Duke Energy

Duke Energy Corporation
McGuire Nuclear Station
12700 Hagers Ferry Road
Huntersville, NC 28078-9340
(704) 875-4800 OFFICE
(704) 875-4809 FAX

H. B. Barron
Vice President

September 26, 2000

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: McGuire Nuclear Station
Docket Nos. 50-369, 50-370
Proposed Correction to License Amendment Nos. 195/176, Facility Operating Licenses NPF-9 and NPF-17, (TAC Nos. MA8696 and MA8697)

By letter dated September 22, 2000, the NRC issued approved License Amendment No. 195 to Facility Operating License NPF-9 and Amendment No. 176 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. The amendments revised the Technical Specifications and associated Bases to reference the Westinghouse Best Estimate Large Break Loss-of-Coolant Accident (LOCA) analysis methodology described in WCAP-12945-P-A, March 1998.

By letter dated September 22, 1999, the NRC issued approved License Amendment No. 188 to Facility Operating License NPF-9 and Amendment No. 169 to Facility Operating License NPF-17. The amendments revised various sections of the Technical Specifications to permit use of Westinghouse's Robust Fuel Assemblies for future core reloads.

During the implementation review of the approved changes, discrepancies were identified on page 5.6-4 of the Technical Specifications. It was discovered that changes previously issued in License Amendment 188/169 had been erroneously eliminated through an administrative error.

Attachment 1 contains a markup of page 5.6-4 from Amendment 188/169 and adds the reference to WCAP-12945-P-A from Amendment 195/176. Attachment 2 contains the corrected page containing both approvals to be implemented during Unit 2, Cycle 14.

A001

Rec'd 10/22/02

Questions regarding this submittal should be directed to Kay Crane, McGuire Regulatory Compliance at (704) 875-4306.

H. B. Barron, Vice President
McGuire Nuclear Station

cc: Mr. Frank Rinaldi, Project Manager
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. Luis Reyes, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
Atlanta Federal Center
61 Forsyth St., SW, Suite 23T85
Atlanta, GA 30323

Mr. Scott Shaeffer
Senior Resident Inspector
McGuire Nuclear Station

Attachment 1

5.6 Reporting Requirements

5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

4. DPC-NE-2011PA, "Duke Power Company Nuclear Design Methodology for Core Operating Limits of Westinghouse Reactors," March, 1990 (DPC Proprietary).

5. DPC-NE-3001PA, "Multidimensional Reactor Transients and Safety Analysis Physics Parameter Methodology," November, 1991 (DPC Proprietary).

6. DPC-NF-2010A, "Duke Power Company McGuire Nuclear Station Catawba Nuclear Station Nuclear Physics Methodology for Reload Design," June, 1985.

7. DPC-NE-3002A, Rev. 3 "FSAR Chapter 15 System Transient Analysis Methodology," SER dated February 5, 1999.

8. DPC-NE-3000PA, Rev. 2 "Thermal-Hydraulic Transient Analysis Methodology," SER dated October 14, 1998. (DPC Proprietary).

9. DPC-NE-1004A, Rev. 1, "Nuclear Design Methodology Using CASMO-3/SIMULATE-3P," SER dated April 26, 1996.

10. DPC-NE-2004P-A, Rev. 1, "Duke Power Company McGuire and Catawba Nuclear Stations Core Thermal-Hydraulic Methodology using VIPRE-01," SER dated February 20, 1997 (DPC Proprietary).

11. DPC-NE-2005P-A, Rev. 1, "Thermal Hydraulic Statistical Core Design Methodology," SER dated November 7, 1996 (DPC Proprietary).

12. DPC-NE-2008P-A, "Fuel Mechanical Reload Analysis Methodology Using TACO3," SER dated April 3, 1995 (DPC Proprietary).

13. WCAP-10054-P-A, "Westinghouse Small Break ECCS Evaluation Model using the NOTRUMP Code," August 1985 (W Proprietary).

14. DPC-NE-2009-P-A, "Westinghouse Fuel Transition Report, "SER dated September 22, 1999 (DPC Proprietary).

15. WCAP-12945-P-A, Volume 1 (Revision 2) and Volumes 2-5 (Revision 1), "Code Qualification Document for Best-Estimate Loss of Coolant Analysis," March 1998, (W Proprietary). (continued)

Attachment 2

5.6 Reporting Requirements

5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

4. DPC-NE-2011PA, "Duke Power Company Nuclear Design Methodology for Core Operating Limits of Westinghouse Reactors," March, 1990 (DPC Proprietary).

5. DPC-NE-3001PA, "Multidimensional Reactor Transients and Safety Analysis Physics Parameter Methodology," November, 1991 (DPC Proprietary).

6. DPC-NF-2010A, "Duke Power Company McGuire Nuclear Station Catawba Nuclear Station Nuclear Physics Methodology for Reload Design," June, 1985.

7. DPC-NE-3002A, Rev. 3, "FSAR Chapter 15 System Transient Analysis Methodology," SER dated February 5, 1999.

8. DPC-NE-3000PA, Rev. 2 "Thermal-Hydraulic Transient Analysis Methodology," SER dated October 14, 1998. (DPC Proprietary).

9. DPC-NE-1004A, Rev. 1, "Nuclear Design Methodology Using CASMO-3/SIMULATE-3P," SER dated April 26, 1996.

10. DPC-NE-2004P-A, Rev. 1, "Duke Power Company McGuire and Catawba Nuclear Stations Core Thermal-Hydraulic Methodology using VIPRE-01," SER dated February 20, 1997 (DPC Proprietary).

11. DPC-NE-2005P-A, Rev. 1, "Thermal Hydraulic Statistical Core Design Methodology," SER dated November 7, 1996 (DPC Proprietary).

12. DPC-NE-2008P-A, "Fuel Mechanical Reload Analysis Methodology Using TACO3," SER dated April 3, 1995 (DPC Proprietary).

13. WCAP-10054-P-A, "Westinghouse Small Break ECCS Evaluation Model using the NOTRUMP Code, " August 1985 (W Proprietary).

14. DPC-NE-2009-P-A, "Westinghouse Fuel Transition Report, "SER dated September 22, 1999 (DPC Proprietary).

15. WCAP-12945-P-A, Volume 1 (Revision 2) and Volumes 2-5 (Revision 1), "Code Qualification Document for Best-Estimate Loss of Coolant Analysis," March 1998, (W Proprietary).

(continued)