| Material | | Environment | Aging Effect/ Mechanism | Program |
|-------------------|----------|--|-------------------------------|-----------------------------------|
| 1. Reactor | coola | nt pressure boundary component | S | |
| System No | 1005 | | | |
| Carbon Steel | | Containment Air, Borated Water Leakage | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| Cast Austenitic S | Stainles | Treated Water (including steam) | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| Nickel-based Allo | у | Containment Air | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| | | Containment Air, Borated Water Leakage | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| | | Treated Water (including steam) | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| Stainless Steel | | Containment Air, Borated Water Leakage | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| | | Treated Water (including steam) | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| System No | 1055 | Rx Vessel Level Instrument | ation / ICCM System | |
| Stainless Steel | | Treated Water (including steam) | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| System No | 2005 | Reactor Coolant System (RO | C) | |
| Carbon Steel | | Air and Gas | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| | | Containment Air, Borated Water Leakage | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| Clad Carbon Stee | el | Treated Water (including steam) | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| Stainless Steel | | Treated Water (including steam) | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| System No | 2045 | Residual Heat Removal Sys | tem (RHR) | |
| Stainless Steel | | Treated Water (including steam) | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| System No | 2060 | Chemical And Volume Con | trol System | |
| Stainless Steel | | Treated Water (including steam) | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| System No | 2080 | Safety Injection System | | |
| Stainless Steel | | Treated Water (including steam) | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |

| Material | | Environment | Aging Effect/ Mechanism | Program |
|-------------------|---------|--|--|---|
| 1. Reactor | · coola | nt pressure boundary components | s | |
| | | • | | |
| System No | 2115 | Primary Sampling System (F | | |
| Stainless Steel | | Treated Water (including steam) | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| System No | 3005 | Steam Generator | | |
| Carbon Steel | | Containment Air, Borated Water Leakage | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| | | Treated Water (including steam) | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| Nickel-based Allo | ру | Treated Water (including steam) | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| Stainless Steel | | Treated Water (including steam) | Cracking from Thermal Fatigue | Fatigue is a TLAA to be evaluated |
| | | tor shell assembly | | |
| System No | 3005 | Steam Generator | | |
| Carbon Steel | | Treated Water (including steam) | Loss of Material from Crevice Corrosion | ASME Section XI, Subsections IWB, IWC and IWD In |
| | | Treated Water (including steam) | Loss of Material from Crevice Corrosion | Water Chemistry Program |
| | | Treated Water (including steam) | Loss of Material from General Corrosion | ASME Section XI, Subsections IWB, IWC and IWD In |
| | | Treated Water (including steam) | Loss of Material from General Corrosion | Water Chemistry Program |
| | | Treated Water (including steam) | Loss of Material from Pitting Corrosion | ASME Section XI, Subsections IWB, IWC and IWD In |
| | | Treated Water (including steam) | Loss of Material from Pitting Corrosion | Water Chemistry Program |
| 3. Pressur | e vesse | el ferritic materials that have a ne | utron fluence greater than 1017 n/cm2 (E>1 M | eV) |
| System No | 1005 | Reactor Vessel And Internal | s System | |
| Carbon Steel | | Containment Air, Borated Water Leakage | Change in Material Properties from Irradiation Embrittlement | Neutron Irradiation Embrittlement is a TLAA to be eva |
| 4. Reactor | vesse | beltline shell and welds | | |
| System No | 1005 | Reactor Vessel And Internal | s System | |
| | | | | |

| Material | | Environment | Aging Effect/ Mechanism | Program |
|-------------------|----------|---------------------------------|---|------------------------------|
| 5. Westing | ghouse | and B&W baffle/former bolts | | |
| System No | 1005 | Reactor Vessel And Intern | als System | |
| Stainless Steel | | Treated Water (including steam) | Reduction of Fracture Toughness from Neutron Irradiation Embrittlement | PWR Vessel Internals Program |
| 6. Small-b | ore re | actor coolant system and connec | eted systems piping | |
| System No | 1005 | Reactor Vessel And Intern | als System | |
| Stainless Steel | | Treated Water (including steam) | Cracking from SCC | Water Chemistry Program |
| System No | 1055 | Rx Vessel Level Instrumer | ntation / ICCM System | |
| Stainless Steel | | Treated Water (including steam) | Cracking from SCC | Water Chemistry Program |
| System No | 2005 | Reactor Coolant System (F | RC) | |
| Stainless Steel | | Treated Water (including steam) | Cracking from SCC | Water Chemistry Program |
| System No | 2060 | Chemical And Volume Co | ntrol System | |
| Stainless Steel | | Treated Water (including steam) | Cracking from SCC | Water Chemistry Program |
| System No | 2115 | Primary Sampling System | (PS) | |
| Stainless Steel | | Treated Water (including steam) | Cracking from SCC | Water Chemistry Program |
| 8. Reactor | inter | nals | | |
| System No | 1005 | Reactor Vessel And Intern | als System | |
| Cast Austenitic S | Stainles | Treated Water (including steam) | Change in Dimensions from Void Swelling | PWR Vessel Internals Program |
| Nickel-based Allo | ру | Treated Water (including steam) | Change in Dimensions from Void Swelling | PWR Vessel Internals Program |
| Stainless Steel | | Treated Water (including steam) | Change in Dimensions from Void Swelling | PWR Vessel Internals Program |

| Material | Environment | Aging Effect/ Mechanism | Program |
|---|--|---|---|
| 10. Cast au | stenitic stainless steel (CASS) rea | ctor coolant system piping | |
| | | | |
| System No | • | | W - Q B |
| Stainless Steel | Treated Water (including steam) | Cracking from SCC | Water Chemistry Program |
| 12. Westing | ghouse and B&W baffle former b | olts | |
| System No | 1005 Reactor Vessel And I | nternals System | |
| Stainless Steel | Treated Water (including steam) | Cracking from IASCC | PWR Vessel Internals Program |
| | Treated Water (including steam) | Cracking from IASCC | Water Chemistry Program |
| | Treated Water (including steam) | Cracking from SCC | Water Chemistry Program |
| | Treated Water (including steam) | Cracking from SCC | PWR Vessel Internals Program |
| System No | ghouse and B&W baffle former b 1005 Reactor Vessel And l | | |
| | To a stand Martin of Construction on the sure | Language Day Land Communication Communication | A ONATE OF SCIENCE VI. ON IN A SCIENCE DAVID DAVID OF SCIENCE DAVID DAVID |
| Stainless Steel | Treated Water (including steam) | Loss of Pre-load from Irradiation Creep | |
| Stainless Steel | Treated Water (including steam) Treated Water (including steam) | Loss of Pre-load from Irradiation Creep Loss of Pre-load from Irradiation Creep | ASME Section XI, Subsections IWB, IWC and IWD In PWR Vessel Internals Program |
| | | Loss of Pre-load from Irradiation Creep | ASME Section XI, Subsections IWB, IWC and IWD II PWR Vessel Internals Program |
| 15. (Alloy 6 | Treated Water (including steam) | Loss of Pre-load from Irradiation Creep | |
| 15. (Alloy 6 | Treated Water (including steam) 500) Steam generator tubes, repair 3005 Steam Generator | Loss of Pre-load from Irradiation Creep | |
| 15. (Alloy 6 | Treated Water (including steam) 500) Steam generator tubes, repair 3005 Steam Generator | Loss of Pre-load from Irradiation Creep r sleeves, and plugs | PWR Vessel Internals Program |
| 15. (Alloy 6 | Treated Water (including steam) 500) Steam generator tubes, repair 3005 Steam Generator Treated Water (including steam) | r sleeves, and plugs Cracking from SCC | PWR Vessel Internals Program Water Chemistry Program |
| 15. (Alloy 6 | Treated Water (including steam) 500) Steam generator tubes, repair 3005 Steam Generator y Treated Water (including steam) Treated Water (including steam) | r sleeves, and plugs Cracking from SCC Cracking from SCC | PWR Vessel Internals Program Water Chemistry Program Steam Generator Tube Integrity Program |
| 15. (Alloy 6 | Treated Water (including steam) 500) Steam generator tubes, repair 3005 Steam Generator Treated Water (including steam) Treated Water (including steam) Treated Water (including steam) | r sleeves, and plugs Cracking from SCC Cracking from SCC Loss of Material from Crevice Corrosion | PWR Vessel Internals Program Water Chemistry Program Steam Generator Tube Integrity Program Steam Generator Tube Integrity Program |
| 15. (Alloy 6 System No Nickel-based Alloy | Treated Water (including steam) 500) Steam generator tubes, repair 3005 Steam Generator y Treated Water (including steam) Treated Water (including steam) Treated Water (including steam) Treated Water (including steam) Treated Water (including steam) | r sleeves, and plugs Cracking from SCC Cracking from SCC Loss of Material from Crevice Corrosion Loss of Material from Crevice Corrosion | Water Chemistry Program Steam Generator Tube Integrity Program Steam Generator Tube Integrity Program Water Chemistry Program |

| Material | | Environment | Aging Effect/ Mechanism | Program |
|-----------------------|---------|-------------------------------------|--|--|
| 15. (Alloy (| 600) Si | team generator tubes, repair sleeve | es, and plugs | |
| Nickel-based Allo | | Treated Water (including steam) | Loss of Material from Pitting Corrosion | Steam Generator Tube Integrity Program |
| 17. Carbon | ı steel | tube support plate | | |
| System No | 3005 | Steam Generator | | |
| Stainless Steel | | Treated Water (including steam) | Cracking from SCC | Steam Generator Tube Integrity Program |
| | | Treated Water (including steam) | Cracking from SCC | Water Chemistry Program |
| | | Treated Water (including steam) | Loss of Material from Crevice Corrosion | Water Chemistry Program |
| | | Treated Water (including steam) | Loss of Material from Crevice Corrosion | Steam Generator Tube Integrity Program |
| | | Treated Water (including steam) | Loss of Material from Erosion | Steam Generator Tube Integrity Program |
| | | Treated Water (including steam) | Loss of Material from Erosion | Water Chemistry Program |
| | | Treated Water (including steam) | Loss of Material from Pitting Corrosion | Water Chemistry Program |
| | | Treated Water (including steam) | Loss of Material from Pitting Corrosion | Steam Generator Tube Integrity Program |
| 19. CASS _I | pump | casing and valve body | | |
| System No | 2005 | Reactor Coolant System (RC |) | |
| Stainless Steel | | Treated Water (including steam) | Reduction of Fracture Toughness from Thermal Embrittlement | ASME Section XI, Subsections IWB, IWC and IWD In |
| System No | 2060 | Chemical And Volume Contr | rol System | |
| Stainless Steel | | Treated Water (including steam) | Reduction of Fracture Toughness from Thermal Embrittlement | Thermal Aging Embrittlement Of Cast Austenitic Stain |
| System No | 2080 | Safety Injection System | | |
| Stainless Steel | | Treated Water (including steam) | Reduction of Fracture Toughness from Thermal Embrittlement | Thermal Aging Embrittlement Of Cast Austenitic Stain |

Reactor Vessel, Internals, and Reactor Coolant

Containment Air, Borated Water Leakage

| Material | | Environment | Aging Effect/ Mechanism | Program |
|-----------------|--------|--|---|--|
| 20. CASS] | piping | | | |
| System No | 2005 | Reactor Coolant System (RC) | | |
| Stainless Steel | | Treated Water (including steam) | Reduction of Fracture Toughness from Thermal Embrittlement | Thermal Aging Embrittlement Of Cast Austenitic Stain |
| 21. BWR p | piping | and fittings; steam generator comp | onents | |
| System No | 3005 | Steam Generator | | |
| Carbon Steel | | Treated Water (including steam) | Loss of Material from FAC | Flow-Accelerated Corrosion Program |
| Carbon Steel | | Containment Air, Borated Water Leakage | Loss of Material from Wear | Bolting Integrity Program |
| System No | 2005 | Reactor Coolant System (RC) | | |
| Carbon Steel | | | | 0 0 , 0 |
| | | Containment Air, Borated Water Leakage | Loss of Material from Wear | ASME Section XI, Subsections IWB, IWC and IWD In |
| | | Containment Air, Borated Water Leakage | Loss of Pre-load from Stress Relaxation | Bolting Integrity Program |
| | | Containment Air, Borated Water Leakage | Loss of Pre-load from Stress Relaxation | Preventive Maintenance Program (Site Specific) |
| Stainless Steel | | Containment Air, Borated Water Leakage | Loss of Material from Wear | ASME Section XI, Subsections IWB, IWC and IWD In |
| | | Treated Water (including steam) | Loss of Material from Wear | Bolting Integrity Program |
| | | Treated Water (including steam) | Loss of Material from Wear | ASME Section XI, Subsections IWB, IWC and IWD In |
| System No | 2060 | Chemical And Volume Contro | ol System | |
| Stainless Steel | | Containment Air, Borated Water Leakage | Loss of Material from Wear | ASME Section XI, Subsections IWB, IWC and IWD In |
| | | Treated Water (including steam) | Loss of Material from Wear | ASME Section XI, Subsections IWB, IWC and IWD In |
| | | Treated Water (including steam) | Loss of Material from Wear | Bolting Integrity Program |
| System No | 2080 | Safety Injection System | | |

Stainless Steel

Loss of Material from Wear

ASME Section XI, Subsections IWB, IWC and IWD In

| Material | | Environment | Aging Effect/ Mechanism | Program |
|---------------------------|--------|---|--|--|
| | | nt pressure boundary (RCPB) ing in high pressure and high t | valve closure bolting, manway and hold emperature systems | ling bolting, |
| Stainless Steel | | Treated Water (including steam) | Loss of Material from Wear | Bolting Integrity Program |
| 23. CRD r | ozzle | | | |
| System No | 1005 | Reactor Vessel And Inte | rnals System | |
| Nickel-based Al | oy | Treated Water (including steam) | Cracking from SCC | Nickel-Alloy Nozzles And Penetrations Program |
| | | Treated Water (including steam) | Cracking from SCC | Water Chemistry Program |
| CASS and | boltin | ng) | ousing; reactor coolant system compone | nts (except |
| System No | 1005 | Reactor Vessel And Inte | rnals System | |
| Stainless Steel | | Treated Water (including steam) | Cracking from SCC | Water Chemistry Program |
| | | Treated Water (including steam) | Cracking from SCC | ASME Section XI, Subsections IWB, IWC and IWD In |
| System No | 2005 | Reactor Coolant System | (RC) | |
| Clad Carbon Ste | eel | Treated Water (including steam) | Cracking from SCC | ASME Section XI, Subsections IWB, IWC and IWD In |
| | | Treated Water (including steam) | Cracking from SCC | Water Chemistry Program |
| Stainless Steel | | Treated Water (including steam) | Cracking from SCC | Water Chemistry Program |
| | | Treated Water (including steam) | Cracking from SCC | ASME Section XI, Subsections IWB, IWC and IWD In |
| | 2045 | Residual Heat Removal | System (RHR) | |
| System No | | Treated Water (including steam) | Cracking from SCC | Water Chemistry Program |
| System No Stainless Steel | | | | |
| • | 2080 | Safety Injection System | | |

| Material | | Environment | Aging Effect/ Mechanism | Program |
|-----------------|---------|---|--|--|
| 26. Extern | al surf | aces of carbon steel components in r | reactor coolant system pressure boundary | |
| System No | 1005 | Reactor Vessel And Internals S | - | |
| Carbon Steel | 1002 | Containment Air, Borated Water Leakage | Loss of Material from Aggressive Chemical Attack | Boric Acid Corrosion Program |
| System No | 2005 | Reactor Coolant System (RC) | | |
| Carbon Steel | | Containment Air, Borated Water Leakage | Loss of Material from Aggressive Chemical Attack | Boric Acid Corrosion Program |
| | | Containment Air, Borated Water Leakage | Loss of Material from Aggressive Chemical Attack | ASME Section XI, Subsections IWB, IWC and IWD In |
| | | Containment Air, Borated Water Leakage | Loss of Mechanical Closure Integrity from Loss of Material due to Aggressive Chemical Attack | Boric Acid Corrosion Program |
| Clad Carbon Ste | el | Containment Air, Borated Water Leakage | Loss of Material from Aggressive Chemical Attack | Boric Acid Corrosion Program |
| | | Containment Air, Borated Water Leakage | Loss of Material from Aggressive Chemical Attack | ASME Section XI, Subsections IWB, IWC and IWD In |
| System No | 2045 | Residual Heat Removal System | n (RHR) | |
| Carbon Steel | | Indoor - Not Air Conditioned, Containment Air, Borated Water Leakage | Loss of Mechanical Closure Integrity from Loss of Material due to Aggressive Chemical Attack | Boric Acid Corrosion Program |
| System No | 2060 | Chemical And Volume Control | System | |
| Carbon Steel | | Indoor - Not Air Conditioned, Containment Air, Borated Water Leakage | Loss of Mechanical Closure Integrity from Loss of Material due to Aggressive Chemical Attack | Boric Acid Corrosion Program |
| System No | 2080 | Safety Injection System | | |
| Carbon Steel | | Indoor - Not Air Conditioned, Containment Air, Borated Water Leakage | Loss of Mechanical Closure Integrity from Loss of Material due to Aggressive Chemical Attack | Boric Acid Corrosion Program |
| System No | 2115 | Primary Sampling System (PS) | | |
| Carbon Steel | | Indoor - Not Air Conditioned, Containment Air, Borated Water Leakage | Loss of Mechanical Closure Integrity from Loss of Material due to Aggressive Chemical Attack | Boric Acid Corrosion Program |
| System No | 3005 | Steam Generator | | |
| Carbon Steel | | Containment Air, Borated Water Leakage | Loss of Material from Aggressive Chemical Attack | Boric Acid Corrosion Program |
| | | Containment Air, Borated Water Leakage | Loss of Mechanical Closure Integrity from Loss of Material due to Aggressive Chemical Attack | Boric Acid Corrosion Program |

| Material | Environment | Aging Effect/ Mechanism | Program |
|--|--|---|--|
| 26. External s | surfaces of carbon steel components | in reactor coolant system pressure boundary | |
| 28. Reactor in | iternals, reactor vessel closure studs | , and core support pads | |
| System No 1 | Reactor Vessel And Interna | als System | |
| Carbon Steel | Containment Air, Borated Water Leakage | Loss of Material from Wear | ASME Section XI, Subsections IWB, IWC and IWD II |
| Nickel-based Alloy | Treated Water (including steam) | Loss of Material from Wear | Flux Thimble Eddy Current Inspection Program |
| | Treated Water (including steam) | Loss of Material from Wear | ASME Section XI, Subsections IWB, IWC and IWD In |
| Stainless Steel | Treated Water (including steam) | Loss of Material from Wear | ASME Section XI, Subsections IWB, IWC and IWD Ir |
| System No. 1 | 005 Reactor Vessel And Interne | ale System | |
| baffle bolts) | | xcept Westinghouse and Babcock & Wilcox [Bo | - |
| System No 1 | Reactor Vessel And Interna | als System | |
| | 7005 Reactor Vessel And International Treated Water (including steam) | Als System Reduction of Fracture Toughness from Neutron Irradiation Embrittlement | PWR Vessel Internals Program |
| Nickel-based Alloy | | Reduction of Fracture Toughness from Neutron Irradiation | PWR Vessel Internals Program PWR Vessel Internals Program |
| Nickel-based Alloy Stainless Steel | Treated Water (including steam) Treated Water (including steam) | Reduction of Fracture Toughness from Neutron Irradiation Embrittlement Reduction of Fracture Toughness from Neutron Irradiation | ů, |
| Nickel-based Alloy Stainless Steel 32 Steam gen | Treated Water (including steam) Treated Water (including steam) | Reduction of Fracture Toughness from Neutron Irradiation Embrittlement Reduction of Fracture Toughness from Neutron Irradiation Embrittlement | ů, |
| Nickel-based Alloy Stainless Steel 32 Steam gen System No 3 | Treated Water (including steam) Treated Water (including steam) erator upper and lower heads; tubes | Reduction of Fracture Toughness from Neutron Irradiation Embrittlement Reduction of Fracture Toughness from Neutron Irradiation Embrittlement | PWR Vessel Internals Program |
| Nickel-based Alloy Stainless Steel 32 Steam gen System No 3 | Treated Water (including steam) Treated Water (including steam) erator upper and lower heads; tubes Steam Generator | Reduction of Fracture Toughness from Neutron Irradiation Embrittlement Reduction of Fracture Toughness from Neutron Irradiation Embrittlement sheets; primary nozzles and safe ends | PWR Vessel Internals Program |
| Nickel-based Alloy Stainless Steel 32 Steam gen System No 3 Stainless Steel | Treated Water (including steam) Treated Water (including steam) erator upper and lower heads; tuber Oos Steam Generator Treated Water (including steam) | Reduction of Fracture Toughness from Neutron Irradiation Embrittlement Reduction of Fracture Toughness from Neutron Irradiation Embrittlement Sheets; primary nozzles and safe ends Cracking from SCC Cracking from SCC | PWR Vessel Internals Program ASME Section XI, Subsections IWB, IWC and IWD In |
| Nickel-based Alloy Stainless Steel 32 Steam gen System No 3 Stainless Steel 33. Vessel int | Treated Water (including steam) Treated Water (including steam) Perator upper and lower heads; tuber Oos Steam Generator Treated Water (including steam) Treated Water (including steam) | Reduction of Fracture Toughness from Neutron Irradiation Embrittlement Reduction of Fracture Toughness from Neutron Irradiation Embrittlement Sheets; primary nozzles and safe ends Cracking from SCC Cracking from SCC Cracking from SCC &W baffle former bolts) | PWR Vessel Internals Program ASME Section XI, Subsections IWB, IWC and IWD In |
| Stainless Steel 32 Steam gen System No 3 Stainless Steel 33. Vessel int System No 1 | Treated Water (including steam) Treated Water (including steam) Perator upper and lower heads; tuber Treated Water (including steam) Treated Water (including steam) Treated Water (including steam) Pernals (except Westing-house and B The steam of | Reduction of Fracture Toughness from Neutron Irradiation Embrittlement Reduction of Fracture Toughness from Neutron Irradiation Embrittlement Sheets; primary nozzles and safe ends Cracking from SCC Cracking from SCC Cracking from SCC &W baffle former bolts) | PWR Vessel Internals Program ASME Section XI, Subsections IWB, IWC and IWD In |
| Nickel-based Alloy Stainless Steel 32 Steam gen System No 3 Stainless Steel 33. Vessel int | Treated Water (including steam) Treated Water (including steam) Perator upper and lower heads; tubes Treated Water (including steam) Treated Water (including steam) Treated Water (including steam) Pernals (except Westing-house and B Reactor Vessel And Internal | Reduction of Fracture Toughness from Neutron Irradiation Embrittlement Reduction of Fracture Toughness from Neutron Irradiation Embrittlement Sheets; primary nozzles and safe ends Cracking from SCC Cracking from SCC W baffle former bolts) als System | PWR Vessel Internals Program ASME Section XI, Subsections IWB, IWC and IWD II Water Chemistry Program |

| Material | Environment | Aging Effect/ Mechanism | Program |
|--------------------|--|----------------------------|------------------------------------|
| 33. Vessel inte | rnals (except Westing-house and l | B&W baffle former bolts) | |
| Nickel-based Alloy | Treated Water (including steam) | Cracking from IASCC | PWR Vessel Internals Program |
| | Treated Water (including steam) | Cracking from SCC | Water Chemistry Program |
| | Treated Water (including steam) | Cracking from SCC | PWR Vessel Internals Program |
| Stainless Steel | Treated Water (including steam) | Cracking from IASCC | PWR Vessel Internals Program |
| | Treated Water (including steam) | Cracking from IASCC | Water Chemistry Program |
| | Treated Water (including steam) | Cracking from SCC | PWR Vessel Internals Program |
| | Treated Water (including steam) | Cracking from SCC | Water Chemistry Program |
| 34. Reactor ve | ssel closure studs and stud assemb | bly | |
| System No 10 | 05 Reactor Vessel And Inter | nals System | |
| Carbon Steel | Containment Air, Borated Water Leakage | Loss of Material from Wear | Reactor Head Closure Studs Program |