



444 South 16th Street Mall  
Omaha NE 68102-2247

October 4, 2002  
LIC-02-0115

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Mail Station P1-137  
Washington, DC 20555

Reference: Docket No. 50-285

**Subject: Licensee Event Report 2002-S01 Revision 0 for the Fort Calhoun Station**

Please find attached Licensee Event Report 2002-S01, Revision 0, dated October 4, 2002. This report is being submitted pursuant to 10 CFR 73.71(a)(4). **This document does not contain any safeguards material.** No commitments are made to the NRC in this letter.

If you should have any questions, please contact me.

Sincerely,

D. J. Bannister  
Manager,  
Fort Calhoun Station

DJB/EPM/epm

Attachment

- c: E. W. Merschoff, NRC Regional Administrator, Region IV  
A. B. Wang, NRC Project Manager  
J. G. Kramer, NRC Senior Resident Inspector  
INPO Records Center  
Winston and Strawn

IE74

**LICENSEE EVENT REPORT (LER)**

(See reverse for required number of digits/characters for each block)

Estimated burden per response to comply with this mandatory information collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records Management Branch (T-6 E6), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to [bjst1@nrc.gov](mailto:bjst1@nrc.gov), and to the Desk Officer, Office of Information and Regulatory Affairs, NEOF-10202 (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

**1. FACILITY NAME**

Fort Calhoun Nuclear Station Unit Number 1

**2. DOCKET NUMBER**

05000285

**3. PAGE**

1 OF 2

**4. TITLE**

Possible Error in Classification of Safeguards Material

**5. EVENT DATE**

MO	DAY	YEAR
09	06	2002

**6. LER NUMBER**

YEAR	SEQUENTIAL NUMBER	REV NO
2002	S01	00

**7. REPORT DATE**

MO	DAY	YEAR
10	04	2002

**8. OTHER FACILITIES INVOLVED**

FACILITY NAME	DOCKET NUMBER
	05000
FACILITY NAME	DOCKET NUMBER
	05000

**9. OPERATING MODE**

1

**10. POWER LEVEL**

100

**11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)**

<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)
<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)
<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)(A)	<input checked="" type="checkbox"/> 73.71(a)(4)
<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(5)
<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	OTHER Specify in Abstract below or in NRC Form 366A
<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	
<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	
<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(vii)	
<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	
<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	

**12. LICENSEE CONTACT FOR THIS LER**

**NAME**

Erick Matzke, Licensing Engineer

**TELEPHONE NUMBER (Include Area Code)**

(402) 533-6855

**13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT**

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

**14. SUPPLEMENTAL REPORT EXPECTED**

YES (If yes, complete EXPECTED SUBMISSION DATE).  NO

**15. EXPECTED SUBMISSION DATE**

MONTH	DAY	YEAR

**16. ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)**

On September 6, 2002, following a review of the approved Abnormal Operating Procedure (AOP) -37, "Security Events," NRC Region IV staff raised an issue regarding information in the procedure. The NRC Staff informed OPPD management that they considered information contained in AOP-37 as Safeguards Information (SGI). The NRC Staff has discussed that the compilation of this type of information into one procedure is what drives them to the conclusion that AOP-37 is SGI.

As a conservative measure, Fort Calhoun Station personnel took immediate action to ensure that the procedure was controlled as SGI, and made a 10 CFR 73.71 report to the NRC on September 6, 2002.

The Omaha Public Power District (OPPD) is discussing this issue with NRC Region IV Staff. It is OPPD's understanding that the NRC regional staff is still determining the correct classification of the document. OPPD will update or cancel this LER as appropriate when the classification of this document has been resolved.

LICENSEE EVENT REPORT (LER)

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE	
Fort Calhoun Nuclear Station Unit Number 1	05000285	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2	OF 2
		2002	- S01	- 00		

17. NARRATIVE (If more space is required, use additional copies of NRC Form 366A)

BACKGROUND

In November 2001, FCS personnel reviewed and approved Abnormal Operating Procedure (AOP) -37, "Security Events." As part of the review process used to approve the procedure, the Omaha Public Power District (OPPD) reviewed AOP-37 using an information classification procedure for security issues. This classification procedure, Nuclear Operations Division (NOD) Quality Procedure (QP) -14, "Protection of Safeguards Information," provides guidance to determine the classification of documents that may contain safeguards information. Key provisions of NOD-QP-14 were generated based upon information in a July 23, 1991 letter from the NRC Director of NRR to the NRC Region Administrators.

During a security event, operations personnel and security officers must communicate effectively. A procedure (AOP-37) needed to be developed that would provide a tie for operations (that use room numbers) to the security officers (that use door numbers) to effectively communicate plant locations to one another. OPPD fashioned AOP-37 to be useful to the operators during potential security events without creating the need to employ safeguards controls to operate the plant

EVENT DESCRIPTION

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As a conservative measure, FCS personnel took immediate action to ensure that the procedure was controlled as SGI, and made a 10 CFR 73.71 report to the NRC on September 6, 2002.

CONCLUSION

OPPD is currently discussing this issue with NRC Region IV Staff. It is OPPD's understanding that the NRC regional staff is still determining the correct classification of the document. OPPD will update or cancel this LER as appropriate when the classification of this document has been resolved.