

October 25, 2002

Mr. James Scarola, Vice President  
Shearon Harris Nuclear Power Plant  
Carolina Power & Light Company  
Post Office Box 165, Mail Code: Zone 1  
New Hill, North Carolina 27562-0165

SUBJECT: SHEARON HARRIS NUCLEAR POWER PLANT - REQUEST FOR ADDITIONAL  
INFORMATION (RAI) ON AMENDMENT APPLICATION REGARDING ADDITION  
OF METHODOLOGY REFERENCES TO THE CORE OPERATING LIMIT  
REPORT (TAC NO. MB6226)

Dear Mr. Scarola:

By letter dated August 28, 2002, you submitted an amendment application regarding the addition of methodology references to the core operating limit report for the Shearon Harris Nuclear Power Plant. The NRC staff reviewed the information that you provided in the subject amendment application and identified the need for further information as described in the enclosed RAI.

This request for information was discussed by the NRC staff, Framatome, and the licensee during a telephone conversation on October 7, 2002, and a mutually agreeable schedule for a response by November 26, 2002, was established for your comprehensive response to the RAI.

If you have any questions, please contact me on (301) 415-1478.

Sincerely,

*/RA/*

Ram Subbaratnam, Project Manager, Section 2  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-400

Enclosure: RAI

cc w/encl: See next page

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SHEARON HARRIS NUCLEAR POWER PLANT  
REQUEST FOR ADDITIONAL INFORMATION (RAI) ON AMENDMENT REQUEST  
REGARDING ADDITION OF METHODOLOGY REFERENCES TO THE CORE OPERATING  
LIMIT REPORT  
TAC NO. MB6226

Carolina Power & Light Company is requested to provide the following additional information relating to the above amendment request:

1. The NRC's safety evaluation of Topical Report EMF-2328(P)(A), Revision 0, "PWR Small Break LOCA Evaluation Model, S-RELAP5 Based" states a condition on the use of the ANF-RELAP code that still applies to the use of S-RELAP5. The statement is:

That while it has been shown in Reference 53 [Loomis, G.G., "Summary of the Semiscale Program (1965-1986)," NUREG/CR-4945, July 1987] that the thermal-hydraulic phenomena observed for breaks up to 10 percent of the cold leg flow area are the same, if the code is used for break sizes larger than 10 percent of the cold leg flow area additional assessments must be performed to ensure that the code is predicting the important phenomena which may occur.

Your submittal did not state the break size range for which you intend to use the code. Please provide information that says what break size range you intend to use the code for, and if you intend to use the code for breaks larger than 10 percent of the cold leg flow area, also provide the required additional assessments.

2. The NRC's safety evaluation of Topical Report EMF-2310(P)(A), Revision 0, "SRP Chapter 15 Non-LOCA Methodology for Pressurized Water Reactors" states in Section 5.0 "Evaluation of S-RELAP5":

The staff notes, however, that a generic topical report describing a code such as S-RELAP5 cannot provide full justification for each specific individual plant application. The individual applicant must still provide justification for the specific application of the code which is expected to include as a minimum, the nodalization, defense of the chosen parameters, any needed sensitivity studies, justification of the conservative nature of the input parameters, and calculated results.

Your submittal did not include any justification of your specific application of the code. Please provide the necessary information.

The delay with submitting this formal RAI was due to clarification of the time period needed by the licensee to submit a comprehensive response.

Mr. James Scarola  
Carolina Power & Light Company

Shearon Harris Nuclear Power Plant  
Unit 1

cc:

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