CP&L
A Progress Energy Company

10 CFR 50.90

NOV 1 3 2002

SERIAL: BSEP 02-0159
TSC-2002-05

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-325 AND 50-324/LICENSE NOS. DPR-71 AND DPR-62
APPLICATION FOR TECHNICAL SPECIFICATION CHANGE REGARDING
ELIMINATION OF POST ACCIDENT SAMPLING SYSTEM REQUIREMENTS

Ladies and Gentlemen:

In accordance with the provisions of 10 CFR 50.90, Carolina Power & Light (CP&L) Company is submitting a request for amendments to the Technical Specifications (TSs) for the Brunswick Steam Electric Plant (BSEP), Unit Nos. 1 and 2. The proposed license amendments eliminate the TS requirements for Post Accident Sampling capability in TS Section 5.5.3. The changes are consistent with NRC approvedTechnical Specification Task Force (TSTF) change TSTF-413, as modified by Federal Register Notice 66 FR 66949, of December 27, 2001, and in response to public comments. The availability of this TS improvement was published in the *Federal Register* on March 20, 2002, as part of the Consolidated Line Item Improvement Process (CLIIP).

CP&L has evaluated the proposed changes in accordance with 10 CFR 50.91(a)(1), using the criteria in 10 CFR 50.92(c), and determined that these changes involve no significant hazards considerations.

CP&L requests approval of the proposed license amendments by March 31, 2003, with the amendments being fully implemented within 180 days after approval.

In accordance with 10 CFR 50.91(b)(1), a copy of this application is being provided to the State of North Carolina.

A001

Brunswick Nuclear Plant
PO Box 10429
Southport, NC 28461

Please refer any questions regarding this submittal to Mr. Edward T. O'Neil, Manager – Regulatory Affairs, at (910) 457-3512.

Sincerely,

John S. Keenan

KMH/kmh

Enclosures:
1. Description and Assessment
2. Marked-Up Technical Specification Changes for Unit 1
3. Marked-Up Technical Specification Changes for Unit 2
4. Revised Technical Specification Pages for Unit 1
5. Revised Technical Specification Pages for Unit 2
6. Regulatory Commitments

John S. Keenan, having been first duly sworn, did depose and say that the information contained herein is true and correct to the best of his information, knowledge and belief; and the sources of his information are officers, employees, and agents of Carolina Power & Light Company.

Notary (Seal)

My commission expires: August 29, 2004

cc (with enclosures):

U. S. Nuclear Regulatory Commission, Region II
ATTN: Mr. Luis A. Reyes, Regional Administrator
Sam Nunn Atlanta Federal Center
61 Forsyth Street, SW, Suite 23T85
Atlanta, GA 30303-8931

U. S. Nuclear Regulatory Commission
ATTN: Mr. Theodore A. Easlick, NRC Senior Resident Inspector
8470 River Road
Southport, NC 28461-8869

U. S. Nuclear Regulatory Commission
ATTN: Ms. Brenda L. Mozafari (Mail Stop OWFN 8G9) **(Electronic Copy Only)**
11555 Rockville Pike
Rockville, MD 20852-2738

U. S. Nuclear Regulatory Commission
ATTN: Leonard N. Olshan (Mail Stop OWFN 8H12)
11555 Rockville Pike
Rockville, MD 20852-2738

U. S. Nuclear Regulatory Commission
ATTN: Allen G. Howe (Mail Stop OWFN 8G9)
11555 Rockville Pike
Rockville, MD 20852-2738

Ms. Jo A. Sanford
Chair - North Carolina Utilities Commission
P.O. Box 29510
Raleigh, NC 27626-0510

Ms. Beverly O. Hall, Section Chief
Radiation Protection Section, Division of Environmental Health
North Carolina Department of Environment and Natural Resources
3825 Barrett Drive
Raleigh, NC 27609-7221

DESCRIPTION AND ASSESSMENT

1.0 DESCRIPTION

The proposed amendments revise the Technical Specification (TS) Section 5.5.3 to eliminate the requirements for having a Post Accident Sampling Sy stem (PASS).

The changes are consistent with NRC approved Technical Specification Task Force (TSTF) change TSTF-413, as modified by Federal Register Notice 66 FR 66949, of December 27, 2001, and in response to public comments. The availability of this TS improvement was published in the *Federal Register* on March 20, 2002, as part of the Consolidated Line Item Improvement Process (CLIIP).

2.0 ASSESSMENT

2.1 Applicability of Published Safety Evaluation

The results of the Boiling Water Reactor Owners' Group (BWROG) Topical Report, NEDO-32991-A, "Regulatory Relaxation for BWR Post-Accident Sampling Stations (PASS)," dated August 2001, confirmed that the BWRPASS does not provide the benefits expected by the NRC when the requirements were promulgated following the Three Mile Island Unit 2 accident. BWR emergency and severe accident response strategies can be implemented using in-plant instrumentation, without reliance on the PASS. Moreover, operating experience has demonstrated that in-plant instrumentation and the associated analysis methods will provide the timely information required to assess core damage and mitigate severe accidents. This information is available from in-plant instrumentation early in the accident scenario and the derived information is as good as or better than the information currently provided by the PASS several hours after initiation of the event. The BWROG has, therefore, concluded that thePASS can be removed without significantly affecting plant safety and recommended that all PASS regulatory requirements be eliminated.

Carolina Power & Light (CP&L) Company has reviewed the safety evaluation published on December 27, 2001, as part of the CLIIP. This included a review of the NRC's evaluation, as well as the information provided to support TSTF-413. CP&L has concluded that the justifications presented in the TSTF proposal and the NRC's safety evaluation are applicable to the Brunswick Steam Electric Plant (BSEP), Unit Nos. 1 and 2, and justify these changes for implementation in the BSEP TSs.

2.2 Optional Changes and Variations

CP&L is proposing deviations from the TS changes described in TSTF-413. The TSTF "Affected Technical Specifications" includes a mark-up of Bases 3.3.3.1.D for Post Accident Monitoring Instrumentation. Based on the existing wording of the BSEP Units 1 and 2 TS Bases 3.3.3.1.D for Post Accident Monitoring Instrumentation, this bases change is not required for BSEP.

There is no other variation nor deviation from the NRC Model Safety Evaluation dated December 27, 2001.

3.0 REGULATORY ANALYSIS

3.1 No Significant Hazards Consideration

CP&L has reviewed the proposed no significant hazards consideration (NSHC) published in the *Federal Register* on December 27, 2001, and has concluded that the proposed NSHC is applicable to BSEP, Units 1 and 2, and is hereby incorporated by reference to satisfy the requirements of 10 CFR 50.91(a).

3.2 Verification and Commitments

As discussed in the notice of availability published in the *Federal Register* on December 27, 2001, for this TS improvement, plant-specific verifications were performed as follows:

1. BSEP will develop contingency plans for obtaining and analyzing highly radioactive samples of reactor coolant, suppression pool, and containment atmosphere. The contingency plans will be contained in applicable radiation control procedures and implementation will be completed within 180 days after the issuance of the license amendment. Establishment and maintenance of the contingency plans are considered to be a regulatory commitment.

2. BSEP will establish the capability of classifying fuel damage events at the Alert level threshold at radioactivity levels of 300 μCi/ml dose equivalent iodine. This capability will utilize the normal sampling system and/or correlations of radiation readings to radioisotope concentrations in the reactor coolant. The method will be described in applicable emergency preparedness procedures and implementation will be completed within 180 days after the issuance of the license amendment. The capability for classifying fuel damage events is considered to be a regulatory commitment.

3. BSEP has established an I-131 site survey detection capability including the ability to assess radioactive iodines released to offsite environs, by using effluent monitoring systems or portable sampling equipment. This capability is currently described in applicable radiation control procedures and will be maintained as a regulatory

commitment within the plant procedures. Annotation of this commitment in applicable procedures will be completed within 180 days after the issuance of the license amendment.

4.0 Environmental Evaluation

CP&L has reviewed the environmental evaluation included in the model safety evaluation published December 27, 2001, as part of the CLIIP and concluded that the NRC's findings are applicable to BSEP, Units 1 and 2. The evaluation is hereby incorporated by reference for this application.

MARKED-UP TECHNICAL SPECIFICATION CHANGES FOR UNIT 1

MARKED-UP TECHNICAL SPECIFICATION CHANGES FOR UNIT 2

REVISED TECHNICAL SPECIFICATION PAGES FOR UNIT 1

REVISED TECHNICAL SPECIFICATION PAGES FOR UNIT 2

REGULATORY COMMITMENTS

The table below lists actions committed to by Carolina Power & Light (CP&L) Company in this document. Any other actions discussed in this submittal represent intended or planned actions by CP&L. They are described for the NRC's information and are not regulatory commitments. Please notify the Manager – Regulatory Affairs at the Brunswick Steam Electric Plant (BSEP) of any questions regarding this document or any associated regulatory commitments.

REGULATORY COMMITMENT	DUE DATE/EVENT
1. BSEP will develop contingency plans for obtaining and analyzing highly radioactive samples of reactor coolant, suppression pool, and containment atmosphere. The contingency plans will be contained in applicable radiation control procedures and implementation will be completed within 180 days after the issuance of the license amendments. Establishment and maintenance of the contingency plans are considered to be a regulatory commitment.	To be implemented within 180 days of issuance of the amendments
2. BSEP will establish the capability of classifying fuel damage events at the Alert level threshold at radioactivity levels of 300 μCi/ml dose equivalent iodine. This capability may utilize the normal sampling system and/or correlations of radiation readings to radioisotope concentrations in the reactor coolant. The method will be described in applicable emergency preparedness procedures and implementation will be completed within 180 days after the issuance of the license amendments. The capability for classifying fuel damage events is considered to be a regulatory commitment.	To be implemented within 180 days of issuance of the amendments
3. BSEP has established an I-131 site survey detection capability including the ability to assess radioactive iodines released to offsite environs, by using effluent monitoring systems or portable sampling equipment. This capability is currently described in applicable radiation control procedures and will be maintained as a regulatory commitment within the plant procedures. Annotation of this commitment in applicable procedures will be completed within 180 days after the issuance of the license amendments.	To be implemented within 180 days of issuance of the amendments