

November 19, 2002

LICENSEE: Rochester Gas and Electric Corporation (RG&E)

SUBJECT: SUMMARY OF OCTOBER 15, 2002, MEETING WITH RG&E REGARDING  
REVIEW OF EQUIPMENT QUALIFICATION (EQ) CALCULATIONS FOR  
R. E. GINNA NUCLEAR POWER STATION

On October 15, 2002, representatives of RG&E met with the Nuclear Regulatory Commission (NRC) staff to review a sample of EQ calculations. A list of meeting attendees is enclosed. The following calculations were reviewed by the staff:

1. 4.4.1.4 Valcor Solenoid Operated Valve Models V526-6042-3 and V526-6042-17
2. 4.4.5.10 General PVC Insulated and Jacketed Control Cable
3. 4.4.5.14 Conax Core Exit Thermocouple Connector/Cable Assemblies
4. 4.4.5.3 Conax Electric Conductor Seal Assembly
5. 4.4.3.2 Westinghouse/WX32714 Electrical Penetration Assembly
6. 4.4.5.1 Kerite 600V HTK Insulated, FR Jacketed Power Cable
7. 4.4.2.1 Westinghouse Containment Recirculation Fan Motor
8. 4.4.6.5 AMP Nuclear PIDG Window Butt Splices and Terminals
9. 4.4.7.2 Conax RTD Model 7DB9-10000
10. 4.4.4.4 Raychem WCSF-N Nuclear Low Voltage Tubing
11. 4.4.5.16 Brand-Rex XLPE Insulated and CSPE Jacketed Cable
12. 4.4.2.2 Limitorque Actuators, Outside Containment
13. 4.4.8 Victoreen High Range Radiation Monitor
14. 4.4.1.2 Valcor Solenoid Operated Valve Model V526-5440-2
15. 4.4.2.4 Limitorque Motor Operated Valve Model SMB-00-15 with Reliance Motor

These sample calculations represented the following equipment: solenoid operated valves, electric motors, electrical penetration assemblies, heat shrink tubing, wire and cable, electrical connectors, resistance temperature detectors, and high range radiation monitor.

For evaluations of the EQ time-limited aging analyses (TLAA) the applicant is implementing 10 CFR 54.21(c)(1)(i), (ii), and (iii). The applicant discussed items mostly falling under 10 CFR 54.21(c)(1)(ii), but also provided examples of the use of 10 CFR 54.21(c)(1)(i) and 10 CFR 54.21(c)(1)(iii). The Arrhenius methodology was used to perform the thermal aging evaluation, and the temperature data used in the evaluation were based on plant design temperature or on the actual plant temperature data.

It was explained by the applicant that, due to the date of installation of certain equipment, reanalysis was not required for license renewal purposes for equipment whose previous thermal and radiation analyses support a qualified life of 40 years.

The applicant also confirmed that there are no plans to extend the qualification of certain equipment which is expected to be replaced at the end of its qualified life. For the equipment

that is not expected to be replaced at the end of its qualified life, the applicant will perform a reanalysis in accordance with 10 CFR 54.21(c)(1)(iii).

In summary, the EQ calculations reviewed at the meeting reflected the reanalysis methods used by the applicant. The NRC staff will consider this information in its review of the applicant's TLAA associated with EQ.

RG&E has had an opportunity to review and comment on this summary.

*/RA/*

Russell Arrighi, Project Manager  
License Renewal Section  
License Renewal and Environmental Impacts Program  
Division of Regulatory Improvement Programs  
Office of Nuclear Reactor Regulation

Docket No.: 50-244

Enclosure: As stated

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NAME	RArrighi	HBerilla		SLee
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RLEP RF

R. Arrighi

**E-MAIL:**

PUBLIC

J. Johnson

W. Borchardt

D. Matthews

F. Gillespie

RidsNrrDe

R. Barrett

E. Imbro

G. Bagchi

K. Manoly

W. Bateman

J. Calvo

C. Holden

P. Shemanski

H. Nieh

G. Holahan

H. Walker

S. Black

B. Boger

D. Thatcher

G. Galletti

C. Li

J. Moore

R. Weisman

M. Mayfield

A. Murphy

W. McDowell

S. Smith (srs3)

T. Kobetz

R. Assa

C. Munson

RLEP Staff

-----

R. Clark

B. Platchek (RI)

A. Fernandez (OGC)

R.E. Ginna Nuclear Power Plant

cc:

Kenneth Kolaczyk, Sr. Resident Inspector  
R.E. Ginna Plant  
U.S. Nuclear Regulatory Commission  
1503 Lake Road  
Ontario, NY 14519

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, PA 19406

Mr. William M. Flynn, President  
New York State Energy, Research,  
and Development Authority  
Corporate Plaza West  
286 Washington Avenue Extension  
Albany, NY 12203-6399

Charles Donaldson, Esquire  
Assistant Attorney General  
New York Department of Law  
120 Broadway  
New York, NY 10271

Daniel F. Stenger  
Ballard Spahr Andrews & Ingersoll, LLP  
601 13<sup>th</sup> Street, N.W., Suite 1000 South  
Washington, DC 20005

Ms. Thelma Wideman, Director  
Wayne County Emergency Management  
Office  
Wayne County Emergency Operations  
Center  
7336 Route 31  
Lyons, NY 14489

Ms. Mary Louise Meisenzahl  
Administrator, Monroe County  
Office of Emergency Preparedness  
1190 Scottsville Road, Suite 200  
Rochester, NY 14624

Dr. Robert C. Mecredy  
Vice President, Nuclear Operations  
Rochester Gas and Electric Corporation  
89 East Avenue  
Rochester, NY 14649

Mr. Paul Eddy  
New York State Department of  
Public Service  
3 Empire State Plaza, 10th Floor  
Albany, NY 12223

Mr. Alan P. Nelson  
Nuclear Energy Institute  
1776 I Street, N.W., Suite 400  
Washington, DC 20006-3708  
APN@NEI.ORG

George Wrobel  
Manager, License Renewal  
R.E.Ginna Nuclear Power Plant  
1503 Lake Rd.  
Ontario, NY 14519

Mr. Denis Wickham  
Sr. Vice President Transmission & Supply  
Energy East Management Corporation  
P.O. Box 5224  
Binghamton, NY 13902

Mr. David F. Wilson  
R.E. Ginna Nuclear Power Plant  
1503 Lake Rd.  
Ontario, NY 14519

RG&E/NRC EQ MEETING  
ATTENDANCE LIST  
OCTOBER 15, 2002

<u>NAME</u>	<u>ORGANIZATION</u>
SABA SABA	NRC/NRR/DE/EEIB
RUSSELL ARRIGHI	NRC/NRR/DRIP/RLEP
AMAR N. PAL	NRC/NRR/DE/EEIB
PAUL SHEMANSKI	NRC/NRR/DE/EEIB
SAMSON LEE	NRC/NRR/DRIP/RLEP
NICK LOVE	RG&E
JARRED JACKSON	RG&E
GEORGE WROBEL	RG&E
RUSS WELLS	CNS
JIT VORA	NRC/RES/DET

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