 3.4-75
B 3.5	PASSIVE CORE COOLING SYSTEM (PXS)	B 3.5-1
B 3.5.1	Accumulators	B 3.5-1
B 3.5.2	Core Makeup Tanks (CMTs) – Operating	B 3.5-8
B 3.5.3	Core Makeup Tanks (CMTs) – Shutdown, RCS Intact	B 3.5-15
B 3.5.4	Passive Residual Heat Removal Heat Exchanger (PRHR HX) – Operating	B 3.5-18
B 3.5.5	Passive Residual Heat Removal Heat Exchanger (PRHR HX) – Shutdown, RCS Intact	B 3.5-25
B 3.5.6	In-containment Refueling Water Storage Tank (IRWST) – Operating	B 3.5-28
B 3.5.7	In-containment Refueling Water Storage Tank (IRWST) – Shutdown, MODE 5	B 3.5-35
B 3.5.8	In-containment Refueling Water Storage Tank (IRWST) – Shutdown, MODE 6	B 3.5-38
B 3.6	CONTAINMENT SYSTEMS	B 3.6-1
B 3.6.1	Containment	B 3.6-1
B 3.6.2	Containment Air Locks	B 3.6-6
B 3.6.3	Containment Isolation Valves	B 3.6-14
B 3.6.4	Containment Pressure	B 3.6-24
B 3.6.5	Containment Air Temperature	B 3.6-27
B 3.6.6	Passive Containment Cooling System (PCS) – Operating	B 3.6-31
B 3.6.7	Passive Containment Cooling System (PCS) – Shutdown	B 3.6-39
B 3.6.8	Containment Penetrations	B 3.6-42
B 3.6.9	pH Adjustment	B 3.6-50

(continued)

TABLE OF CONTENTS (continued)

B 3.7	PLANT SYSTEMS	B 3.7-1
B 3.7.1	Main Steam Safety Valves (MSSVs)	B 3.7-1
B 3.7.2	Main Steam Isolation Valves (MSIVs)	B 3.7-7
B 3.7.3	Main Feedwater Isolation and Control Valves (MFIVs and MFCVs)	B 3.7-15
B 3.7.4	Secondary Specific Activity	B 3.7-20
B 3.7.5	Spent Fuel Pool Water Level	B 3.7-23
B 3.7.6	Main Control Room Emergency Habitability System	B 3.7-26
B 3.7.7	Startup Feedwater Isolation and Control Valves	B 3.7-33
B 3.7.8	Main Steam Line Leakage	B 3.7-37
B 3.7.9	Fuel Storage Pool Makeup Water Sources	B 3.7-40
B 3.7.10	Steam Generator Isolation Valves	B 3.7-45
B 3.8	ELECTRICAL POWER SYSTEMS	B 3.8-1
B 3.8.1	DC Sources – Operating	B 3.8-1
B 3.8.2	DC Sources – Shutdown	B 3.8-12
B 3.8.3	Inverters – Operating	B 3.8-16
B 3.8.4	Inverters – Shutdown	B 3.8-21
B 3.8.5	Distribution Systems – Operating	B 3.8-25
B 3.8.6	Distribution System – Shutdown	B 3.8-38
B 3.8.7	Battery Parameters	B 3.8-42
B 3.9	REFUELING OPERATIONS	B 3.9-1
B 3.9.1	Boron Concentration	B 3.9-1
B 3.9.2	Unborated Water Source Flow Paths	B 3.9-5
B 3.9.3	Nuclear Instrumentation	B 3.9-8
B 3.9.4	Refueling Cavity Water Level	B 3.9-11
B 3.9.5	Containment Penetrations	B 3.9-14
B 3.9.6	Containment Air Filtration System (VFS)	B 3.9-20

1.0 USE AND APPLICATION

1.1 Definitions

-----NOTE-----
The defined terms of this section appear in capitalized type and are applicable throughout these Technical Specifications and Bases.

<u>Term</u>	<u>Definition</u>
ACTIONS	ACTIONS shall be that part of a Specification that prescribes Required Actions to be taken under designated Conditions within specified Completion Times.
ACTUATION DEVICE TEST	An ACTUATION DEVICE TEST is a test of the actuated equipment. This test may consist of verification of actual operation but shall, at a minimum, consist of a continuity check of the associated actuated devices. The ACTUATION DEVICE TEST shall be conducted such that it provides component overlap with the ACTUATION LOGIC TEST.
ACTUATION LOGIC TEST	An ACTUATION LOGIC TEST shall be the application of various simulated or actual input combinations in conjunction with each possible interlock logic state and the verification of the required logic output. The ACTUATION LOGIC TEST shall be conducted such that it provides component overlap with the ACTUATION DEVICE TEST.
AXIAL FLUX DIFFERENCE (AFD)	AFD shall be the difference in normalized flux signals between the top and bottom halves of a two-section excore neutron detector.
CHANNEL CALIBRATION	A CHANNEL CALIBRATION shall be the adjustment, as necessary, of the channel so that it responds within the required range and accuracy to known values of the parameter that the channel monitors. The CHANNEL CALIBRATION shall encompass all devices in the channel required for OPERABILITY.

(continued)

1.1 Definitions

CHANNEL CALIBRATION (continued)

Calibration of instrument channels with resistance temperature detector (RTD) or thermocouple sensors may consist of an inplace qualitative assessment of sensor behavior and normal calibration of the remaining adjustable devices in the channel. The CHANNEL CALIBRATION may be performed by means of any series of sequential, overlapping, or total channel steps.

CHANNEL CHECK

A CHANNEL CHECK shall be the qualitative assessment, by observation, of channel behavior during operation. This determination shall include, where possible, comparison of the channel indication and status to other indications or status derived from independent instrument channels measuring the same parameter.

CHANNEL OPERATIONAL TEST (COT)

A COT shall be the injection of a simulated or actual signal into the channel as close to the sensor as practicable to verify the OPERABILITY of all devices in the channel required for channel OPERABILITY. The COT shall include adjustments, as necessary, of the required alarm, interlock, and trip setpoints required for channel OPERABILITY such that the setpoints are within the necessary range and accuracy. The COT may be performed by means of any series of sequential, overlapping, or total channel steps.

CORE ALTERATION

CORE ALTERATION shall be the movement of any fuel, sources, or reactivity control components, within the reactor vessel with the vessel head removed and fuel in the vessel. Suspension of CORE ALTERATIONS shall not preclude completion of movement of a component to a safe position.

(continued)

1.1 Definitions (continued)

CORE OPERATING
LIMITS REPORT (COLR)

The COLR is the unit specific document that provides cycle specific parameter limits for the current reload cycle. These cycle specific parameter limits shall be determined for each reload cycle in accordance with Specification 5.6.5. Plant operation within these parameter limits is addressed in individual Specifications.

DOSE EQUIVALENT I-131

DOSE EQUIVALENT I-131 shall be that concentration of I-131 (microcuries/gram) that alone would produce the same committed effective dose equivalent as the quantity and isotopic mixture of I-130, I-131, I-132, I-133, I-134, and I-135 actually present. The dose conversion factors used for this calculation shall be those listed in Table 2.1 of EPA Federal Guidance Report No. 11, "Limiting Values of Radionuclide Intake and Air Concentration and Dose Conversion Factors for Inhalation, Submersion, and Ingestion," EPA-520/1-88-020, September 1988.

DOSE EQUIVALENT Xe-133

DOSE EQUIVALENT Xe-133 shall be that concentration of Xe-133 (microcuries per gram) that alone would produce the same effective dose equivalent as the quantity and isotopic mixture of noble gases (Kr-85m, Kr-85, Kr-87, Kr-88, Xe-131m, Xe-133m, Xe-133, Xe-135m, Xe-135, and Xe-138) actually present. The dose conversion factors used for this calculation shall be those listed in Table III.1 of EPA Federal Guidance Report No. 12, "External Exposure to Radionuclides in Air, Water, and Soil," EPA 402-R-93-081, September 1993.

ENGINEERED SAFETY
FEATURE (ESF) RESPONSE
TIME

The ESF RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its actuation setpoint at the channel sensor until the ESF equipment is capable of performing its safety function (i.e., the valves travel to their required positions). The response time may be measured by means of any series of sequential, overlapping, or total steps so that the entire response time is measured. In lieu of measurement, response time may be verified for selected components provided that the components and methodology for verification have been previously reviewed and approved by the NRC.

(continued)

1.1 Definitions (continued)

LEAKAGE

LEAKAGE shall be:

a. Identified LEAKAGE

1. LEAKAGE, such as that from seals or valve packing, that is captured and conducted to collection systems or a sump or collecting tank;
2. LEAKAGE into the containment atmosphere from sources that are both specifically located and known either not to interfere with the operation of leakage detection systems or not to be pressure boundary LEAKAGE;
3. Reactor Coolant System (RCS) LEAKAGE through a steam generator (SG) to the Secondary System; or
4. RCS LEAKAGE through the passive residual heat removal heat exchanger (PRHR HX) to the In-containment Refueling Water Storage Tank (IRWST).

b. Unidentified LEAKAGE

All LEAKAGE that is not identified LEAKAGE.

c. Pressure Boundary LEAKAGE

LEAKAGE (except SG LEAKAGE and PRHR HX tube LEAKAGE) through a nonisolatable fault in a RCS component body, pipe wall, or vessel wall.

MODE

A MODE shall correspond to any one inclusive combination of core reactivity condition, power level, average reactor coolant temperature, and reactor vessel head closure bolt tensioning specified in Table 1.1-1 with fuel in the reactor vessel.

(continued)

1.1 Definitions (continued)

OPERABLE-OPERABILITY

A system, subsystem, train, component, or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified safety function(s) and when all necessary attendant instrumentation, controls, normal or emergency electrical power, cooling and seal water, lubrication, and other auxiliary equipment that are required for the system, subsystem, train, component, or device to perform its specified safety function(s) are also capable of performing their related support function(s).

PHYSICS TESTS

PHYSICS TESTS shall be those tests performed to measure the fundamental nuclear characteristics of the reactor core and related instrumentation. These tests are:

- a. Described in Chapter 14, Initial Test Program;
- b. Authorized under the provisions of 10 CFR 50.59;
or
- c. Otherwise approved by the Nuclear Regulatory Commission.

PRESSURE AND
TEMPERATURE LIMITS
REPORT (PTLR)

The PTLR is the unit specific document that provides the reactor vessel pressure and temperature limits, including heatup and cooldown rates, for the current reactor vessel fluence period. These pressure and temperature limits shall be determined for each fluence period in accordance with Specification 5.6.6. Plant operation within these operating limits is addressed in LCO 3.4.3, "RCS Pressure and Temperature (P/T) Limits" and LCO 3.4.15, "Low Temperature Overpressure Protection (LTOP) System."

(continued)

1.1 Definitions (continued)

QUADRANT POWER TILT RATIO (QPTR)	QPTR shall be the ratio of the maximum upper excore detector calibrated output to the average of the upper excore detector calibrated outputs, or the ratio of maximum lower excore detector calibrated output to the average of the lower excore detector calibrated outputs, whichever is greater.
RATED THERMAL POWER (RTP)	RTP shall be a total reactor core heat transfer rate to the reactor coolant of 3400 Mwt.
REACTOR TRIP CHANNEL OPERATIONAL TEST (RTCOT)	A RTCOT shall be the injection of a simulated or actual signal into the RT (Reactor Trip) CHANNEL as close to the sensor as practicable to verify OPERABILITY of the required interlock and/or trip functions. The REACTOR TRIP CHANNEL OPERATIONAL TEST may be performed by means of a series of sequential, overlapping, or total channel steps so that the entire channel is tested from the signal conditioner through the trip logic.
REACTOR TRIP SYSTEM (RTS) RESPONSE TIME	The RTS RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its RTS trip setpoint at the channel sensor until loss of stationary gripper coil voltage. The response time may be measured by means of any series of sequential, overlapping, or total steps so that the entire response time is measured. In lieu of measurement, response time may be verified for selected components provided that the components and methodology for verification have been previously reviewed and approved by the NRC.
SHUTDOWN MARGIN (SDM)	<p>SDM shall be the instantaneous amount of reactivity by which the reactor is subcritical or would be subcritical from its present condition assuming:</p> <p>a. All rod cluster control assemblies (RCCAs) and grey rod cluster assemblies (GRCAs) are fully inserted except for the single assembly of highest reactivity worth, which is assumed to be fully withdrawn. However, with all RCCAs and GRCAs verified fully inserted by two independent means, it is not necessary to account for a stuck RCCA or GRCA in the SDM calculation. With any rod assembly(s) not capable of being fully inserted, the reactivity worth of these assemblies must be accounted for in the determination of SDM; and</p>

(continued)

1.1 Definitions

SHUTDOWN MARGIN (SDM)
(continued)

b. In MODES 1 and 2, the fuel and moderator temperatures are changed to the nominal zero power design level.

STAGGERED TEST BASIS

A STAGGERED TEST BASIS shall consist of the testing of one of the systems, subsystems, channels, or other designated components during the interval specified by the Surveillance Frequency, so that all systems, subsystems, channels, or other designated components are tested during n Surveillance Frequency intervals, where n is the total number of systems, subsystems, channels, or other designated components in the associated function.

THERMAL POWER

THERMAL POWER shall be the total reactor core heat transfer rate to the reactor coolant.

TRIP ACTUATING DEVICE
OPERATIONAL TEST (TADOT)

A TADOT shall consist of operating the trip actuating device and verifying the OPERABILITY of all devices in the channel required for trip actuating device OPERABILITY. The TADOT shall include adjustment, as necessary, of the trip actuating device so that it actuates at the required setpoint within the required accuracy. The TADOT may be performed by means of any series of sequential, overlapping, or total channel steps.

Table 1.1-1 (page 1 of 1)
MODES

MODES	TITLE	REACTIVITY CONDITION (K_{eff})	% RATED THERMAL POWER(a)	AVERAGE REACTOR COOLANT TEMPERATURE (°F)
1	Power Operation	≥ 0.99	> 5	NA
2	Startup	≥ 0.99	≤ 5	NA
3	Hot Standby	< 0.99	NA	> 420
4	Safe Shutdown(b)	< 0.99	NA	$420 \geq T_{avg} > 200$
5	Cold Shutdown(b)	< 0.99	NA	≤ 200
6	Refueling(c)	NA	NA	NA

(a) Excluding decay heat.

(b) All reactor vessel head closure bolts fully tensioned.

(c) One or more reactor vessel head closure bolts less than fully tensioned.

1.0 USE AND APPLICATION

1.2 Logical Connectors

PURPOSE The purpose of this section is to explain the meaning of logical connectors.

Logical connectors are used in Technical Specifications to discriminate between, and yet connect, discrete Conditions, Required Actions, Completion Times, Surveillances, and Frequencies. The only logical connectors that appear in Technical Specifications are AND and OR. The physical arrangement of these connectors constitutes logical conventions with specific meaning.

BACKGROUND Several levels of logic may be used to state Required Actions. These levels are identified by the placement (or nesting) of the logical connectors and the number assigned to each Required Action. The first level of logic is identified by the first digit of the number assigned to a Required Action and the placement of the logical connector in the first level of nesting (i.e., left justified with the number of the Required Action). The successive levels of logic are identified by additional digits of the Required Action number and by successive indentions of the logical connectors.

When logical connectors are used to state a Condition, Completion Time, Surveillance, or Frequency, only the first level of logic is used, and the logical connector is left justified with the statement of the Condition, Completion Time, Surveillance, or Frequency.

(continued)

1.2 Logical Connectors (continued)

EXAMPLES

The following examples illustrate the use of logical connectors.

EXAMPLE 1.2-1

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. LCO not met.	A.1 Verify ... <u>AND</u> A.2 Restore...	

In this example, the logical connector AND is used to indicate that when in Condition A, both Required Actions A.1 and A.2 must be completed.

(continued)

1.2 Logical Connectors

EXAMPLES
(continued)

EXAMPLE 1.2-2

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. LCO not met.	A.1 Trip.. <u>OR</u> A.2.1 Verify... <u>AND</u> A.2.2.1 Reduce.. <u>OR</u> A.2.2.2 Perform.. <u>OR</u> A.3 Align...	

This example represents a more complicated use of logical connectors. Required Actions A.1, A.2, and A.3 are alternative choices, only one of which must be performed as indicated by the use of the logical connector OR and the left justified placement. Any one of these three Actions may be chosen. If A.2 is chosen, then both A.2.1 and A.2.2 must be performed as indicated by the logical connector AND. Required Action A.2.2 is met by performing A.2.2.1 or A.2.2.2. The indented position of the logical connector OR indicates that A.2.2.1 and A.2.2.2 are alternative choices, only one of which must be performed.

1.0 USE AND APPLICATION

1.3 Completion Times

PURPOSE	The purpose of this section is to establish the Completion Time convention and to provide guidance for its use.
BACKGROUND	Limiting Conditions for Operation (LCOs) specify minimum requirements for ensuring safe operation of the unit. The ACTIONS associated with an LCO state Conditions that typically describe the ways in which the requirements of the LCO can fail to be met. Specified with each stated Condition are Required Action(s) and Completion Time(s).
DESCRIPTION	<p>The Completion Time is the amount of time allowed for completing a Required Action. It is referenced to the time of discovery of a situation (e.g., inoperable equipment or variable not within limits) that requires entering an ACTIONS Condition unless otherwise specified, providing the unit is in a MODE or specified condition stated in the Applicability of the LCO. Required Actions must be completed prior to the expiration of the specified Completion Time. An ACTIONS Condition remains in effect and the Required Actions apply until the Condition no longer exists or the unit is not within the LCO Applicability.</p> <p>If situations are discovered that require entry into more than one Condition at a time within a single LCO (multiple Conditions), the Required Actions for each Condition must be performed within the associated Completion Time. When in multiple Conditions, separate Completion Times are tracked for each Condition starting from the time of discovery of the situation that required entry into the Condition.</p> <p>Once a Condition has been entered, subsequent trains, subsystems, components, or variables expressed in the Condition, discovered to be inoperable or not within limits, will <u>not</u> result in separate entry into the Condition, unless specifically stated. The Required Actions of the Condition continue to apply to each additional failure, with Completion Times based on initial entry into the Condition.</p>

(continued)

1.3 Completion Times

DESCRIPTION
(continued)

However, when a subsequent train, subsystem, component, or variable, expressed in the Condition, is discovered to be inoperable or not within limits, the Completion Time(s) may be extended. To apply this Completion Time extension, two criteria must first be met. The subsequent inoperability:

- a. Must exist concurrent with the first inoperability; and
- b. Must remain inoperable or not within limits after the first inoperability is resolved.

The total Completion Time allowed for completing a Required Action to address the subsequent inoperability shall be limited to the more restrictive of either:

- a. The stated Completion Time, as measured from the initial entry into the Condition, plus an additional 24 hours; or
- b. The stated Completion Time as measured from discovery of the subsequent inoperability.

The above Completion Time extensions do not apply to those Specifications that have exceptions that allow completely separate re-entry into the Condition (for each train, subsystem, component, or variable expressed in the Condition) and separate tracking of Completion Times based on this re-entry. These exceptions are stated in individual Specifications.

The above Completion Time extension does not apply to a Completion Time with a modified "time zero." This modified "time zero" may be expressed as a repetitive time (i.e., "once per 8 hours," where the Completion Time is referenced from a previous completion of the Required Action versus the time of Condition entry) or as a time modified by the phrase "from discovery" Example 1.3-3 illustrates one use of this type of Completion Time. The 10 day Completion Time specified for Conditions A and B in example 1.3-3 may not be extended.

(continued)

1.3 Completion Times (continued)

EXAMPLES

The following examples illustrate the use of Completion Times with different types of Conditions and changing Conditions.

EXAMPLE 1.3-1

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3.	6 hours
	<u>AND</u> B.2 Be in MODE 5.	36 hours

Condition B has two Required Actions. Each Required Action has its own separate Completion Time. Each Completion Time is referenced to the time that Condition B is entered.

The Required Actions of Condition B are to be in MODE 3 within 6 hours AND in MODE 5 in 36 hours. A total of 6 hours is allowed for reaching MODE 3 and a total of 36 hours (not 42 hours) is allowed for reaching MODE 5 from the time that Condition B was entered. If MODE 3 is reached within 3 hours, the time allowed for reaching MODE 5 is the next 33 hours because the total time allowed for reaching MODE 5 is 36 hours.

If Condition B is entered while in MODE 3, the time allowed for reaching MODE 5 is the next 36 hours.

(continued)

1.3 Completion Times

EXAMPLES (continued)

EXAMPLE 1.3-2

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One valve inoperable.	A.1 Restore valve to OPERABLE status.	7 days
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3.	6 hours
	<u>AND</u> B.2 Be in MODE 5.	36 hours

When a valve is declared inoperable, Condition A is entered. If the valve is not restored to OPERABLE status within 7 days, Condition B is also entered and the Completion time clocks for Required Actions B.1 and B.2 start. If the inoperable valve is restored to OPERABLE status after Condition B is entered, Condition A and B are exited, and therefore, the Required Actions of Condition B may be terminated.

When a second valve is declared inoperable while the first valve is still inoperable, Condition A is not re-entered for the second valve. LCO 3.0.3 is entered, since the ACTIONS do not include a Condition for more than one inoperable valve. The Completion Time clock for Condition A does not stop after LCO 3.0.3 is entered, but continues to be tracked from the time Condition A was initially entered.

While in LCO 3.0.3, if one of the inoperable valves is restored to OPERABLE status and the Completion Time for Condition A has not expired, LCO 3.0.3 may be exited and operation continued in accordance with Condition A.

(continued)

1.3 Completion Times

EXAMPLES

EXAMPLE 1.3-2 (continued)

While in LCO 3.0.3, if one of the inoperable valves is restored to OPERABLE status and the Completion Time for Condition A has expired, LCO 3.0.3 may be exited and operation continued in accordance with Condition B. The Completion Time for Condition B is tracked from the time the Condition A Completion Time expired.

On restoring one of the valves to OPERABLE status the Condition A Completion Time is not reset, but continues from the time the first valve was declared inoperable. This Completion Time may be extended if the valve restored to OPERABLE status was the first inoperable valve. A 24 hour extension to the stated 7 days is allowed, provided this does not result in the second valve being inoperable for > 7 days.

(continued)

1.3 Completion Times

EXAMPLES (continued)

EXAMPLE 1.3-3

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One Function X train inoperable.	A.1 Restore Function X train to OPERABLE status.	7 days <u>AND</u> 10 days from discovery of failure to meet the LCO
B. One Function Y train inoperable.	B.1 Restore Function Y train to OPERABLE status.	72 hours <u>AND</u> 10 days from discovery of failure to meet the LCO
C. One Function X train inoperable. <u>AND</u> One Function Y train inoperable.	C.1 Restore Function X train to OPERABLE status. <u>OR</u> C.2 Restore Function Y train to OPERABLE status.	72 hours 72 hours

When one Function X train and one Function Y train are inoperable, Condition A and Condition B are concurrently applicable. The Completion Times for Condition A and

(continued)

1.3 Completion Times

EXAMPLES

EXAMPLE 1.3-3 (continued)

Condition B are tracked separately for each train starting from the time each train was declared inoperable and the Condition was entered. A separate Completion Time is established for Condition C and tracked from the time the second train was declared inoperable (i.e., the time the situation described in Condition C was discovered).

If Required Action C.2 is completed within the specified Completion Time, Conditions B and C are exited. If the Completion Time for Required Action A.1 has not expired, operation may continue in accordance with Condition A. The remaining Completion Time in Condition A is measured from the time the affected train was declared inoperable (i.e., initial entry into Condition A).

The Completion Times of Conditions A and B are modified by a logical connector with a separate 10 day Completion Time measured from the time it was discovered the LCO was not met. In this example, without the separate Completion Time, it would be possible to alternate between Conditions A, B, and C in such a manner that operation could continue indefinitely without ever restoring systems to meet the LCO. The separate Completion Time modified by the phrase "from discovery of failure to meet the LCO" is designed to prevent indefinite continued operation while not meeting the LCO. This Completion Time allows for an exception to the normal "time zero" for beginning the Completion Time "clock". In this instance, the Completion Time "time zero" is specified as commencing at the time the LCO was initially not met, instead of at the time the associated Condition was entered.

(continued)

1.3 Completion Times

EXAMPLES (continued)

EXAMPLE 1.3-4

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more valves inoperable.	A.1 Restore valve(s) to OPERABLE status.	4 hours
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3.	6 hours
	<u>AND</u> B.2 Be in MODE 5.	36 hours

A single Completion Time is used for any number of valves inoperable at the same time. The Completion Time associated with Condition A is based on the initial entry into Condition A and is not tracked on a per valve basis. Declaring subsequent valves inoperable, while Condition A is still in effect, does not trigger the tracking of separate Completion Times.

Once one of the valves has been restored to OPERABLE status, the Condition A Completion Time is not reset, but continues from the time the first valve was declared inoperable. The Completion Time may be extended if the valve restored to OPERABLE status was the first inoperable valve. The Condition A Completion Time may be extended for up to 4 hours provided this does not result in any subsequent valve being inoperable for > 4 hours. If the Completion Time of 4 hours (including the extension) expires while one or more valves are still inoperable, Condition B is entered.

(continued)

1.3 Completion Times

EXAMPLES
(continued)

EXAMPLE 1.3-5

ACTIONS

-----NOTE-----
Separate Condition entry is allowed for each inoperable valve.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more valves inoperable.	A.1 Restore valve to OPERABLE status.	4 hours
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3.	6 hours
	<u>AND</u> B.2 Be in MODE 5.	36 hours

The Note above the ACTIONS Table is a method of modifying how the Completion Time is tracked. If this method of modifying how the Completion Time is tracked was only applicable to a specific Condition, the Note would appear in that Condition rather than at the top of the ACTIONS Table.

The Note allows Condition A to be entered separately for each inoperable valve, and Completion Times tracked on a per valve basis. When a valve is declared inoperable, Condition A is entered and its Completion Time starts. If subsequent valves are declared inoperable, Condition A is entered for each valve and separate Completion Times start and are tracked for each valve.

If the Completion Time associated with a valve in Condition A expires, Condition B is entered for that valve. If the Completion Times associated with subsequent valves in

(continued)

1.3 Completion Times

EXAMPLES

EXAMPLE 1.3-5 (continued)

Condition A expire, Condition B is entered separately for each valve and separate Completion Times start and are tracked for each valve. If a valve which caused entry into Condition B is restored to OPERABLE status, Condition B is exited for that valve. Since the Note in this example allows multiple Condition entry and tracking of separate Completion Times, Completion Time extensions do not apply.

EXAMPLE 1.3-6

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One channel inoperable.	A.1 Perform SR 3.x.x.x.	Once per 8 hours
	<u>OR</u> A.2 Reduce THERMAL POWER to $\leq 50\%$ RTP.	8 hours
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3.	6 hours

Entry into Condition A offers a choice between Required Action A.1 or A.2. Required Action A.1 has a "once per" Completion Time, which qualifies for the 25% extension, per SR 3.0.2, to each performance after the initial performance. The initial 8 hours interval of Required Action A.1 begins when Condition A is entered and the initial performance of

(continued)

1.3 Completion Times

EXAMPLES

EXAMPLE 1.3-6 (continued)

Required Action A.1 must be complete within the first 8 hour interval. If Required Action A.1 is followed, and the Required Action is not met within the Completion Time (plus the extension allowed by SR 3.0.2), Condition B is entered. If Required Action A.2 is followed and the Completion Time of 8 hours is not met, Condition B is entered.

If after entry into Condition B, Required Action A.1 or A.2 is met, Condition B is exited and operation may then continue in Condition A.

EXAMPLE 1.3-7

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One subsystem inoperable.	A.1 Verify affected subsystem isolated.	1 hour <u>AND</u> Once per 8 hours thereafter
	<u>AND</u> A.2 Restore subsystem to OPERABLE status.	72 hours
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3.	6 hours
	<u>AND</u> B.2 Be in MODE 5.	36 hours

(continued)

1.3 Completion Times

EXAMPLES

EXAMPLE 1.3-7 (continued)

Required Action A.1 has two Completion Times. The 1 hour Completion Time begins at the time the Condition is entered and each "Once per 8 hours thereafter" interval begins upon performance of Required Action A.1.

If after Condition A is entered, Required Action A.1 is not met within either the initial 1 hour, or any subsequent 8 hour interval from the previous performance (plus the extension allowed by SR 3.0.2), Condition B is entered. The Completion Time clock for Condition A does not stop after Condition B is entered, but continues from the time Condition A was initially entered. If Required Action A.1 is met after Condition B is entered, Condition B is exited and operation may continue in accordance with Condition A, provided the Completion Time for Required Action A.2 has not expired.

IMMEDIATE COMPLETION TIME

When "Immediately" is used as a Completion Time, the Required Action should be pursued without delay and in a controlled manner.

1.0 USE AND APPLICATION

1.4 Frequency

PURPOSE	The purpose of this section is to define the proper use and application of Frequency requirements.
---------	----------------------------------------------------------------------------------------------------

DESCRIPTION	Each Surveillance Requirement (SR) has a specified Frequency in which the surveillance must be met in order to meet the associated LCO. An understanding of the correct application of the specified Frequency is necessary for compliance with the SR.
-------------	---------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------

The "specified Frequency" is referred to throughout this section and each of the Specifications of Section 3.0, Surveillance Requirement (SR) Applicability. The "specified Frequency" consists of the requirements of the Frequency column of each SR as well as certain Notes in the Surveillance column that modify performance requirements.

Sometimes special situations dictate when the requirements of a Surveillance are to be met. They are "otherwise stated" conditions allowed by SR 3.0.1. They may be stated as clarifying Notes in the Surveillance, as part of the Surveillances, or both.

Situations where a Surveillance could be required (i.e., its Frequency could expire), but where it is not possible or not desired that it be performed until sometime after the associated LCO is within its Applicability, represent potential SR 3.0.4 conflicts. To avoid these conflicts, the SR (i.e., the Surveillance or the Frequency) is stated such that it is only "required" when it can be and should be performed. With an SR satisfied, SR 3.0.4 imposes no restriction.

The use of "met" or "performed" in these instances conveys specific meanings. A Surveillance is "met" only when the acceptance criteria are satisfied. Known failure of the requirements of a Surveillance, even without a Surveillance specifically being "performed," constitutes a Surveillance not "met." "Performance" refers only to the requirement to specifically determine the ability to meet the acceptance criteria.

Some Surveillances contain notes that modify the Frequency of performance or the conditions during which the acceptance criteria must be satisfied. For these Surveillances, the MODE-entry restrictions of SR 3.0.4 may not apply. Such a Surveillance is not required to be performed prior to entering a MODE or other specified condition in the Applicability of the associated LCO if any of the following three conditions are satisfied:

- a. The Surveillance is not required to be met in the MODE or other specified condition to be entered; or
- b. The Surveillance is required to be met in the MODE or other specified condition to be entered, but has been performed within the specified Frequency (i.e., it is current) and is known not to be failed; or
- c. The Surveillance is required to be met, but not performed, in the MODE or other specified condition to be entered, and is known not to be failed.

Examples 1.4-3, 1.4-4, 1.4-5, and 1.4-6 discusses these special situations.

EXAMPLES

The following examples illustrate the various ways that Frequencies are specified. In these examples, the Applicability of the LCO (LCO not shown) is MODES 1, 2, and 3.

EXAMPLE 1.4-1

SURVEILLANCE REQUIREMENT

SURVEILLANCE	FREQUENCY
Perform CHANNEL CHECK.	12 hours

(continued)

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1.4 Frequency

EXAMPLES

EXAMPLE 1.4-1 (continued)

Example 1.4-1 contains the type of SR most often encountered in the Technical Specifications (TS). The Frequency specifies an interval (12 hours) during which the associated surveillance must be performed at least one time. Performance of the surveillance initiates the subsequent interval. Although the Frequency is stated as 12 hours, an extension of the time interval to 1.25 times the stated Frequency is allowed by SR 3.0.2 for operational flexibility. The measurement of this interval continues at all times, even when the SR is not required to be met per SR 3.0.1 (such as when the equipment is inoperable, a variable is outside the specified limits, or the Unit is outside the Applicability of the LCO). If the interval specified by SR 3.0.2 is exceeded while the unit is in a MODE or other specified condition in the Applicability of the LCO, and the performance of the Surveillance is not otherwise modified (refer to Example 1.4-3), then SR 3.0.3 becomes applicable.

If the interval specified by SR 3.0.2 is exceeded while the unit is not in a MODE or other specified condition in the Applicability of the LCO for which performance of the SR is required, the Surveillance must be performed within the Frequency requirements of SR 3.0.2 prior to entry into the MODE or other specified condition. Failure to do so would result in a violation of SR 3.0.4.

EXAMPLE 1.4-2

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
Verify flow is within limits.	Once within 12 hours after $\geq 25\%$ RTP <u>AND</u> 24 hours thereafter

(continued)

1.4 Frequency

EXAMPLES

EXAMPLE 1.4-2 (continued)

Example 1.4-2 has two Frequencies. The first is a one time performance Frequency, and the second is of the type shown in Example 1.4-1. The logical connector "AND" indicates that both Frequency requirements must be met. Each time the reactor power is increased from a power level $< 25\%$ RTP to $\geq 25\%$ RTP, the surveillance must be performed within 12 hours.

The use of "Once" indicates a single performance will satisfy the specified Frequency (assuming no other Frequencies are connected by "AND"). This type of Frequency does not qualify for the 25% extension allowed by SR 3.0.2. "Thereafter" indicates future performances must be established per SR 3.0.2, but only after a specified condition is first met (i.e., the "once" performance in this example). If reactor power decreases to $< 25\%$ RTP, the measurement of both intervals stops. New intervals start upon reactor power reaching 25% RTP.

EXAMPLE 1.4-3SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
-----NOTE----- Not required to be performed until 12 hours after $\geq 25\%$ RTP. -----	
Perform channel adjustment.	7 days

The interval continues, whether or not the unit operation is $< 25\%$ RTP between performances.

As the Note modifies the required performance of the Surveillance, it is construed to be part of the "specified Frequency." Should the 7 day interval be exceeded while operation is $< 25\%$ RTP, this Note allows 12 hours after power reaches $\geq 25\%$ RTP to perform the Surveillance. The Surveillance is still considered to be performed within

(continued)

1.4 Frequency

EXAMPLES

EXAMPLE 1.4-3 (continued)

the "specified Frequency." Therefore, if the Surveillance were not performed within the 7 day (plus the extension allowed by SR 3.0.2) interval, but operation was < 25% RTP, it would not constitute a failure of the SR or failure to meet the LCO. Also, no violation of SR 3.0.4 occurs when changing MODES, even with the 7 day Frequency not met, provided operation does not exceed 12 hours with power \geq 25% RTP.

Once the unit reaches 25% RTP, 12 hours would be allowed for completing the Surveillance. If the Surveillance were not performed within this 12 hour interval, there would then be a failure to perform a Surveillance within the specified Frequency, and the provisions of SR 3.0.3 would apply.

EXAMPLE 1.4-4

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
<p>-----NOTE----- Only required to be met in MODE 1. -----</p> <p>Verify leakage rates are within limits.</p>	24 hours

Example 1.4-4 specifies that the requirements of this Surveillance do not have to be met until the unit is in MODE 1. The interval measurement for the Frequency of this Surveillance continues at all times, as described in Example 1.4-1. However, the Note constitutes an "otherwise stated" exception to the Applicability of this Surveillance. Therefore, if the Surveillance were not performed within the 24 hour interval (plus the extension allowed by SR 3.0.2), but the unit was not in MODE 1, there would be no failure of the SR nor failure to meet the LCO. Therefore, no violation of SR 3.0.4 occurs when changing MODES, even with the 24 hour Frequency exceeded, provided the MODE change was not made into MODE 1. Prior to entering MODE 1 (assuming again that the 24 hour Frequency were not met), SR 3.0.4 would require satisfying the SR.

EXAMPLE 1.4-5

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
<p>-----NOTE----- Only required to be performed in MODE 1. -----</p> <p>Perform complete cycle of the valve.</p>	7 days

The interval continues, whether or not the unit operation is in MODE 1, 2, or 3 (the assumed Applicability of the associated LCO) between performances.

As the Note modifies the required performance of the Surveillance, the Note is construed to be part of the "specified Frequency." Should the 7 day interval be exceeded while operation is not in MODE 1, this Note allows entry into and operation in MODES 2 and 3 to perform the Surveillance. The Surveillance is still considered to be performed within the "specified Frequency" if completed prior to entering MODE 1. Therefore, if the Surveillance were not performed within the 7 day (plus the extension allowed by SR 3.0.2) interval, but operation was not in MODE 1, it would not constitute a failure of the SR or failure to meet the LCO. Also, no violation of SR 3.0.4 occurs when changing MODES, even with the 7 day Frequency not met, provided operation does not result in entry into MODE 1.

Once the unit reaches MODE 1, the requirement for the Surveillance to be performed within its specified Frequency applies and would require that the Surveillance had been performed. If the Surveillance were not performed prior to entering MODE 1, there would then be a failure to perform a Surveillance within the specified Frequency, and the provisions of SR 3.0.3 would apply.

EXAMPLE 1.4-6

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
<p>-----NOTE----- Not required to be met in MODE 3. -----</p> <p>Verify parameter is within limits.</p>	24 hours

Example 1.4-[6] specifies that the requirements of this Surveillance do not have to be met while the unit is in MODE 3 (the assumed Applicability of the associated LCO is MODES 1, 2, and 3). The interval measurement for the Frequency of this Surveillance continues at all times, as described in Example 1.4-1. However, the Note constitutes an "otherwise stated" exception to the Applicability of this Surveillance. Therefore, if the Surveillance were not performed within the 24 hour interval (plus the extension allowed by SR 3.0.2), and the unit was in MODE 3, there would be no failure of the SR nor failure to meet the LCO. Therefore, no violation of SR 3.0.4 occurs when changing MODES to enter MODE 3, even with the 24 hour Frequency exceeded, provided the MODE change does not result in entry into MODE 2. Prior to entering MODE 2 (assuming again that the 24 hour Frequency were not met), SR 3.0.4 would require satisfying the SR.

2.0 SAFETY LIMITS

2.1 SLs

2.1.1 Reactor Core SLs

In MODES 1 and 2, the combination of THERMAL POWER, Reactor Coolant System (RCS) highest loop average temperature, and pressurizer pressure shall not exceed the limits specified in the COLR; and the following SLs shall not be exceeded:

2.1.1.1 The departure from nucleate boiling ratio (DNBR) shall be maintained \geq [1.14 for the WRB-2M DNB correlations].

2.1.1.2 The peak fuel centerline temperature shall be maintained $<$ [5080°F, decreasing by 58°F per 10,000 MWD/MTU of burnup].

2.1.2 RCS Pressure SL

In MODES 1, 2, 3, 4, and 5 the RCS pressure shall be maintained \leq 2733.5 psig.

2.2 SL Violations

2.2.1 If SL 2.1.1 is violated, restore compliance and be in MODE 3 within 1 hour.

2.2.2 If SL 2.1.2 is violated:

2.2.2.1 In MODE 1 or 2, restore compliance and be in MODE 3 within 1 hour.

2.2.2.2 In MODE 3, 4, or 5, restore compliance within 5 minutes.

Figure 2.1.1-1 not used.

3.0 LIMITING CONDITIONS FOR OPERATION (LCO) APPLICABILITY

LCO 3.0.1 LCOs shall be met during the MODES or other specified conditions in the Applicability, except as provided in LCO 3.0.2.

LCO 3.0.2 Upon discovery of a failure to meet an LCO, the Required Actions of the associated Conditions shall be met, except as provided in LCO 3.0.5 and 3.0.6.

If the LCO is met, or is no longer applicable prior to expiration of the specified Completion Time(s), completion of the Required Action(s) is not required, unless otherwise stated.

LCO 3.0.3 When an LCO is not met and the associated ACTIONS are not met, an associated ACTION is not provided, or if directed by the associated ACTIONS, the unit shall be placed in a MODE or other specified condition in which the LCO is not applicable. Action shall be initiated within 1 hour to place the unit, as applicable, in:

- a. MODE 3 within 7 hours; and
- b. MODE 4 within 13 hours; and
- c. MODE 5 within 37 hours.

Exceptions to this Specification are stated in the individual Specifications.

Where corrective measures are completed that permit operation in accordance with the LCO or ACTIONS, completion of the actions required by LCO 3.0.3 is not required.

LCO 3.0.3 is only applicable in MODES 1, 2, 3, and 4.

LCO 3.0.4 When an LCO is not met, entry into a MODE or other specified condition in the Applicability shall not be made except when the associated ACTIONS to be entered permit continued operation in the MODE or other specified condition in the Applicability for an unlimited period of time. This Specification shall not prevent changes in MODES or other

(continued)

3.0 LCO APPLICABILITY

LCO 3.0.4
(continued) specified conditions in the Applicability that are required to comply with ACTIONS or are part of a shutdown of the unit.

Exceptions to this Specification are stated in the individual Specifications.

LCO 3.0.4 is only applicable for entry into a MODE or other specified condition in the Applicability in MODES 1, 2, 3, and 4.

LCO 3.0.5 Equipment removed from service or declared inoperable to comply with ACTIONS may be returned to service under administrative control solely to perform testing required to demonstrate its OPERABILITY or the OPERABILITY of other equipment. This is an exception to LCO 3.0.2 for the system returned to service under administrative control to perform the test required to demonstrate OPERABILITY.

LCO 3.0.6 When a supported system LCO is not met solely due to a support system LCO not being met, the Conditions and Required Actions associated with this supported system are not required to be entered. Only the support system LCO ACTIONS are required to be entered. This is an exception to LCO 3.0.2 for the supported system. In this event, additional evaluations and limitations may be required in accordance with Specification 5.5.8, "Safety Function Determination Program (SFDP)." If a loss of safety function is determined to exist by this program, the appropriate Conditions and Required Actions of the LCO in which the loss of safety function exists are required to be entered.

When a support system's Required Action directs a supported system to be declared inoperable or directs entry into Conditions and Required Actions for a supported system, the applicable Conditions and Required Actions shall be entered in accordance with LCO 3.0.2.

(continued)

3.0 LCO APPLICABILITY (continued)

LCO 3.0.7 Test Exception LCO 3.1.8 allows specified Technical Specification (TS) requirements to be changed to permit performance of special tests and operations. Unless otherwise specified, all other TS requirements remain unchanged. Compliance with Test Exception LCOs is optional. When a Test Exception LCO is desired to be met but is not met, the ACTIONS of the Test Exception LCO shall be met. When a Test Exception LCO is not desired to be met, entry into a MODE or other specified condition in the Applicability shall be made in accordance with the other applicable Specifications.

LCO 3.0.8 When an LCO is not met and the associated ACTIONS are not met or an associated ACTION is not provided, action shall be initiated within 1 hour to:

- a. Restore inoperable equipment and
- b. Monitor Safety System Shutdown Monitoring Trees parameters

Exceptions to this Specification are stated in the individual Specifications.

Where corrective measures are completed that permit operation in accordance with the LCO or ACTIONS, completion of the actions required by LCO 3.0.8 is not required.

LCO 3.0.8 is only applicable in MODES 5 and 6.

3.0 SURVEILLANCE REQUIREMENT (SR) APPLICABILITY

SR 3.0.1 SRs shall be met during the MODES or other specified Conditions in the Applicability of individual LCOs, unless otherwise stated in the SR. Failure to meet a Surveillance, whether such failure is experienced during the performance of the surveillance or between performances of the Surveillance, shall be a failure to meet the LCO. Failure to perform a Surveillance within the specified Frequency shall be failure to meet the LCO except as provided in SR 3.0.3. Surveillances do not have to be performed on inoperable equipment or variables outside specified limits.

SR 3.0.2 The specified Frequency for each SR is met if the Surveillance is performed within 1.25 times the interval specified in the Frequency, as measured from the previous performance or as measured from the time a specified condition of the Frequency is met.

For Frequencies specified as "once", the above interval extension does not apply.

If a Completion Time requires periodic performance on a "once per..." basis, the above Frequency extension applies to each performance after the initial performance.

Exceptions to this Specification are stated in the individual Specifications.

SR 3.0.3 If it is discovered that a Surveillance was not performed within its specified Frequency, then compliance with the requirement to declare the LCO not met may be delayed, from the time of discovery, up to 24 hours or up to the limit of the specified Frequency, whichever is greater. This delay period is permitted to allow performance of the Surveillance. A risk evaluation shall be performed for any Surveillance delayed greater than 24 hours and the risk impact shall be managed.

If the Surveillance is not performed within the delay period, the LCO must immediately be declared not met, and the applicable Condition(s) must be entered.

When the Surveillance is performed within the delay period, and the Surveillance is not met, the LCO must immediately be declared not met, and the applicable Condition(s) must be entered.

(continued)

3.0 SR APPLICABILITY (continued)

SR 3.0.4 Entry into a MODE or other specified condition in the Applicability of a LCO shall not be made unless the LCO's Surveillances have been met within their specified Frequency. This provision shall not prevent entry into MODES or other specified conditions in the Applicability that are required to comply with ACTIONS or that are part of a shutdown of the unit.

SR 3.0.4 is only applicable for entry into a MODE or other specified condition in the Applicability in MODES 1, 2, 3, and 4.

3.1 REACTIVITY CONTROL SYSTEMS

3.1.1 SHUTDOWN MARGIN (SDM)

LC0 3.1.1 The SDM shall be within the limits specified in the COLR.

APPLICABILITY: MODE 2 with $k_{eff} < 1.0$,
MODES 3, 4, and 5.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. SDM not within limit.	A.1 Initiate boration to restore SDM to within limits.	15 minutes

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.1.1.1 Verify SDM to be within limits.	24 hours

3.1 REACTIVITY CONTROL SYSTEMS

3.1.2 Core Reactivity

LC0 3.1.2 The measured core reactivity shall be within $\pm 1\%$ $\Delta k/k$ of the normalized predicted values.

APPLICABILITY: MODES 1 and 2.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Measured core reactivity not within limit.	A.1 Re-evaluate core design and safety analysis, and determine that the reactor core is acceptable for continued operation.	7 days
	<u>AND</u> A.2 Establish appropriate operating restrictions and SRs.	7 days
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3.	6 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
<p>SR 3.1.2.1 -----NOTE----- The predicted reactivity values may be adjusted (normalized) to correspond to the measured core reactivity prior to exceeding a fuel burnup of 60 effective full power days (EFPD) after each fuel loading. -----</p> <p>Verify measured core reactivity is within $\pm 1\%$ $\Delta k/k$ of predicted values.</p>	<p>Prior to entering MODE 1 after each refueling</p> <p><u>AND</u></p> <p>-----NOTE----- Only required after 60 EFPD -----</p> <p>31 EFPD thereafter</p>

3.1 REACTIVITY CONTROL SYSTEMS

3.1.3 Moderator Temperature Coefficient (MTC)

LCO 3.1.3 The MTC shall be maintained within the limits specified in the COLR.

APPLICABILITY: MODE 1, and MODE 2 with $k_{eff} \geq 1.0$ for the upper MTC limit, MODES 1, 2, and 3 for the lower MTC limit.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. MTC not within upper limit.	A.1 Establish administrative withdrawal limits for control banks to maintain MTC within limit.	24 hours
B. Required Action and associated Completion Time of Condition A not met.	B.1 Be in MODE 2 with $k_{eff} < 1.0$.	6 hours
C. MTC not within lower limit.	C.1 Be in MODE 4.	12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.1.3.1 Verify MTC within upper limit.	Prior to entering MODE 1 after each refueling
<div> <div>SR 3.1.3.2</div> <div> <p>-----NOTES-----</p> <ol style="list-style-type: none"> 1. Not required to be performed until 7 effective full power days (EFPD) after reaching the equivalent of an equilibrium RTP all rods out (ARO) boron concentration of 300 ppm. 2. If the MTC is more negative than the 300 ppm Surveillance limit (not LCO limit) specified in the COLR, SR 3.1.3.2 shall be repeated once per 14 EFPD during the remainder of the fuel cycle. 3. SR 3.1.3.2 need not be repeated if the MTC measured at the equivalent of equilibrium RTP-ARO boron concentration of ≤ 60 ppm is less negative than the 60 ppm Surveillance limit specified in the COLR. <p>-----</p> <p>Verify MTC is within lower limit.</p> </div> </div>	Once each cycle

3.1 REACTIVITY CONTROL SYSTEMS

3.1.4 Rod Group Alignment Limits

LC0 3.1.4 All shutdown and control rods shall be OPERABLE. Individual indicated rod positions shall be within 12 steps of their group step counter demand position.

APPLICABILITY: MODES 1 and 2.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more rod(s) inoperable.	A.1.1 Verify SDM to be within the limits specified in the COLR.	1 hour
	<u>OR</u>	
	A.1.2 Initiate boration to restore SDM within limit.	1 hour
	<u>AND</u>	
	A.2 Be in MODE 3.	6 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. One rod not within alignment limits.	B.1 Restore rod, to within alignment limits.	8 hours with the On-Line Power Distribution Monitoring System (OPDMS) OPERABLE
	<u>OR</u>	<u>OR</u>
	B.2.1.1 Verify SDM to be within the limits specified in the COLR.	1 hour with the OPDMS inoperable
	<u>OR</u>	1 hour
	B.2.1.2 Initiate boration to restore SDM within limit.	1 hour
	<u>AND</u>	
	B.2.2 Reduce THERMAL POWER to $\leq 75\%$ RTP.	2 hours
	<u>AND</u>	
		(continued)

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. (continued)	<p>B.2.3 Verify SDM is within the limits specified in the COLR.</p> <p><u>AND</u></p> <p>-----NOTE----- Only required to be performed when OPDMS is inoperable. -----</p>	Once per 12 hours
	<p>B.2.4 Perform SR 3.2.1.1 ($F_Q(Z)$ verification) and SR 3.2.1.2 ($F_Q^N(Z)$ verification).</p> <p><u>AND</u></p> <p>-----NOTE----- Only required to be performed when OPDMS is inoperable. -----</p>	72 hours
	<p>B.2.5 Perform SR 3.2.2.1 ($F_{\Delta H}^N$ verification).</p> <p><u>AND</u></p>	72 hours
	<p>B.2.6 Re-evaluate safety analyses and confirm results remain valid for duration of operation under these conditions.</p>	5 days

C. Required Action and
associated Completion
Time for Condition B
not met.

C.1 Be in MODE 3.

6 hours

(continued)

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ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. More than one rod not within alignment limit.	D.1.1 Verify SDM is within the limits specified in the COLR.	1 hour
	<u>OR</u>	
	D.1.2 Initiate boration to restore required SDM to within limit.	1 hour
	<u>AND</u>	
	D.2 Be in MODE 3.	6 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.1.4.1 Verify individual rod positions within alignment limit.	12 hours

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.1.4.2 Verify rod freedom of movement (trippability) by moving each rod not fully inserted in the core ≥ 10 steps in either direction.</p>	<p>92 days</p>
<p>SR 3.1.4.3 Verify rod drop time of each rod, from the fully withdrawn position, is $\leq [2.47]$ seconds from the beginning of decay of stationary gripper coil voltage to dashpot entry, with:</p> <p>a. $T_{avg} \geq 500^{\circ}\text{F}$, and</p> <p>b. All reactor coolant pumps operating.</p>	<p>Prior to reactor criticality after each removal of the reactor head</p>

3.1 REACTIVITY CONTROL SYSTEMS

3.1.5 Shutdown Bank Insertion Limits

LCO 3.1.5 Each Shutdown Bank shall be within insertion limits specified in the COLR.

APPLICABILITY: MODES 1 and 2.

-----NOTE-----
This LCO is not applicable while performing SR 3.1.4.2.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more shutdown banks not within limits.	A.1.1 Verify SDM is within the limits specified in the COLR.	1 hour
	<u>OR</u>	
	A.1.2 Initiate boration to restore SDM to within limit.	1 hour
	<u>AND</u>	
	A.2 Restore shutdown banks to within limits.	2 hours
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3.	6 hours

SURVEILLANCE REQUIREMENTS

. SURVEILLANCE		FREQUENCY
SR 3.1.5.1	Verify each shutdown bank is within the insertion limits specified in the COLR.	12 hours

3.1 REACTIVITY CONTROL SYSTEMS

3.1.6 Control Bank Insertion Limits

LCO 3.1.6 Control banks shall be within the insertion, sequence, and overlap limits specified in the COLR.

APPLICABILITY: MODE 1,
MODE 2 with $k_{eff} \geq 1.0$.

-----NOTE-----
This LCO is not applicable while performing SR 3.1.4.2.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Control Bank insertion limits not met.	A.1.1 Verify SDM is within the limits specified in the COLR.	1 hour
	<u>OR</u>	
	A.1.2 Initiate boration to restore SDM to within limit.	1 hour
	<u>AND</u>	
	A.2 Restore control bank(s) to within limits.	2 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. Control bank sequence or overlap limits not met.	B.1.1 Verify SDM is within the limits specified in the COLR.	1 hour
	<u>OR</u>	
	B.1.2 Initiate boration to restore SDM to within limit.	1 hour
	<u>AND</u>	
	B.2 Restore control bank sequence and overlap to within limits.	2 hours
C. Required Action and associated Completion Time not met.	C.1 Be in MODE 2 with $k_{eff} < 1.0$.	6 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.1.6.1	Verify the estimated critical control bank position is within limits specified in the COLR.	Within 4 hours prior to achieving criticality
SR 3.1.6.2	Verify each control bank insertion is within the limits specified in the COLR.	12 hours
SR 3.1.6.3	Verify sequence and overlap limits, specified in the COLR, are met for control banks not fully withdrawn from the core.	12 hours

3.1 REACTIVITY CONTROL SYSTEMS

3.1.7 Rod Position Indication

LC0 3.1.7 The Digital Rod Position Indication (DRPI) System and the Bank Demand Position Indication System shall be OPERABLE.

APPLICABILITY: MODES 1 and 2.

ACTIONS

-----NOTE-----
Separate Condition entry is allowed for each inoperable rod position indicator and each demand position indicator.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One DRPI per group inoperable for one or more groups.	A.1 Verify the position of the rods with inoperable position indicators by using the On-line Power Distribution Monitoring System (OPDMS).	Once per 8 hours
	<u>OR</u> A.2 Reduce THERMAL POWER to $\leq 50\%$ RTP.	8 hours
B. More than one [D]RPI per group inoperable.	B.1 Place the control rods under manual control.	Immediately
	<u>AND</u> B.2 Monitor and Record RCS T_{avg} . <u>AND</u>	Once per 1 hour

	<p>B.3 Verify the position of the rods with inoperable position indicators indirectly by using the incore detectors.</p> <p><u>AND</u></p> <p>B.4 Restore inoperable position indicators to OPERABLE status such that a maximum of one [D]RPI per group is inoperable.</p>	<p>Once per 8 hours</p> <p>24 hours</p>
<p>C. One or more rods with inoperable position indicators have been moved in excess of 24 steps in one direction since the last determination of the rod's position.</p>	<p>C.1 Verify the position of the rods with inoperable position indicators by using the OPDMS.</p> <p><u>OR</u></p> <p>C.2 Reduce THERMAL POWER to $\leq 50\%$ RTP.</p>	<p>4 hours</p> <p>8 hours</p>

(continued)

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ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. One demand position indicator per bank inoperable for one or more banks.	D.1.1 Verify by administrative means all DRPIs for the affected banks are OPERABLE.	Once per 8 hours
	<u>AND</u>	
	D.1.2 Verify the most withdrawn rod and the least withdrawn rod of the affected banks are ≤ 12 steps apart.	Once per 8 hours
	<u>OR</u>	
	D.2 Reduce THERMAL POWER to $\leq 50\%$ RTP.	8 hours
E. Required Action and associated Completion Time not met.	E.1 Be in MODE 3.	6 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.1.7.1 Verify each DRPI agrees within 12 steps of the group demand position for the [full indicated range] of rod travel.	Prior to criticality after each removal of the reactor head

3.1 REACTIVITY CONTROL SYSTEMS

3.1.8 PHYSICS TESTS Exceptions – MODE 2

LCO 3.1.8 During the performance of PHYSICS TESTS, the requirements of

- LCO 3.1.3 "Moderator Temperature Coefficient,"
- LCO 3.1.4 "Rod Group Alignment Limits,"
- LCO 3.1.5 "Shutdown Bank Insertion Limit,"
- LCO 3.1.6 "Control Bank Insertion Limits," and
- LCO 3.4.2 "RCS Minimum Temperature for Criticality"

may be suspended, and the number of required channels for LCO 3.3.1, "RTS Instrumentation," Functions 2, 3, 6, and 18.c, may be reduced to 3 provided:

- a. RCS lowest loop average temperature is \geq [535°F],
- b. SDM is within the limits specified in the COLR, and
- c. THERMAL POWER is $<$ 5% RTP.

APPLICABILITY: During PHYSICS TESTS initiated in MODE 2.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. SDM not within limit.	A.1 Initiate boration to restore SDM to within limit.	15 minutes
	<u>AND</u> A.2 Suspend PHYSICS TEST exceptions.	1 hour
B. THERMAL POWER not within limit	B.1 Open reactor trip breakers.	Immediately

C. RCS lowest loop
average temperature
not within limit.

C.1 Restore RCS lowest loop
average temperature to
within limit.

15 minutes

(continued)

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ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. Required Action and Associated Completion Time of Condition C not met.	D.1 Be in MODE 3.	15 minutes

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.1.8.1	Perform a CHANNEL OPERATIONAL TEST on power range and intermediate range channels per SR 3.3.1.5.	Prior to initiation of PHYSICS TESTS
SR 3.1.8.2	Verify the RCS lowest loop average temperature is \geq [535] °F.	30 minutes
SR 3.1.8.3	Verify THERMAL POWER is $<$ 5% RTP.	30 minutes
SR 3.1.8.4	Verify SDM is within the limits specified in the COLR.	24 hours

3.1 REACTIVITY CONTROL

3.1.9 Chemical and Volume Control System (CVS) Demineralized Water Isolation Valves

LCO 3.1.9 Two CVS Demineralized Water Isolation Valves shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, 4, and 5.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One CVS demineralized water isolation valve inoperable.	A.1 Restore two CVS demineralized water isolation valves to OPERABLE status.	72 hours
<p>B. Required Action and associated Completion Time of Condition not met.</p> <p><u>OR</u></p> <p>Two CVS demineralized water isolation valves inoperable.</p>	<p>-----NOTE----- Flow path(s) may be unisolated intermittently under administrative controls. -----</p> <p>B.1 Isolate the flow path from the demineralized water storage tank to the Reactor Coolant System by use of at least one closed manual or one closed and de-activated automatic valve.</p>	1 hour

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.1.9.1	Verify two CVS demineralized water isolation valves are OPERABLE by stroking the valve closed.	In accordance with the Inservice Testing Program

3.2 POWER DISTRIBUTION LIMITS

3.2.1 Heat Flux Hot Channel Factor ($F_Q(Z)$) (F_Q Methodology)

LC0 3.2.1 $F_Q(Z)$, as approximated by $F_Q^C(Z)$ and $F_Q^W(Z)$, shall be within the limits specified in the COLR.

APPLICABILITY: MODE 1 with On-line Power Distribution Monitoring System (OPDMS) inoperable.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>-----NOTE----- Required Action A.4 shall be completed whenever this Condition is entered. -----</p>		
A. $F_Q^C(Z)$ not within limit.	<p>A.1 Reduce THERMAL POWER $\geq 1\%$ RTP for each $1\% F_Q^C(Z)$ exceeds limit.</p> <p><u>AND</u></p> <p>A.2 Reduce Power Range Neutron Flux High trip setpoints $\geq 1\%$ for each $1\% F_Q^C(Z)$ exceeds limit.</p> <p><u>AND</u></p> <p>A.3 Reduce Overpower ΔT trip setpoints $\geq 1\%$ for each $1\% F_Q^C(Z)$ exceeds limit.</p> <p><u>AND</u></p>	<p>15 minutes after each $F_Q^C(Z)$ determination</p> <p>72 hours after each $F_Q^C(Z)$ determination</p> <p>72 hours after each $F_Q^C(Z)$ determination</p>

A.4 Perform SR 3.2.1.1 and
SR 3.2.1.2.

Prior to
increasing
THERMAL POWER
above the limit
of Required
Action A.1

(continued)

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ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>-----NOTE----- Required Action B.4 shall be completed whenever this Condition is entered. -----</p>		
B. F _Q (Z) not within limits.	B.1 Reduce AFD limits $\geq 1\%$ for each 1% F _Q (Z) exceeds limit.	4 hours
	<u>AND</u>	
	B.2 Reduce Power Range Neutron Flux - High trip setpoints $\geq 1\%$ for each 1% that the maximum allowable power of the AFD limits is reduced.	72 hours
	<u>AND</u>	
	B.3 Reduce Overpower ΔT trip setpoints $\geq 1\%$ for each 1% that the maximum allowable power of the AFD limits is reduced.	72 hours
	<u>AND</u>	
	B.4 Perform SR 3.2.1.1 and SR 3.2.1.2.	Prior to increasing THERMAL POWER above the maximum allowable power of the AFD limits

C. Required Action and
associated
Completion Time
not met.

C.1 Be in MODE 2.

6 hours

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SURVEILLANCE REQUIREMENTS

-----NOTES-----

1. During power escalation at the beginning of each cycle, THERMAL POWER may be increased until a power level for extended operation has been achieved at which a power distribution map is obtained.
2. If the OPDMS becomes inoperable while in MODE 1 these surveillances must be performed within 31 days of the last verification of OPDMS parameters.

SURVEILLANCE	FREQUENCY
SR 3.2.1.1 Verify $F_Q(Z)$ within limit.	Once after each refueling prior to THERMAL POWER exceeding 75% RTP <u>AND</u> Once within 12 hours after achieving equilibrium conditions after exceeding, by $\geq 10\%$ RTP, the THERMAL POWER at which $F_Q(Z)$ was last verified <u>AND</u> 31 EFPD thereafter

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.2.1.2</p> <p style="text-align: center;">-----NOTE-----</p> <p>If F_Q^W(Z) measurements indicate maximum over z[F_Q^C(Z)] has increased since the previous evaluation of F_Q^C(Z):</p> <p>a. Increase F_Q^W(Z) by the appropriate factor and reverify F_Q^W(Z) is within limits; or</p> <p>b. Repeat SR 3.2.1.2 once per 7 EFPD until two successive flux maps indicate maximum over z[F_Q^C(Z)] has not increased.</p> <p>-----</p> <p>Verify F_Q^W(Z) within limits.</p>	<p>Once after each refueling prior to THERMAL POWER exceeding 75% RTP</p> <p><u>AND</u></p> <p>Once within 12 hours after achieving equilibrium conditions after exceeding, by ≥ 10% RTP, the THERMAL POWER at which F_Q^W(Z) was last verified</p> <p><u>AND</u></p> <p>31 EFPD thereafter</p>

3.2 POWER DISTRIBUTION LIMITS

3.2.2 Nuclear Enthalpy Rise Hot Channel Factor ($F_{\Delta H}^N$)

LCO 3.2.2 $F_{\Delta H}^N$ shall be within the limits specified in the COLR.

APPLICABILITY: MODE 1 with On-line Power Distribution Monitoring System (OPDMS) inoperable.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>A. -----NOTE----- Required Actions A.2 and A.3 must be completed whenever Condition A is entered. -----</p> <p>$F_{\Delta H}^N$ not within limit.</p>	<p>A.1.1 Restore $F_{\Delta H}^N$ to within limit.</p> <p><u>OR</u></p> <p>A.1.2.1 Reduce THERMAL POWER to < 50% RTP.</p> <p><u>AND</u></p> <p>A.1.2.2 Reduce Power Range Neutron Flux - High trip setpoints to $\leq 55\%$ RTP.</p> <p><u>AND</u></p>	<p>4 hours</p> <p>4 hours</p> <p>72 hours</p> <p>(continued)</p>

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. (continued)	A.2 Perform SR 3.2.2.1. <u>AND</u>	24 hours
	A.3 -----NOTE----- THERMAL POWER does not have to be reduced to comply with this Required Action. ----- Perform SR 3.2.2.1.	Prior to THERMAL POWER exceeding 50% RTP <u>AND</u> Prior to THERMAL POWER exceeding 75% RTP <u>AND</u> 24 hours after THERMAL POWER reaching ≥ 95% RTP
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 2.	6 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
<p>SR 3.2.2.1</p> <p>-----NOTE-----</p> <p>If the OPDMS becomes inoperable while in MODE 1 these Surveillances must be performed within 31 days of the last verification of OPDMS parameters.</p> <p>-----</p> <p>Verify $F_{\Delta H}^N$ within limits specified in the COLR.</p>	<p>Once after each refueling prior to THERMAL POWER exceeding 75% RTP</p> <p><u>AND</u></p> <p>31 EFPD thereafter</p>

3.2 POWER DISTRIBUTION LIMITS

3.2.3 AXIAL FLUX DIFFERENCE (AFD) (Relaxed Axial Offset Control
(RAOC) Methodology)

LC0 3.2.3 The AFD in %-flux-difference units shall be maintained within the limits specified in the COLR.

-----NOTE-----
The AFD shall be considered outside limits when two or more OPERABLE excore channels indicate AFD to be outside limits.

APPLICABILITY: MODE 1 with THERMAL POWER \geq 50% RTP and with the On-Line Power Distribution Monitoring System (OPDMS) inoperable.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. AFD not within limits.	A.1 Reduce THERMAL POWER to < 50% RTP.	30 minutes

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.2.3.1	Verify AFD within limits for each OPERABLE excore channel.	7 days

3.2 POWER DISTRIBUTION LIMITS

3.2.4 QUADRANT POWER TILT RATIO (QPTR)

LCO 3.2.4 The QPTR shall be ≤ 1.02 .

APPLICABILITY: MODE 1 with THERMAL POWER > 50% RTP and with the OPDMS inoperable.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. QPTR not within limit.	A.1 Reduce THERMAL POWER $\geq 3\%$ from RTP for each 1% of QPTR > 1.00.	2 hours after each QPTR determination
	<u>AND</u>	
	A.2 Perform SR 3.2.4.1.	Once per 12 hours
	<u>AND</u>	
	A.3 Perform SR 3.2.1.1 and SR 3.2.2.1.	24 hours after achieving equilibrium conditions from a THERMAL POWER reduction per Required Action A.1
	<u>AND</u>	Once per 7 days thereafter (continued)

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. (continued)	A.4 Reevaluate safety analyses and confirm results remain valid for duration of operation under this condition.	Prior to increasing THERMAL POWER above the limit of Required Action A.1
	<u>AND</u>	
	A.5 -----NOTE----- 1. Perform Required Action A.5 only after Required Action A.4 is completed. 2. Required Action A.6 shall be completed whenever Required Action A.5 is performed. -----	Prior to increasing THERMAL POWER above the limit of Required Action A.1
	Normalize excore detectors to restore QPTR to within limit. <u>AND</u>	
	A.6 -----NOTE----- Perform Required Action A.6 only after Required Action A.5 is completed. ----- Perform SR 3.2.1.1, SR 3.2.1.2, and SR 3.2.2.1.	Within 24 hours after achieving equilibrium conditions at RTP not to exceed 48 hours

after increasing
THERMAL POWER
above the limit
of Required
Action A.1

(continued)

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ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. Required Action and associated Completion Time not met.	B.1 Reduce THERMAL POWER to $\leq 50\%$ RTP.	4 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
<p>SR 3.2.4.1</p> <p>-----NOTES-----</p> <p>1. With one power range channel inoperable and THERMAL POWER $< 75\%$ RTP, the remaining three power range channels can be used for calculating QPTR.</p> <p>2. SR 3.2.4.2 may be performed in lieu of this Surveillance.</p> <p>-----</p> <p>Verify QPTR within limit by calculation.</p>	7 days

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.2.4.2</p> <p>-----NOTE----- Not required to be performed until 12 hours after input from one or more Power Range Neutron Flux channels are inoperable with THERMAL POWER $\geq 75\%$ RTP. -----</p> <p>Verify QPTR is within limit using a minimum of 4 symmetric pairs of fixed incore detectors.</p>	<p>12 hours</p>

3.2 POWER DISTRIBUTION LIMITS

3.2.5 OPDMS-Monitored Power Distribution Parameters

LC0 3.2.5 The following parameters shall not exceed their operating limits as specified in the COLR:

- a. Peak kw/ft(Z)
- b. $F_{\Delta H}^N$
- c. DNBR.

APPLICABILITY: MODE 1 with THERMAL POWER > 50% RTP with OPDMS OPERABLE.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more of the parameters a. through c. above not within limits.	A.1 Restore all parameters to within limits.	1 hour
B. Required Action and associated Completion Time not met.	<p>B.1 -----NOTE----- If the power distribution parameters are restored to within their limits while power is being reduced, operation may continue at the power level where this occurs. -----</p> <p>Reduce THERMAL POWER to < 50% RTP.</p>	4 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.2.5.1	Verify the parameters a. through c. to be within their limits.	24 hours with OPDMS alarms OPERABLE <u>OR</u> 12 hours with OPDMS alarms inoperable

3.3 INSTRUMENTATION

3.3.1 Reactor Trip System (RTS) Instrumentation

LC0 3.3.1 The RTS instrumentation for each Function in Table 3.3.1-1 shall be OPERABLE.

APPLICABILITY: According to Table 3.3.1-1.

ACTIONS

-----NOTE-----
Separate Condition entry is allowed for each Function.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more Functions with one or more required channels inoperable.	A.1 Enter the Condition referenced in Table 3.3.1-1 for the channel(s).	Immediately
B. One manual initiation device inoperable.	B.1 Restore manual initiation device to OPERABLE status.	48 hours
	<u>OR</u>	
	B.2.1 Be in MODE 3.	54 hours
	<u>AND</u>	
	B.2.2 Open reactor trip breakers (RTBs).	55 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. One manual initiation device inoperable.	C.1 Restore manual initiation device to OPERABLE status.	48 hours
	<u>OR</u> C.2 Open RTBs.	49 hours
D. One or two Power Range Neutron Flux-High channels inoperable.	D.1.1 Reduce THERMAL POWER to $\leq 75\%$ RTP.	12 hours
	<u>AND</u>	
	D.1.2 Place one inoperable channel in bypass or trip.	6 hours
	<u>AND</u>	
	D.1.3 With two inoperable channels, place one channel in bypass and one channel in trip.	6 hours
	<u>OR</u>	
	D.2.1 Place inoperable channel(s) in bypass.	6 hours
	<u>AND</u>	
	-----NOTE----- Only required to be performed when OPDMS is inoperable and the Power Range Neutron Flux input to QPTR is inoperable. -----	
	D.2.2 Perform SR 3.2.4.2 (QPTR verification).	Once per 12 hours
	<u>OR</u>	
	D.3 Be in MODE 3.	12 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
E. One or two channels inoperable.	E.1.1 Place one inoperable channel in bypass or trip.	6 hours
	<u>AND</u>	
	E.1.2 With two channels inoperable, place one channel in bypass and one channel in trip.	6 hours
	<u>OR</u>	
	E.2 Be in MODE 3.	12 hours
F. THERMAL POWER between P-6 and P-10, one or two Intermediate Range Neutron Flux channels inoperable.	F.1.1 Place one inoperable channel in bypass or trip.	2 hours
	<u>AND</u>	
	F.1.2 With two channels inoperable, place one channel in bypass and one channel in trip.	2 hours
	<u>OR</u>	
	F.2 Reduce THERMAL POWER to < P-6.	2 hours
	<u>OR</u>	
	F.3 Increase THERMAL POWER to > P-10.	2 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
G. THERMAL POWER between P-6 and P-10, three Intermediate Range Neutron Flux channels inoperable.	G.1 Suspend operations involving positive reactivity additions.	Immediately
	<u>AND</u> G.2 Reduce THERMAL POWER to < P-6.	2 hours
H. THERMAL POWER < P-6, one or two Intermediate Range Neutron Flux channels inoperable.	H.1 Restore three of four channels to OPERABLE status.	Prior to increasing THERMAL POWER to > P-6
I. One or two Source Range Neutron Flux channels inoperable.	I.1 Suspend operations involving positive reactivity additions.	Immediately
J. Three Source Range Neutron Flux channels inoperable.	J.1 Open RTBs.	Immediately

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
K. One or two channels inoperable.	K.1.1 Place one inoperable channel in bypass or trip.	6 hours
	<u>AND</u>	
	K.1.2 With two channels inoperable, place one channel in bypass and one channel in trip.	6 hours
L. One or two channels inoperable.	<u>OR</u>	
	K.2 Reduce THERMAL POWER to < P-10.	12 hours
L. One or two channels inoperable.	L.1.1 Place one inoperable channel in bypass or trip.	6 hours
	<u>AND</u>	
	L.1.2 With two channels inoperable, place one channel in bypass and one channel in trip.	6 hours
M. One or two channels/divisions inoperable.	<u>OR</u>	
	L.2 Reduce THERMAL POWER to < P-8.	10 hours
M. One or two channels/divisions inoperable.	M.1 Restore three of four channels/divisions to OPERABLE status.	6 hours
	<u>OR</u>	
	M.2 Be in MODE 3.	12 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
N. One or two interlock channels inoperable.	N.1 Verify the interlocks are in required state for existing plant conditions.	1 hour
	<u>OR</u>	
	N.2.1 Place the Functions associated with one inoperable interlock channel in bypass or trip.	7 hours
	<u>AND</u>	
	N.2.2 With two interlock channels inoperable, place the Functions associated with one inoperable interlock channel in bypass and with one inoperable interlock channel in trip.	7 hours
	<u>OR</u>	
	N.3 Be in MODE 3.	13 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
0. One or two interlock channels inoperable.	0.1 Verify the interlocks are in required state for existing plant conditions.	1 hour
	<u>OR</u>	
	0.2.1 Place the Functions associated with one inoperable interlock channel in bypass.	7 hours
	<u>AND</u>	
	0.2.2 With two interlock channels inoperable, place the Functions associated with one inoperable interlock channel in bypass and with one inoperable interlock channel in trip.	7 hours
P. One division inoperable.	<u>OR</u>	
	0.3 Be in MODE 2.	13 hours
	P.1 Open RTBs in inoperable division.	8 hours
	<u>OR</u>	
	P.2.1 Be in MODE 3, 4, or 5.	14 hours
	<u>AND</u>	
	P.2.2 Open RTBs.	14 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
Q. Two divisions inoperable.	Q.1 Restore three of four divisions to OPERABLE status.	1 hour
	<u>OR</u>	
	Q.2.1 Be in MODE 3, 4, or 5.	7 hours
	<u>AND</u>	
	Q.2.2 Open RTBs.	7 hours
R. One or two channels/divisions inoperable.	R.1 Restore three of four channels/divisions to OPERABLE status.	48 hours
	<u>OR</u>	
	R.2 Open RTBs.	49 hours
S. One or two Source Range Neutron Flux channel inoperable.	S.1 Restore three of four channels to OPERABLE status.	48 hours
	<u>OR</u>	
	S.2 Open RTBs.	49 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
T. Required Source Range Neutron Flux channel inoperable.	T.1 Suspend operations involving positive reactivity additions.	Immediately
	<u>AND</u>	
	T.2 Close unborated water source isolation valves.	1 hour
	<u>AND</u>	
	T.3 Perform SR 3.1.1.1.	1 hour
		<u>AND</u>
		Once per 12 hours thereafter

SURVEILLANCE REQUIREMENTS

-----NOTE-----
Refer to Table 3.3.1-1 to determine which SRs apply for each RTS Function.

SURVEILLANCE		FREQUENCY
SR 3.3.1.1	Perform CHANNEL CHECK.	12 hours
SR 3.3.1.2	<p>-----NOTES-----</p> <ol style="list-style-type: none"> 1. Adjust nuclear instrument channel in the Protection and Safety Monitoring System (PMS) if absolute difference is > 2% RTP. 2. Required to be met within 12 hours after reaching 15% RTP. 3. If the calorimetric heat balance is < 70% RTP, and if the nuclear instrumentation channel indicated power is: <ol style="list-style-type: none"> a. lower than the calorimetric measurement by > 2%, then adjust the nuclear instrumentation channel upward to match the calorimetric measurement. b. higher than the calorimetric measurement, then no adjustment is required. <p>-----</p> <p>Compare results of calorimetric heat balance to nuclear instrument channel output.</p>	24 hours

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.3.1.3 -----NOTES-----</p> <ol style="list-style-type: none"> 1. Adjust nuclear instrument channel in PMS if absolute difference is $\geq 3\%$ AFD. 2. Required to be met within 24 hours after reaching 20% RTP. <p>-----</p> <p>Compare results of the incore detector measurements to nuclear instrument channel AXIAL FLUX DIFFERENCE.</p>	<p>31 effective full power days (EFPD)</p>
<p>SR 3.3.1.4 -----NOTE-----</p> <p>Required to be met within 24 hours after reaching 50% RTP.</p> <p>-----</p> <p>Calibrate excore channels to agree with incore detector measurements.</p>	<p>92 EFPD</p>
<p>SR 3.3.1.5 -----NOTE-----</p> <p>This Surveillance must be performed on both reactor trip breakers associated with a single division.</p> <p>-----</p> <p>Perform TADOT.</p>	<p>92 days on a STAGGERED TEST BASIS</p>
<p>SR 3.3.1.6 -----NOTES-----</p> <p>Not required to be performed for source range instrumentation prior to entering MODE 3 from MODE 2 until 4 hours after entry into MODE 3.</p> <p>-----</p> <p>Perform RTCOT.</p>	<p>92 days</p>

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.3.1.7</p> <p>-----NOTE----- This Surveillance shall include verification that interlocks P-6 and P-10 are in their required state for existing unit conditions. -----</p> <p>Perform RTCOT.</p>	<p>-----NOTE----- Only required when not performed within previous 92 days -----</p> <p>Prior to reactor startup</p> <p><u>AND</u></p> <p>Four hours after reducing power below P-10 for power and intermediate instrumentation</p> <p><u>AND</u></p> <p>Four hours after reducing power below P-6 for source range instrumentation</p> <p><u>AND</u></p> <p>Every 92 days thereafter</p>

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.3.1.8 -----NOTE----- This Surveillance shall include verification that the time constants are adjusted to the prescribed values. -----</p> <p>Perform CHANNEL CALIBRATION.</p>	<p>24 months</p>
<p>SR 3.3.1.9 -----NOTE----- Neutron detectors are excluded from CHANNEL CALIBRATION. -----</p> <p>Perform CHANNEL CALIBRATION.</p>	<p>24 months</p>
<p>SR 3.3.1.10 -----NOTE----- Verification of setpoint is not required. -----</p> <p>Perform TADOT.</p>	<p>24 months</p>
<p>SR 3.3.1.11 -----NOTE----- Neutron detectors are excluded from response time testing. -----</p> <p>Verify RTS RESPONSE TIME is within limits.</p>	<p>24 months on a STAGGERED TEST BASIS</p>

Table 3.3.1-1 (page 1 of 5)
Reactor Trip System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
1. Manual Reactor Trip	1,2	2	B	SR 3.3.1.10		N/A
	3(a),4(a),5(a)	2	C	SR 3.3.1.10		N/A
2. Power Range Neutron Flux						
a. High Setpoint	1,2	4	D	SR 3.3.1.1 SR 3.3.1.2 SR 3.3.1.6 SR 3.3.1.9 SR 3.3.1.11		[≤ 118% RTP]
b. Low Setpoint	1 ^(b) ,2	4	E	SR 3.3.1.1 SR 3.3.1.7 SR 3.3.1.9 SR 3.3.1.11		[≤ 35% RTP]
3. Power Range Neutron Flux High Positive Rate	1,2	4	E	SR 3.3.1.6 SR 3.3.1.9 SR 3.3.1.11		[≤ 5.0% RTP with time constant ≥ 2 sec*]
4. Intermediate Range Neutron Flux	1 ^(b) ,2 ^(c)	4	F,G	SR 3.3.1.1 SR 3.3.1.7 SR 3.3.1.9		[≤ 25% RTP*]
	2 ^(d)	4	H	SR 3.3.1.1 SR 3.3.1.7 SR 3.3.1.9		[≤ 25% RTP*]
5. Source Range Neutron Flux High Setpoint	2 ^(d)	4	I,J	SR 3.3.1.1 SR 3.3.1.7 SR 3.3.1.9 SR 3.3.1.11		[≤ 1.0E5 cps*]
	3(a),4(a),5(a)	4	J,S	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.9 SR 3.3.1.11		[≤ 1.0E5 cps*]
	3(e),4(e),5(e)	1	T	SR 3.3.1.1 SR 3.3.1.9		N/A

(continued)

(a) With Reactor Trip Breakers (RTBs) closed and Plant Control System capable of rod withdrawal.

(b) Below the P-10 (Power Range Neutron Flux) interlocks.

(c) Above the P-6 (Intermediate Range Neutron Flux) interlocks.

(d) Below the P-6 (Intermediate Range Neutron Flux) interlocks.

(e) With RTBs open. In this condition, Source Range Function does not provide reactor trip but does provide indication.

[Reviewer Note: The values specified in brackets in the Trip Setpoint column are the Chapter 15 safety analysis values and are included for reviewer information only.]

The values specified in brackets followed by " * " in the Trip Setpoint column are typical values for the Function. No credit was assumed for these Functions (typically diverse trips/actuators) in the Chapter 15 safety analyses and no safety analysis value is available.

In all cases, the values specified in brackets must be replaced, following the plant-specific setpoint study, with the actual Trip Setpoints. Upon selection of the plant specific instrumentation, the Trip Setpoints will be calculated in accordance with the setpoint methodology described in WCAP-14606. (WCAP-14606 is an AP600 document that describes a methodology that is applicable to AP1000. AP1000 has some slight differences in instrument spans as a result of the higher power level.) Allowable Values will be calculated in accordance with the setpoint methodology and specified in the Allowable Value column. The plant specific setpoint calculations will reflect the latest licensing analysis/design basis and may incorporate NRC accepted improvements in setpoint methodology.]

Table 3.3.1-1 (page 2 of 5)
Reactor Trip System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
6. Overtemperature ΔT	1,2	4	E	SR 3.3.1.1 SR 3.3.1.3 SR 3.3.1.4 SR 3.3.1.6 SR 3.3.1.8 SR 3.3.1.11	Refer to Note 1 (Page 3.3-18)	Refer to Note 1 (Page 3.3-18)
7. Overpower ΔT	1,2	4	E	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.8 SR 3.3.1.11	Refer to Note 2 (Page 3.3-18)	Refer to Note 2 (Page 3.3-18)
8. Pressurizer Pressure						
a. Low Setpoint	1 ^(f)	4	K	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.8 SR 3.3.1.11		[\geq 1785 psig]
b. High Setpoint	1,2	4	E	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.8 SR 3.3.1.11		[\leq 2445 psig]
9. Pressurizer Water Level - High 3	1 ^(f)	4	K	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.8		[\leq 80%*]
10. Reactor Coolant Flow - Low						
a. Single Cold Leg	1 ^(g)	4 per cold leg	L	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.8 SR 3.3.1.11		[\geq 87% ⁽ⁱ⁾]
b. Two Cold Legs	1 ^(h)	4 per cold leg	K	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.8 SR 3.3.1.11		[\geq 87% ⁽ⁱ⁾]

(continued)

(f) Above the P-10 (Power Range Neutron Flux) interlock.

(g) Above the P-8 (Power Range Neutron Flux) interlock.

(h) Above the P-10 (Power Range Neutron Flux) interlock and below the P-8 (Power Range Neutron Flux) interlock.

(i) Percent of thermal design flow.

Table 3.3.1-1 (page 3 of 5)
Reactor Trip System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
11. Reactor Coolant Pump (RCP) Bearing Water Temperature - High						
a. Single Pump	1 ^(g)	4 per RCP	L	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.8		[≥ 320°F*]
b. Two Pumps	1 ^(h)	4 per RCP	K	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.8		[≥ 320°F*]
12. RCP Speed - Low	1 ^(f)	4	K	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.8		[≥ 90%]
13. Steam Generator (SG) Narrow Range Water Level - Low	1,2	4 per SG	E	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.8 SR 3.3.1.11		[≥ 95000 1bm]
14. Steam Generator (SG) Narrow Range Water Level - High 2	1,2 ^(k)	4 per SG	E	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.8 SR 3.3.1.11		[≤ 100%]
15. Safeguards Actuation Input from Engineered Safety Feature Actuation System						
a. Manual	1, 2	2	B	SR 3.3.1.10		N/A
b. Automatic	1, 2	4	M	SR 3.3.1.6		N/A

(continued)

(f) Above the P-10 (Power Range Neutron Flux) interlock.

(g) Above the P-8 (Power Range Neutron Flux) interlock.

(h) Above the P-10 (Power Range Neutron Flux) interlock and below the P-8 (Power Range Neutron Flux) interlock.

(k) Above the P-11 (Pressurizer Pressure) interlock.

Table 3.3.1-1 (page 4 of 5)
Reactor Trip System Instrumentation

FUNCTION	APPLICABLE MODES OR SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
16. Reactor Trip System Interlocks						
a. Intermediate Range Neutron Flux, P-6	2	4	N	SR 3.3.1.6 SR 3.3.1.9		[$\geq 1E-10$ amps]
b. Power Range Neutron Flux, P-8	1	4	O	SR 3.3.1.6 SR 3.3.1.9		[$\leq 48\%$ RTP*]
c. Power Range Neutron Flux, P-10	1,2	4	N	SR 3.3.1.6 SR 3.3.1.9		[10% RTP]
d. Pressurizer Pressure, P-11	1,2	4	N	SR 3.3.1.6 SR 3.3.1.9		[≤ 1970 psig]
17. Reactor Trip Breakers	1,2 3(j),4(j),5(j)	4 divisions with 2 RTBs per division	P,Q	SR 3.3.1.5		N/A
18. Reactor Trip Breaker (RTB) Undervoltage and Shunt Trip Mechanisms	1,2 3(j),4(j),5(j)	1 each per RTB mechanism	P,Q	SR 3.3.1.5		N/A
19. Automatic Trip Logic	1,2 3(j),4(j),5(j)	4 divisions 4 divisions	M R	SR 3.3.1.6 SR 3.3.1.6		N/A N/A
20. ADS Stages 1, 2, and 3 Actuation input from engineered safety feature actuation system						
a. Manual	1,2 3(j),4(j),5(j)	2 switch sets 2 switch sets	B B	SR 3.3.1.10 SR 3.3.1.10		N/A N/A
b. Automatic	1,2 3(j),4(j),5(j)	4 4	M R	SR 3.3.1.6 SR 3.3.1.6		N/A N/A
21. Core Makeup Tank Actuation input from engineered safety feature actuation system						
a. Manual	1,2 3(j),4(j),5(j)	2 switch sets 2 switch sets	B B	SR 3.3.1.10 SR 3.3.1.10		N/A N/A
b. Automatic	1,2 3(j),4(j),5(j)	4 4	M R	SR 3.3.1.6 SR 3.3.1.6		N/A N/A

(j) With Reactor Trip Breakers closed and Plant Control System capable of rod withdrawal.

Table 3.3.1-1 (page 5 of 5)
Reactor Trip System Instrumentation

Note 1: Overtemperature ΔT

The Overtemperature ΔT Function Allowable Value shall not exceed the following nominal Trip Setpoint by more than [TBD]% of ΔT span.

$$\Delta T \frac{(1+\tau_4 s)}{(1+\tau_5 s)} \leq \Delta T_0 \left\{ K_1 - K_2 \frac{(1+\tau_1 s)}{(1+\tau_2 s)} [T - T'] + K_3 (P - P') - f_1(\Delta I) \right\}$$

Where: ΔT is measured RCS ΔT , °F.
 ΔT_0 is the indicated ΔT at RTP, °F.
 s is the Laplace transform operator, sec⁻¹.
 T is the measured RCS average temperature, °F.
 T' is the indicated T_{avg} at RTP, \leq [°F].

P is the measured pressurizer pressure, psig.
 P' is the nominal RCS operating pressure, 2235 psig.
 $K_1 \leq$ [°F] $K_2 \geq$ [°F]/°F $K_3 \geq$ [°F]/psig
 $\tau_1 \geq$ [°F] sec $\tau_2 \leq$ [°F] sec
 $\tau_4 \geq$ [°F] sec $\tau_5 \leq$ [°F] sec

$$f_1(\Delta I) = \begin{cases} \frac{[^\circ\text{F}]}{0\% \text{ of RTP}} \{ [^\circ\text{F}] + (q_c - q_b) \} & \text{when } q_c - q_b \leq -[^\circ\text{F}] \% \text{ RTP} \\ -[^\circ\text{F}] \{ (q_c - q_b) - [^\circ\text{F}] \} & \text{when } -[^\circ\text{F}] \% \text{ RTP} < q_c - q_b \leq [^\circ\text{F}] \% \text{ RTP} \\ & \text{when } q_c - q_b > [^\circ\text{F}] \% \text{ RTP} \end{cases}$$

where q_c and q_b are percent RTP in the upper and lower halves of the core respectively, and $q_c + q_b$ is the total THERMAL POWER in percent RTP.

*These values denoted with [°F] are specified in the COLR.

Note 2: Overpower ΔT

The Overpower ΔT Function Allowable Value shall not exceed the following nominal Trip Setpoint by more than [TBD]% of ΔT span.

$$\Delta T \frac{(1+\tau_4 s)}{(1+\tau_5 s)} \leq \Delta T_0 \left\{ K_4 - K_5 \frac{\tau_3 s}{1+\tau_3 s} T - K_6 [T - T'] - f_2(\Delta I) \right\}$$

Where: ΔT is measured RCS ΔT , °F.
 ΔT_0 is the indicated ΔT at RTP, °F.
 s is the Laplace transform operator, sec⁻¹.
 T is the measured RCS average temperature, °F.
 T' is the nominal T_{avg} at RTP, \leq [°F].
 $K_4 \leq$ [°F] $K_5 \geq$ [°F]/°F for increasing T_{avg} $K_6 \geq$ [°F]/°F when $T > T'$
 $\tau_3 \geq$ [°F] sec $\tau_4 \geq$ [°F] sec $\tau_5 \leq$ [°F] sec when $T \leq T'$
 $f_2(\Delta I) =$ [°F]

*These values denoted with [°F] are specified in the COLR.

3.3 INSTRUMENTATION

3.3.2 Engineered Safety Feature Actuation System (ESFAS) Instrumentation

LCO 3.3.2 The ESFAS instrumentation for each function in Table 3.3.2-1 shall be OPERABLE.

APPLICABILITY: According to Table 3.3.2-1.

ACTIONS

- NOTES-----
1. Separate condition entry is allowed for each Function.
 2. The Conditions for each Function are given in Table 3.3.2-1. If the Required Actions and associated Completion Times of the first Condition are not met, refer to the second Condition.
-

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more Functions with one or more required channels or divisions inoperable.	A.1 Enter the Condition referenced in Table 3.3.2-1 for the channel(s) or division(s).	Immediately
B. One or two channels or divisions inoperable.	B.1 Place one inoperable channel or division in bypass or trip.	6 hours
	<u>AND</u> B.2 With two inoperable channels or divisions, place one inoperable channel or division in bypass and one inoperable channel or division in trip.	6 hours

C. One channel
inoperable.

C.1 Place inoperable
channel in bypass.

6 hours

(continued)

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ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. One required division inoperable.	D.1 Restore required division to OPERABLE status.	6 hours
E. One switch or switch set inoperable.	E.1 Restore switch and switch set to OPERABLE status.	48 hours
F. One channel inoperable	F.1 Restore channel to OPERABLE status.	72 hours
	<u>OR</u> F.2.1 Verify alternate radiation monitors are OPERABLE.	72 hours
	<u>AND</u> F.2.2 Verify control room isolation and air supply initiation manual controls are OPERABLE.	72 hours
G. One switch, switch set, channel, or division inoperable.	G.1 Restore switch, switch set, channel, and division to OPERABLE status.	72 hours
H. One channel inoperable.	H.1 Place channel in trip.	6 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
I. One or two channels inoperable.	I.1 Place one inoperable channel in bypass or trip.	6 hours
	<u>AND</u> I.2 With two inoperable channels, place one channel in bypass and one channel in trip.	6 hours
J. One or two interlock channels inoperable.	J.1 Verify the interlocks are in the required state for the existing plant conditions.	1 hour
	<u>OR</u> J.2.1 Place the Functions associated with one inoperable interlock channel in bypass or trip.	7 hours
	<u>AND</u> J.2.2 With two interlock channels inoperable, place the Functions associated with one inoperable interlock channel in bypass and with one inoperable interlock channel in trip.	7 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
K. Required Action and associated Completion Time not met.	K.1 -----NOTE----- LCO 3.0.8 is not applicable. ----- Suspend movement of irradiated fuel assemblies.	Immediately
L. Required Action and associated Completion Time not met.	L.1 Be in MODE 3.	6 hours
M. Required Action and associated Completion Time not met.	M.1 Be in MODE 3. <u>AND</u> M.2 Be in MODE 4.	6 hours 12 hours
N. Required Action and associated Completion Time not met.	N.1 Be in MODE 3. <u>AND</u> N.2 Be in MODE 4 with the RCS cooling provided by the RNS.	6 hours 24 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
O. Required Action and associated Completion Time not met.	0.1 Be in MODE 3.	6 hours
	<u>AND</u> 0.2 Be in MODE 5.	36 hours
P. Required Action and associated Completion Time not met.	P.1 -----NOTE----- Flow path(s) may be unisolated intermittently under administrative controls. ----- Isolate the affected flow path(s).	24 hours
	<u>AND</u> P.2.1 Isolate the affected flow path(s) by use of at least one closed and deactivated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.	7 days
	<u>OR</u> P.2.2 Verify the affected flow path is isolated.	Once per 7 days

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
Q. Required Action and associated Completion Time not met.	Q.1 -----NOTE----- Flow path(s) may be unisolated intermittently under administrative controls. -----	
	Isolate the affected flow path(s) by use of at least one closed manual or closed and de-activated automatic valve.	6 hours
	<u>OR</u> Q.2.1 Be in MODE 3.	12 hours
	<u>AND</u> Q.2.2 Be in MODE 4.	18 hours
R. Required Action and associated Completion Time not met.	R.1 Be in MODE 3.	6 hours
	<u>AND</u> R.2.1.1 -----NOTE----- Flow path(s) may be unisolated intermittently under administrative controls. ----- Isolate the affected flow path(s). <u>AND</u>	12 hours
	R.2.1.2 Verify the affected flow path is isolated.	Once per 7 days
		(continued)

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
R. (continued)	<p><u>OR</u></p> <p>R.2.2 Be in MODE 4 with the RCS cooling provided by the RNS.</p>	30 hours
S. Required Action and associated Completion Time not met.	<p>S.1 Be in MODE 3.</p> <p><u>AND</u></p> <p>S.2.1.1 Be in MODE 4 with the RCS cooling provided by the RNS.</p> <p><u>AND</u></p> <p>S.2.1.2 -----NOTE----- Flow path(s) may be unisolated intermittently under administrative controls. -----</p> <p>Isolate the affected flow path(s).</p> <p><u>AND</u></p> <p>S.2.1.3 Verify the affected flow path is isolated.</p> <p><u>OR</u></p> <p>S.2.2 Be in MODE 5.</p>	<p>6 hours</p> <p>24 hours</p> <p>30 hours</p> <p>Once per 7 days</p> <p>42 hours</p>

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
T. Required Action and associated Completion Time not met.	T.1.1 -----NOTE----- Flow path(s) may be unisolated intermittently under administrative controls. ----- Isolate the affected flow path(s).	6 hours
	<u>AND</u>	
	T.1.2.1 Isolate the affected flow path(s) by use of at least one closed and deactivated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.	7 days
	<u>OR</u>	
	T.1.2.2 Verify the affected flow path is isolated.	Once per 7 days
	<u>OR</u>	
	T.2.1 Be in MODE 3.	12 hours
	<u>AND</u>	
	T.2.2 Be in MODE 5.	42 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
U. Required Action and associated Completion Time not met.	U.1 Be in MODE 5.	12 hours
	<u>AND</u> U.2 Initiate action to open the RCS pressure boundary and establish a pressurizer level $\geq 20\%$.	12 hours
V. Required Action and associated Completion Time not met.	V.1 Restore the inoperable channel(s).	168 hours
	<u>OR</u> V.2.1 Be in MODE 5.	180 hours
	<u>AND</u> V.2.2 Initiate action to open the RCS pressure boundary and establish a pressurizer level $\geq 20\%$.	180 hours
W. Required Action and associated Completion Time not met.	W.1 If in MODE 5 with the RCS open and $< 20\%$ pressurizer level, initiate action to be MODE 5 with the RCS pressure boundary open and $\geq 20\%$ pressurizer level.	Immediately
	<u>AND</u> W.2 If in MODE 5, isolate the flow path from the demineralized water storage tank to the RCS by use of at least one closed and de-activated automatic valve or closed manual valve.	Immediately
	<u>AND</u>	(continued)

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
W. (continued)	W.3 If in MODE 6, initiate action to be in MODE 6 with the water level ≥ 23 feet above the top of the reactor vessel flange.	Immediately
	<u>AND</u> W.4 Suspend positive reactivity additions.	Immediately
X. Required Action and associated Completion Time not met.	X.1 If in MODE 5 with RCS open and $< 20\%$ pressurizer level, initiate action to be in MODE 5 with RCS open and $\geq 20\%$ pressurizer level.	Immediately
	<u>AND</u> X.2 If in MODE 6 with upper internals in place, initiate action to be in MODE 6 with the upper internals removed.	Immediately
	<u>AND</u> X.3 Suspend positive reactivity additions.	Immediately

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
Y. Required Action and associated Completion Time not met.	Y.1 Suspend positive reactivity additions.	Immediately
	<u>AND</u>	
	Y.2 If in MODE 4, be in MODE 5.	12 hours
	<u>AND</u>	
	Y.3 If in MODE 4 or 5, initiate action to establish a pressurizer level $\geq 20\%$ with the RCS pressure boundary intact.	12 hours
	<u>AND</u>	
	Y.4 If in MODE 6, initiate action to be in MODE 6 with the water level ≥ 23 feet above the top of the reactor vessel flange.	Immediately
Z. Required Action and associated Completion Time not met.	Z.1 -----NOTE----- Flow path(s) may be unisolated intermittently under administrative controls. -----	(continued)

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
Z. (continued)	Isolate the affected flow path(s) by use of at least one closed manual or closed and deactivated automatic valve.	6 hours
	<u>OR</u> Z.2.1 Be in MODE 3.	12 hours
	<u>AND</u> Z.2.2 Be in MODE 4 with the RCS cooling provided by the RNS.	30 hours
AA. Required Action and associated Completion Time not met.	AA.1.1 -----NOTE----- Flow path(s) may be unisolated intermittently under administrative controls. ----- Isolate the affected flow path(s).	24 hours
	<u>AND</u> AA.1.2.1 Isolate the affected flow path(s) by use of at least one closed and deactivated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.	7 days
	<u>OR</u> AA.1.2.2 Verify the affected flow path is isolated.	Once per 7 days
	<u>OR</u>	
		(continued)

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
AA. (continued)	AA.2.1 If in MODE 4, be in MODE 5. <u>AND</u>	12 hours
	AA.2.2 If in MODE 4 or 5, initiate action to establish a pressurizer level $\geq 20\%$. <u>AND</u>	12 hours
	AA.2.3 If in MODE 6, initiate action to be in MODE 6 with the water level ≥ 23 feet above the top of the reactor vessel flange.	Immediately
BB. One channel inoperable.	BB.1 Place channel in bypass. <u>AND</u>	6 hours
	BB.2 Continuously monitor hot leg level.	6 hours

SURVEILLANCE REQUIREMENTS

-----NOTE-----
Refer to Table 3.3.2-1 to determine which SRs apply for each Engineered Safety Features (ESF) Function.

SURVEILLANCE		FREQUENCY
SR 3.3.2.1	Perform CHANNEL CHECK.	12 hours
SR 3.3.2.2	Perform ACTUATION LOGIC TEST.	92 days on a STAGGERED TEST BASIS
SR 3.3.2.3	-----NOTE----- Verification of setpoint not required for manual initiation functions. ----- Perform TRIP ACTUATING DEVICE OPERATIONAL TEST (TADOT).	24 months
SR 3.3.2.4	-----NOTE----- This surveillance shall include verification that the time constants are adjusted to the prescribed values. ----- Perform CHANNEL CALIBRATION.	24 months
SR 3.3.2.5	Perform CHANNEL OPERATIONAL TEST (COT).	92 days
SR 3.3.2.6	Verify ESFAS RESPONSE TIMES are within limit.	24 months on a STAGGERED TEST BASIS

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.3.2.7 -----NOTE----- This Surveillance is not required to be performed for actuated equipment which is included in the Inservice Test (IST) Program. ----- Perform ACTUATION DEVICE TEST.</p>	24 months
<p>SR 3.3.2.8 Perform ACTUATION DEVICE TEST for squib valves.</p>	24 months
<p>SR 3.3.2.9 Perform ACTUATION DEVICE TEST for pressurizer heater circuit breakers.</p>	24 months

Table 3.3.2-1 (page 1 of 13)
Engineered Safeguards Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
1. Safeguards Actuation						
a. Manual Initiation	1,2,3,4	2 switches	E,O	SR 3.3.2.3		N/A
	5	2 switches	G,Y	SR 3.3.2.3		N/A
b. Containment Pressure - High 2	1,2,3,4	4	B,O	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[\leq 8.0 psig]
c. Pressurizer Pressure - Low	1,2,3(a)	4	B,M	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[\geq 1685 psig]
d. Steam Line Pressure - Low	1,2,3(a)	4 per steam line	B,M	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[\geq 390 psig](b)
e. RCS Cold Leg Temperature (T _{cold}) - Low	1,2,3(a)	4 per loop	B,M	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[\geq 500°F]

(continued)

(a) Above the P-11 (Pressurizer Pressure) interlock, when the RCS boron concentration is below that necessary to meet the shutdown margin requirements at an RCS temperature of 200°F.

(b) Time constants used in the lead/lag controller are $\tau_1 \geq [50]$ seconds and $\tau_2 \leq [5]$ seconds.

Reviewer Note: The values specified in brackets in the Trip Setpoint column are the Chapter 15 safety analysis values and are included for reviewer information only.

The values specified in brackets followed by " * " in the Trip Setpoint column are typical values for the Function. No credit was assumed for these Functions (typically diverse trips/actuators) in the Chapter 15 safety analyses and no safety analysis value is available.

The "Battery Charger Input Voltage - Low" Functions (15.c and 20.b) value specified is a typical value for the Function. The actual value will depend on the capabilities of the equipment selected with regard to its ability to function with degraded voltage as well as the setpoint methodology.

Following the setpoint study, the values specified in brackets must be replaced with the actual Trip Setpoints. Upon selection of the instrumentation the Trip Setpoints will be calculated in accordance with the setpoint methodology described in WCAP-14606. (WCAP-14606 is an AP600 document that describes a methodology that is applicable to AP1000. AP1000 has some slight differences in instrument spans as a result of the higher power level.) Allowable Values will be calculated in accordance with the setpoint methodology and specified in the Allowable Value column. The setpoint calculations will reflect the design basis and incorporate NRC accepted setpoint methodology.

The Containment Pressure - High 2 setpoint (Functions 1.b, 4.b, and 12.b) should be specified as low as reasonable, without creating potential for spurious trips during normal operations, consistent with the TMI action item (II.E.4.2) guidance.

Table 3.3.2-1 (page 2 of 13)
Engineered Safeguards Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPPOINT
2. Core Makeup Tank (CMT) Actuation						
a. Manual Initiation	1,2,3,4(j) 4(n), 5(I)	2 switches 2 switches	E,N E,U	SR 3.3.2.3 SR 3.3.2.3		N/A N/A
b. Pressurizer Water Level - Low 2	1,2,3,4(j) 4(n), 5(I)	4 4	B,N B,V	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6 SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[\geq 7.0%*] [\geq 1.0%] [\geq 7.0%*] [\geq 1.0%]
c. Safeguards Actuation	1,2,3,4,5(I)		Refer to Function 1 (Safeguards Actuation) for initiating functions and requirements.			
d. ADS Stages 1, 2, & 3 Actuation	1,2,3,4,5(I)		Refer to Function 9 (ADS Stages 1, 2 & 3 Actuation) for all initiating functions and requirements.			
3. Containment Isolation						
a. Manual Initiation	1,2,3,4 5(m), 6(m)	2 switches 2 switches	E,O G,Y	SR 3.3.2.3 SR 3.3.2.3		N/A N/A
b. Manual Initiation of Passive Containment Cooling	1,2,3,4,5(e,m), 6(e,m)		Refer to Function 12.a (Passive Containment Cooling Actuation) for initiating functions and requirements.			
c. Safeguards Actuation	1,2,3,4,5(m)		Refer to Function 1 (Safeguards Actuation) for initiating functions and requirements.			

(continued)

- (e) With decay heat > 9.0 Mwt.
- (I) With the RCS pressure boundary intact.
- (j) With the RCS not being cooled by the Normal Residual Heat Removal System (RNS).
- (m) Not applicable for valve isolation Functions whose associated flow path is isolated.
- (n) With the RCS being cooled by the RNS.

Table 3.3.2-1 (page 3 of 13)
Engineered Safeguards Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
4. Steam Line Isolation						
a. Manual Initiation	1,2(1),3(1),4(1)	2 switches	E,S	SR 3.3.2.3		N/A
b. Containment Pressure - High 2	1,2(1),3(1),4(1)	4	B,N	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤ 8.0 psig]
c. Steam Line Pressure						
(1) Steam Line Pressure - Low	1,2(1),3(a,1)	4 per steam line	B,M	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 390 psig] ^(b)
(2) Steam Line Pressure- Negative Rate - High	3(d,1)	4 per steam line	B,M	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤ 100 psi with time constant ≥ 50 seconds]
d. T _{cold} - Low	1,2(1),3(a,1)	4 per loop	B,M	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 500°F]
5. Turbine Trip						
a. Manual Main Feedwater Isolation	1,2	Refer to Function 6.a (Manual Main Feedwater Control Valve Isolation) for requirements.				
b. SG Narrow Range Water Level - High 2	1,2	4 per SG	B,L	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤ 100%]
c. Safeguards Actuation	1,2	Refer to Function 1 (Safeguards Actuation) for initiating functions and requirements.				
d. Reactor Trip	1,2	Refer to Function 18.a (ESFAS Interlocks, Reactor Trip, P-4) for requirements.				

(continued)

- (a) Above the P-11 (Pressurizer Pressure) interlock, when the RCS boron concentration is below that necessary to meet the shutdown margin requirements at an RCS temperature of 200°F.
- (b) Time constants used in the lead/lag controller are $\tau_1 \geq [50]$ seconds and $\tau_2 \leq [5]$ seconds.
- (d) Below the P-11 (Pressurizer Pressure) interlock.
- (1) Not applicable if all MSIVs are closed.

Table 3.3.2-1 (page 4 of 13)
Engineered Safeguards Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
6. Main Feedwater Control Valve Isolation						
a. Manual Initiation	1,2,3,4 ^(m)	2 switches	E,S	SR 3.3.2.3		N/A
b. SG Narrow Range Water Level - High 2	1,2,3,4 ^(j,m)	4 per SG	B,R	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤ 100%]
c. Safeguards Actuation	1,2,3,4 ^(m)	Refer to Function 1 (Safeguards Actuation) for all initiating functions and requirements.				
d. Reactor Coolant Average Temperature (T _{avg}) - Low 1	1,2	4	B,L	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 542°F*]
Coincident with Reactor Trip	1,2	Refer to Function 18.a (ESFAS Interlocks, Reactor Trip, P-4) for requirements.				
7. Main Feedwater Pump Trip and Valve Isolation						
a. Manual Initiation	Refer to Function 6.a (Manual Main Feedwater Control Valve Isolation) for requirements.					
b. SG Narrow Range Water Level - High 2	1,2,3,4 ^(j,m)	4 per SG	B,R	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤ 100%]
c. Safeguards Actuation	1,2,3,4 ^(m)	Refer to Function 1 (Safeguards Actuation) for initiating functions and requirements.				
d. Reactor Coolant Average Temperature T _{avg} - Low 2	1,2	2 per loop	B,L	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 542°F*]
Coincident with Reactor Trip	1,2	Refer to Function 18.a (ESFAS Interlocks, Reactor Trip, P-4) for requirements.				

(continued)

(j) With the RCS not being cooled by the Normal Residual Heat Removal System (RNS).

(m) Not applicable for valve isolation Functions whose associated flow path is isolated.

Table 3.3.2-1 (page 5 of 13)
Engineered Safeguards Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
8. Startup Feedwater Isolation						
a. SG Narrow Range Water Level - High 2	1,2,3,4(o)	4 per SG	B,S	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤ 100%]
b. T _{cold} - Low	1,2,3(a)	4 per loop	B,M	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 500°F]
c. Manual Initiation	Refer to Function 6.a (Manual Main Feedwater Control Valve Isolation) for requirements.					
9. ADS Stages 1, 2 & 3 Actuation						
a. Manual Initiation	1,2,3,4	2 switch sets	E,O	SR 3.3.2.3		N/A
	5(k),6(g,k)	2 switch sets	G,X	SR 3.3.2.3		N/A
b. Core Makeup Tank (CMT) Level - Low 1	1,2,3,4	4 per tank	B,O	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 67.5%] volume
	5(c,k)	4 per OPERABLE tank	B,V	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 67.5%] volume
Coincident with CMT Actuation	Refer to Function 2 (CMT Actuation) for all initiating functions and requirements.					

(continued)

- (a) Above the P-11 (Pressurizer Pressure) interlock, when the RCS boron concentration is below that necessary to meet the shutdown margin requirements at an RCS temperature of 200°F.
- (c) With pressurizer level ≥ 20%.
- (g) With upper internals in place.
- (o) Not applicable when the startup feedwater flow paths are isolated.
- (k) Not applicable when the required ADS valves are open. See LCO 3.4.13 and LCO 3.4.14 for ADS valve and equivalent relief area requirements.

Table 3.3.2-1 (page 6 of 13)
Engineered Safeguards Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
10. ADS Stage 4 Actuation						
a. Manual Initiation Coincident with	1,2,3,4	2 switch sets	E,O	SR 3.3.2.3		N/A
	5(k),6(g,k)	2 switch sets	G,X	SR 3.3.2.3		N/A
RCS Wide Range Pressure - Low, or	1,2,3,4	4	B,O	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 1200 psig]
	5(k),6(g,k)	4	B,X	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 1200 psig]
ADS Stages 1, 2 & 3 Actuation	Refer to Function 9 (Stages 1, 2, & 3 Actuation) for initiating functions and requirements					
b. CMT Level - Low 2	1,2,3,4	4 per tank	B,O	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 20% volume level span]
	5(c,k)	4 per OPERABLE tank	B,V	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 20% volume level span]
Coincident with RCS Wide Range Pressure - Low, and	1,2,3,4	4	B,O	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 1200 psig]
	5(c,k)	4	B,V	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 1200 psig]
Coincident with ADS Stages 1, 2 & 3 Actuation	1,2,3,4,5(c,k)	Refer to Function 9 (ADS Stages 1, 2 & 3 Actuation) for initiating functions and requirements				
c. Coincident RCS Loop 1 and 2 Hot Leg Level - Low 2	4(n),5(k),6(k)	1 per loop	BB,Y	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 3 in. above inside surface of the bottom of the hot legs]

(continued)

(c) With pressurizer level ≥ 20%.

(g) With upper internals in place.

(k) Not applicable when the required ADS valves are open. See LCO 3.4.13 and LCO 3.4.14 for ADS valve and equivalent relief area requirements.

Table 3.3.2-1 (page 7 of 13)
Engineered Safeguards Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
11. Reactor Coolant Pump Trip						
a. ADS Stages 1, 2 & 3 Actuation	Refer to Function 9 (ADS Stages 1, 2 & 3 Actuation) for initiating functions and requirements.					
b. Reactor Coolant Pump Bearing Water Temperature - High	1,2	4 per RCP	B,L	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤ 320°F*]
c. Manual CMT Actuation	Refer to Function 2.a (Manual CMT Actuation) for requirements.					
d. Pressurizer Water Level - Low 2	1,2,3,4(j)	4	B,N	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 7.0%*] [≥ 1.0%]
	4(n), 5(c,j)	4	B,V	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 7.0%*] [≥ 1.0%]
e. Safeguards Actuation	Refer to Function 1 (Safeguards Actuation) for initiating functions and requirements.					
12. Passive Containment Cooling Actuation						
a. Manual Initiation	1,2,3,4 5(e),6(e)	2 switches	E,O	SR 3.3.2.3		N/A
		2 switches	G,Y	SR 3.3.2.3		N/A
b. Containment Pressure - High 2	1,2,3,4	4	B,O	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤ 8.0 psig]

(continued)

- (c) With pressurizer level ≥ 20%.
- (e) With decay heat > 9.0 Mwt.
- (j) With the RCS not being cooled by the Normal Residual Heat Removal System (RNS).
- (n) With the RCS being cooled by the RNS.

Table 3.3.2-1 (page 8 of 13)
Engineered Safeguards Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
13. Passive Residual Heat Removal Heat Exchanger Actuation						
a. Manual Initiation	1,2,3,4 5(I)	2 switches	E,O	SR 3.3.2.3		N/A
		2 switches	E,U	SR 3.3.2.3		N/A
b. SG Narrow Range Water Level - Low	1,2,3,4(j)	4 per SG	B,N	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 95,000 lbm]
Coincident with Startup Feedwater Flow - Low	1,2,3,4(j)	2 per feedwater line	H,N	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 200 gpm per SG*]
c. SG Wide Range Water Level - Low	1,2,3,4(j)	4 per SG	B,N	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 55,000 lbm]
d. ADS Stages 1,2 & 3 Actuation	1,2,3,4,5(I)	Refer to Function 9 (ADS Stages 1, 2 & 3 Actuation) for initiating functions and requirements.				
e. CMT Actuation	Refer to Function 2 (CMT Actuation) for initiating functions and requirements.					
f. Pressurizer Water Level, High 3	1,2,3,4(j,p)	4	B,N	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤80%*]

(continued)

(I) With the RCS pressure boundary intact.

(j) With the RCS not being cooled by the Normal Residual Heat Removal System (RNS).

(p) Above the P-19 (RCS Pressure) interlock.

Table 3.3.2-1 (page 9 of 13)
Engineered Safeguards Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
14. SG Blowdown Isolation						
a. Passive Residual Heat Removal Heat Exchanger Actuation	1,2,3,4(j,m)	Refer to Function 13 (Passive Residual Heat Removal Heat Exchanger Actuation) for all initiating functions and requirements.				
b. SG Narrow Range Water Level - Low	1,2,3,4(j)	4 per SG	B,R	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 95,000 lbm]
15. Boron Dilution Block						
a. Source Range Neutron Flux Multiplication	2(f),3,4(m)	4	B,T	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤ Source Range Flux X 1.6 in 50 minutes]
	5(m)	4	B,P	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤ Source Range Flux X 1.6 in 50 minutes]
b. Reactor Trip	Refer to Function 18.a (ESFAS Interlocks, Reactor Trip, P-4) for all requirements.					
c. Battery Charger Input Voltage - Low	1,2,3,4(m)	4 divisions	B,T	SR 3.3.2.3 SR 3.3.2.4		[≥ 343 v*]
	5(m)	4 divisions	B,P	SR 3.3.2.3 SR 3.3.2.4		[≥ 343 v*]
16. Chemical Volume and Control System Makeup Isolation						
a. SG Narrow Range Water Level - High 2	1,2,3(m),4(j,m)	4 per SG	B,R	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤ 100%]
b. Pressurizer Water Level - High 1	1,2,3(m)	4	B,Q	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤ 30%*]
	Coincident with Safeguards Actuation	1,2,3(m)	Refer to Function 1 (Safeguards Actuation) for initiating functions and requirements.			
c. Pressurizer Water Level - High 2	1,2,3,4(j,m,p)	4	B,T	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤ 67%]
d. Containment Radioactivity - High 2	1,2,3(m)	4	B,Q	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤ 100 R/hr]
e. Manual Initiation	1,2,3(m),4(j,m)	2 switches	E,R	SR 3.3.2.3		N/A

(continued)

- (f) Below the P-6 (Intermediate Range Neutron Flux) interlocks.
(j) With the RCS not being cooled by the Normal Residual Heat Removal System (RNS).
(m) Not applicable for valve isolation Functions whose associated flow path is isolated.
(p) Above the P-19 (RCS Pressure) interlock.

Table 3.3.2-1 (page 10 of 13)
Engineered Safeguards Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
17. Normal Residual Heat Removal System Isolation						
a. Containment Radioactivity - High 2	1,2,3 ^(m)	4	B,Q	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤ 100 R/hr]
b. Safeguards Actuation	1,2,3 ^(m)	Refer to Function 1 (Safeguards Actuation) for all initiating functions and requirements.				
c. Manual Initiation	1,2,3 ^(m)	2 switch sets	E,Q	SR 3.3.2.3		N/A
18. ESFAS Interlocks						
a. Reactor Trip, P-4	1,2,3	3 divisions	D,M	SR 3.3.2.3		N/A
b. Pressurizer Pressure, P-11	1,2,3	4	J,M	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤ 1970 psig]
c. Intermediate Range Neutron Flux, P-6	2	4	J,L	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 1E-10 amps]
d. Pressurizer Level, P-12	1,2,3,4,5,6	4	J,M BB,Y	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[Above Pressurizer Water Level - Low 1 setpoint of 20%]
e. RCS Pressure, P-19	1,2,3,4 ^(j)	4	J,N	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 700 psig]
19. Containment Air Filtration System Isolation						
a. Containment Radioactivity - High 1	1,2,3,4 ^(j)	4	B,Z	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≤ 2 R/hr]
b. Containment Isolation	Refer to Function 3 (Containment Isolation) for initiating functions and requirements.					

(continued)

(j) With the RCS not being cooled by the Normal Residual Heat Removal System (RNS).

(m) Not applicable for valve isolation Functions whose associated flow path is isolated.

Table 3.3.2-1 (page 11 of 13)
Engineered Safeguards Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPPOINT
20. Main Control Room Isolation and Air Supply Initiation						
a. Control Room Air Supply Radiation - High 2	1,2,3,4	2	F,O	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[$\leq 2 \times 10^{-6}$ curies/m ³ Dose Equivalent I-131]
	Note (h)	2	G,K	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[$\leq 2 \times 10^{-6}$ curies/m ³ Dose Equivalent I-131]
b. Battery Charger Input Voltage - Low	1,2,3,4	4 divisions	B,O	SR 3.3.2.3 SR 3.3.2.4		[≥ 343 V*]
	Note (h)	4 divisions	G,K	SR 3.3.2.3 SR 3.3.2.4		[≥ 343 V*]
21. Auxiliary Spray and Purification Line Isolation						
a. Pressurizer Water Level - Low 1	1,2	4	B,L	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[20.0%*]
b. Manual Initiation	1,2	Refer to Function 16.e (Manual Chemical Volume Control System (Makeup Isolation) for requirements.				
22. In-Containment Refueling Water Storage Tank (IRWST) Injection Line Valve Actuation						
a. Manual Initiation	1,2,3,4(j)	2 switch sets	E,N	SR 3.3.2.3		N/A
	4(n),5,6	2 switch sets	G,Y	SR 3.3.2.3		N/A
b. ADS 4th Stage Actuation	Refer to Function 10 (ADS 4th Stage Actuation) for initiating functions and requirements.					
c. Coincident RCS Loop 1 and 2 Hot Leg Level - Low 2	4(n),5,6	1 per loop	BB,Y	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 3 in. above inside surface of the bottom of the hot legs]

(continued)

- (h) During movement of irradiated fuel assemblies.
(j) With the RCS not being cooled by the Normal Residual Heat Removal System (RNS).
(n) With the RCS being cooled by the RNS.

Table 3.3.2-1 (page 12 of 13)
Engineered Safeguards Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
23. IRWST Containment Recirculation Valve Actuation						
a. Manual Initiation	1,2,3,4(j)	2 switch sets	E,N	SR 3.3.2.3		N/A
	4(n),5,6	2 switch sets	G,Y	SR 3.3.2.3		N/A
b. ADS Stage 4 Actuation	Refer to Function 10 (ADS Stage 4 Actuation) for all initiating functions and requirements.					
Coincident with IRWST Level - Low 3	1,2,3,4(j)	4	B,N	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ Contain- ment Elevation @ 107'2"]
	4(n),5(k),6(k)	4	I,Y	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ Contain- ment Elevation @ 107'2"]
24. Refueling Cavity Isolation						
a. Spent Fuel Pool Level - Low	6	3	H,P	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[37.5 ft.]
25. ESF Coincidence Logic						
a. Coincidence Logic	1,2,3,4	4 divisions, 1 battery- backed subsystem per division	D,O	SR 3.3.2.2		N/A
	5,6	4 divisions, 1 battery- backed subsystem per division	G,W	SR 3.3.2.2		N/A

(continued)

(k) Not applicable when the required ADS valves are open. See LCO 3.4.13 and LCO 3.4.14 for ADS valve and equivalent relief area requirements.

Table 3.3.2-1 (page 13 of 13)
Engineered Safeguards Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE	TRIP SETPOINT
26. ESF Actuation						
a. ESF Actuation Subsystem	1,2,3,4	4 divisions, 1 battery-backed subsystem per division	D,O	SR 3.3.2.2 SR 3.3.2.7 SR 3.3.2.8		N/A
	5,6	4 divisions, 1 battery-backed subsystem per division	G,W	SR 3.3.2.2 SR 3.3.2.7		N/A
27. Pressurizer Heater Trip						
a. Core Makeup Tank Actuation	1,2,3,4(j,p)	Refer to Function 2 (Core Makeup Tank Actuation) for all initiating functions and requirements. In addition to the requirements for Function 2, SR 3.3.2.9 also applies.				
b. Pressurizer Water Level, High 3	1,2,3,4(j,p)	4	B,N	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[80%*]
28. Chemical and Volume Control System Letdown Isolation						
a. Hot Leg Level - Low 1	4(n),5,6(q)	1 per loop	C,AA	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[≥ 18 in. above inside surface of the bottom of the hot legs]
29. SG Power Operated Relief Valve and Block Valve Isolation						
a. Manual Initiation	1,2,3,4(j)	2 switches	E,N	SR 3.3.2.3		N/A
b. Steam Line Pressure - Low	1,2,3,4(j)	4 per steam line	B,N	SR 3.3.2.1 SR 3.3.2.4 SR 3.3.2.5 SR 3.3.2.6		[390 psig](b)

- (b) Time constants used in the lead/lag controller are $\tau_1 \geq [50]$ seconds and $\tau_2 \leq [5]$ seconds.
- (j) With the RCS not being cooled by the Normal Residual Heat Removal System (RNS).
- (m) Not applicable for valve isolation Functions whose associated flow path is isolated.
- (n) With the RCS being cooled by the RNS.
- (p) Above the P-19 (RCS Pressure) interlock.
- (q) With the water level < 23 feet above the top of the reactor vessel flange.

3.3 INSTRUMENTATION

3.3.3 Post Accident Monitoring (PAM) Instrumentation

LCO 3.3.3 The PAM instrumentation for each Function in Table 3.3.3-1 shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTIONS

- NOTES-----
1. LCO 3.0.4 not applicable.
 2. Separate Condition entry is allowed for each Function.
-

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more Functions with one required channel inoperable.	A.1 Restore required channel to OPERABLE status.	30 days
B. Required Action and associated Completion Time of Condition A not met.	B.1 Initiate action in accordance with Specification 5.6.7.	Immediately
C. One or more Functions with two required channels inoperable.	C.1 Restore one channel to OPERABLE status.	7 days

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. Required Action and associated Completion Time of Condition C not met.	D.1 Enter the Condition referenced in Table 3.3.3-1 for the channel.	Immediately
E. As required by Required Action D.1 and referenced in Table 3.3.3-1.	E.1 Be in MODE 3. <u>AND</u> E.2 Be in MODE 4.	6 hours 12 hours

SURVEILLANCE REQUIREMENTS

-----NOTE-----
SR 3.3.3.1 and SR 3.3.3.2 apply to each PAM instrumentation Function in Table 3.3.3-1.

SURVEILLANCE		FREQUENCY
SR 3.3.3.1	Perform CHANNEL CHECK for each required instrumentation channel that is normally energized.	31 days
SR 3.3.3.2	-----NOTE----- Neutron detectors are excluded from CHANNEL CALIBRATION. ----- Perform CHANNEL CALIBRATION.	24 months

Table 3.3.3-1 (page 1 of 1)
Post-Accident Monitoring Instrumentation

FUNCTION		REQUIRED CHANNELS/ DIVISIONS	CONDITION REFERENCED FROM REQUIRED ACTION D.1
1.	Neutron Flux (Intermediate Range)	2	E
2.	Reactor Coolant System (RCS) Hot Leg Temperature (Wide Range)	2	E
3.	RCS Cold Leg Temperature (Wide Range)	2	E
4.	RCS Pressure (Wide Range)	2	E
5.	Pressurizer Pressure and RCS Subcooling Monitor ^(a)	2	E
6.	Containment Water Level	2	E
7.	Containment Pressure	2	E
8.	Containment Pressure (Extended Range)	2	E
9.	Containment Area Radiation (High Range)	2	E
10.	Pressurizer Level and Associated Reference Leg Temperature	2	E
11.	IRWST Water Level	2	E
12.	PRHR Flow and PRHR Outlet Temperature	2 flow & 1 temperature	E
13.	Core Exit Temperature--Quadrant 1	2(b)	E
14.	Core Exit Temperature--Quadrant 2	2(b)	E
15.	Core Exit Temperature--Quadrant 3	2(b)	E
16.	Core Exit Temperature--Quadrant 4	2(b)	E
17.	PCS Storage Tank Level and PCS Flow	2 level & 1 flow	E
18.	Remotely Operated Containment Isolation Valve Position	1/valve ^(c)	E
19.	IRWST to RNS Suction Valve Status	2	E

(a) RCS Subcooling calculated from pressurizer pressure and RCS hot leg temperature.

(b) A channel consists of two thermocouples within a single division. Each quadrant contains two divisions. The minimum requirement is two OPERABLE thermocouples in each of the two divisions.

(c) Not required for isolation valves whose associated penetration is isolated by at least one closed and deactivated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.

3.3 INSTRUMENTATION

3.3.4 Remote Shutdown Workstation

LCO 3.3.4 The Remote Shutdown Workstation (RSW) shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and
MODE 4 with RCS average temperature (T_{avg}).

ACTIONS

-----NOTE-----
LCO 3.0.4 is not applicable.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. RSW inoperable.	A.1 Restore to OPERABLE status.	30 days
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3.	6 hours
	<u>AND</u> B.2 Be in MODE 4 with $T_{avg} < 350^{\circ}\text{F}$.	12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.3.4.1	Verify each required transfer switch is capable of performing the required function.	24 months
SR 3.3.4.2	Verify that the RSW communicates indication and controls with Division A, B, C and D of the PMS.	24 months
SR 3.3.4.3	Verify the OPERABILITY of the RSW hardware and software.	24 months
SR 3.3.4.4	Perform TADOT of the reactor trip breaker open/closed indication.	24 months

3.3 INSTRUMENTATION

3.3.5 Diverse Actuation System (DAS) Manual Controls

LC0 3.3.5 The DAS manual controls for each function in Table 3.3.5-1 shall be operable.

APPLICABILITY: According to Table 3.3.5-1.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more manual DAS controls inoperable.	A.1 Restore DAS manual controls to OPERABLE status.	30 days
B. Completion Time of Required Action A not met for inoperable DAS manual reactor trip control.	B.1 Perform SR 3.3.1.5. <u>AND</u> B.2 Restore all controls to OPERABLE status.	Once per 31 days on a STAGGERED TEST BASIS Prior to entering MODE 2 following next MODE 5 entry
C. Completion Time of Required Action A not met for inoperable DAS manual actuation control other than reactor trip.	C.1 Perform SR 3.3.2.3. <u>AND</u> C.2 Restore all controls to OPERABLE status.	Once per 31 days on a STAGGERED TEST BASIS Prior to entering MODE 2 following next MODE 5 entry

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. Completion Time of Required Action B not met.	D.1 Be in MODE 3.	6 hours
	<u>AND</u>	
<u>OR</u>	D.2 Be in MODE 5.	36 hours
Completion Time of Required Action C not met.		

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.3.5.1 -----NOTE ----- Verify of setpoint not required. -----	
Perform TRIP ACTUATION DEVICE OPERATIONAL TEST (TADOT).	24 months

Table 3.3.5-1 (page 1 of 1)
DAS Manual Controls

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CONTROLS
1. Reactor Trip Manual Controls	1,2	2 switches
2. PRHR HX control and IRWST Gutter control valves	1,2,3,4,5(a)	2 switches
3. CMT isolation valves	1,2,3,4,5(a)	2 switches
4. ADS stage 1 valves	1,2,3,4,5(a)	2 switches
5. ADS stage 2 valves	1,2,3,4,5(a)	2 switches
6. ADS stage 3 valves	1,2,3,4,5(a)	2 switches
7. ADS stage 4 valves	1,2,3,4,5,6(c)	2 switches
8. IRWST injection squib valves	1,2,3,4,5,6	2 switches
9. Containment recirc valves	1,2,3,4,5,6	2 switches
10. Passive Containment Cooling Drain Valves	1,2,3,4,5(b),6(b)	2 switches
11. Selected Containment Isolation Valves	1,2,3,4,5,6	2 switches

(a) With RCS pressure boundary intact.

(b) With the calculated reactor decay heat > 9.0 MWt.

(c) In MODE 6 with reactor internals in place.