

November 19, 2002

LICENSEE: Rochester Gas and Electric Corporation (RG&E)

SUBJECT: SUMMARY OF OCTOBER 15, 2002, MEETING WITH RG&E REGARDING
REVIEW OF EQUIPMENT QUALIFICATION (EQ) CALCULATIONS FOR
R. E. GINNA NUCLEAR POWER STATION

On October 15, 2002, representatives of RG&E met with the Nuclear Regulatory Commission (NRC) staff to review a sample of EQ calculations. A list of meeting attendees is enclosed. The following calculations were reviewed by the staff:

1. 4.4.1.4 Valcor Solenoid Operated Valve Models V526-6042-3 and V526-6042-17
2. 4.4.5.10 General PVC Insulated and Jacketed Control Cable
3. 4.4.5.14 Conax Core Exit Thermocouple Connector/Cable Assemblies
4. 4.4.5.3 Conax Electric Conductor Seal Assembly
5. 4.4.3.2 Westinghouse/WX32714 Electrical Penetration Assembly
6. 4.4.5.1 Kerite 600V HTK Insulated, FR Jacketed Power Cable
7. 4.4.2.1 Westinghouse Containment Recirculation Fan Motor
8. 4.4.6.5 AMP Nuclear PIDG Window Butt Splices and Terminals
9. 4.4.7.2 Conax RTD Model 7DB9-10000
10. 4.4.4.4 Raychem WCSF-N Nuclear Low Voltage Tubing
11. 4.4.5.16 Brand-Rex XLPE Insulated and CSPE Jacketed Cable
12. 4.4.2.2 Limitorque Actuators, Outside Containment
13. 4.4.8 Victoreen High Range Radiation Monitor
14. 4.4.1.2 Valcor Solenoid Operated Valve Model V526-5440-2
15. 4.4.2.4 Limitorque Motor Operated Valve Model SMB-00-15 with Reliance Motor

These sample calculations represented the following equipment: solenoid operated valves, electric motors, electrical penetration assemblies, heat shrink tubing, wire and cable, electrical connectors, resistance temperature detectors, and high range radiation monitor.

For evaluations of the EQ time-limited aging analyses (TLAA) the applicant is implementing 10 CFR 54.21(c)(1)(i), (ii), and (iii). The applicant discussed items mostly falling under 10 CFR 54.21(c)(1)(ii), but also provided examples of the use of 10 CFR 54.21(c)(1)(i) and 10 CFR 54.21(c)(1)(iii). The Arrhenius methodology was used to perform the thermal aging evaluation, and the temperature data used in the evaluation were based on plant design temperature or on the actual plant temperature data.

It was explained by the applicant that, due to the date of installation of certain equipment, reanalysis was not required for license renewal purposes for equipment whose previous thermal and radiation analyses support a qualified life of 40 years.

The applicant also confirmed that there are no plans to extend the qualification of certain equipment which is expected to be replaced at the end of its qualified life. For the equipment

that is not expected to be replaced at the end of its qualified life, the applicant will perform a reanalysis in accordance with 10 CFR 54.21(c)(1)(iii).

In summary, the EQ calculations reviewed at the meeting reflected the reanalysis methods used by the applicant. The NRC staff will consider this information in its review of the applicant's TLAA associated with EQ.

RG&E has had an opportunity to review and comment on this summary.

/RA/

Russell Arrighi, Project Manager
License Renewal Section
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket No.: 50-244

Enclosure: As stated

cc w/enclosure: See next page

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RG&E/NRC EQ MEETING
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OCTOBER 15, 2002

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Enclosure