November 7, 2002

Mr. Thomas Coutu
Site Vice President and Interim Plant Manager
Kewaunee Nuclear Power Plant
Nuclear Management Company, LLC
N490 State Highway 42
Kewaunee, WI 54216

SUBJECT: KEWAUNEE NUCLEAR POWER PLANT - ISSUANCE OF AMENDMENT (TAC NO. MB5624)

Dear Mr. Coutu:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 164 to Facility Operating License No. DPR-43 for the Kewaunee Nuclear Power Plant (KNPP). This amendment revises the Technical Specifications (TS) in response to your application dated July 12, 2002.

The amendment revises the KNPP TS 3.1.a.3, " Pressurizer Safety Valves" to make it consistent with the Improved Standard Technical Specification to improve clarity. The amendment allows both pressurizer safety valves to be inoperable or removed while the reactor vessel head is on, provided the reactor coolant system (RCS) cold legs temperature is below 200 degrees F, which is in MODE 5 configuration. During MODE 5 configuration, the low temperature over pressure protection system is available and operable to protect the RCS from overpressure.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

/RA/

John G. Lamb, Project Manager, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-305

Enclosures: 1. Amendment No. 164 to License No. DPR-43
2. Safety Evaluation

cc w/encls: See next page

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DISTRIBUTION:
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PD 3-1 r/f ACRS
OGC GGrant, RIII

**No legal objection
ACCESSION NO.: ML022840430 *See previous concurrence

OFFICE	PM:PD3-1	LA:PD3-1	SC:SRXB	OGC/NLO**	SC:PD3-1
NAME	JLamb	THarris	FAkstulewicz*	SUttal	LRaghavan
DATE	10/17/02	10/17/02	10/17/02	10/29/02	10/29/02

OFFICIAL RECORD COPY

NUCLEAR MANAGEMENT COMPANY, LLC

DOCKET NO. 50-305

KEWAUNEE NUCLEAR POWER PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 164
License No. DPR-43

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Nuclear Management Company, LLC (NMC or the licensee), dated July 12, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-43 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 164 , are hereby incorporated in the license. The licensees shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance, and is to be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

L. Raghavan, Chief, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: November 7, 2002

ATTACHMENT TO LICENSE AMENDMENT NO. 164

FACILITY OPERATING LICENSE NO. DPR-43

DOCKET NO. 50-305

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contains marginal lines indicating the areas of change.

REMOVE	INSERT
Table of Contents, TS i	Table of Contents, TS i
TS 3.1-2	TS 3.1-2
TS 3.1-3	TS 3.1-3
TS 3.1-4	TS 3.1-4
TS 3.1-5	TS 3.1-5
TS 3.1-6	TS 3.1-6
TS 3.1-7	TS 3.1-7
TS 3.1-8	TS 3.1-8
TS 3.1-9	TS 3.1-9
TS 3.1-10	TS 3.1-10
TS B3.1-2	TS B3.1-2

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATING TO AMENDMENT NO. 164 TO FACILITY OPERATING LICENSE NO. DPR-43

NUCLEAR MANAGEMENT COMPANY, LLC

KEWAUNEE NUCLEAR POWER PLANT

DOCKET NO. 50-305

1.0 INTRODUCTION

In a letter dated July 12, 2002, Nuclear Management Company (NMC or the licensse) requested a license amendment to revise Kewaunee Nuclear Power Plant (KNPP) Technical Specification (TS) 3.1.a.3, " Pressurizer Safety Valves." The proposed change will make the current TS consistent with the Improved Standard Technical Specification (ISTS) to improve clarity. The proposal will allow both pressurizer safety valves to be inoperable or removed while the reactor vessel head is on, provided the reactor coolant system (RCS) cold legs temperature is below 200 degrees Fahrenheit, which is in MODE 5 configuration. During MODE 5 configuration, the low temperature overpressure protection (LTOP) system is available and operable to protect the RCS from overpressure. Westinghouse NUREG - 1431, Revision 1 ISTS limiting condition of operation (LCO) 3.4.10 is only applicable in MODES 1, 2, 3, 4, and does not apply while the system is in MODE 5 configuration. KNPP has custom TS and this proposed change will improve clarity in this TS.

2.0 BACKGROUND

The Commission's regulatory requirements related to the contents of TS are set forth in 10 CFR 50.36 which ensures that the TS specified limiting conditions for operations are consistent with assumed values of the initial conditions in the licensee's safety analyses. In accordance with 10 CFR 50.36, the U.S. Nuclear Regulatory Commission (NRC) staff and the nuclear steam system supplier Owner's groups developed ISTS, which meets 10 CFR 50.36 (c)(2) ii and 10 CFR 50.36 (c)(3) requirements. Westinghouse Owners Group NUREG-1431, Revision 1, is the staff guidance to effectively implement 10 CFR 50.36 provided no unique plant design that changes requirements. The licensee is using the guidance from the staff approved NUREG-1431, Revision 1 and the guidance from NUREG-0800, Standard Review Plan, Section 5.2.2 requirements as appropriate for their plant.

3.0 REGULATORY EVALUATION

The acceptance criteria for the overpressure protection system are based on meeting the relevant requirements of the following regulations:

1. General Design Criterion (GDC) 15, as it relates to the design of RCS and associated auxiliary, control, and protection systems with sufficient margin to assure that the design

conditions of the reactor coolant pressure boundary are not exceeded during any condition of normal operation, including anticipated operational occurrences.

2. GDC 31, as it relates to the design of reactor coolant pressure boundary with sufficient margin to assure that it behaves in a nonbrittle manner and that the probability of rapidly propagating fracture is minimized.

The LTOP system shall be designed in accordance with the requirements of the NRC's Branch Technical Position RSB 5-2, "Overpressurization Protection of Pressurized Water Reactors While Operating At Low Temperature." The LTOP system shall be operable during startup and shutdown conditions below the enable temperature as defined in paragraph B.2 of RSB 5-2.

4.0 TECHNICAL EVALUATION

The pressurizer safety valves provide, in conjunction with the reactor protection system, overpressure protection for the RCS. The pressurizer safety valves are totally enclosed pop type, spring-loaded, self-actuated valves with backpressure compensation. The safety valves are designed to prevent the system pressure from exceeding the system safety limit of 2735 psig, which is 110 percent of the design pressure. The pressurizer safety valves are part of the primary success path and mitigate the effects of postulated accidents. Operability of the safety valves ensures that the RCS pressure will be limited to 110 percent of design pressure.

The licensee proposes to revise their current TS 3.1.a.3 to make it similar to Westinghouse NUREG-1431, Revision 1, ISTS LCO 3.4.10. LCO 3.4.10 requires that pressurizer safety valves should be available in MODES 1, 2, 3 and 4 provided any cold legs temperature is equal to or greater than 275 degrees F. In MODE 5 (less than or equal to 200 degrees F), cold leg temperature is less than 275 degrees F and LTOP system provides the overpressure protection. Therefore, in Mode 5, pressurizer safety valves are not required. This proposal will allow both pressurizer safety valves to be inoperable or removed while the reactor vessel head is on. This proposal would be only applicable to TS 3.1.a.3 when the temperature and pressures are low enough that the LTOP system is available and operable below cold legs temperature of less than 200 degrees Farenheit. Under this plant configuration, the plant is in MODE 5 condition and pressurizer safety valves are not required. The proposal meets the intent of the ISTS LCO 3.4.10.

The NRC staff has reviewed this proposal to improve the clarity for TS 3.1.a.3, "Pressurizer Safety Valves." A LCO will be added to allow both pressurizer safety valves to be inoperable or removed provided that the LTOP system is in-service. At temperatures and pressure when the LTOP system is required, the pressurizer safeties no longer provide RCS overpressure protection. The LTOP system protects the RCS from non-ductile failure during low temperature overpressure transients. During low temperature conditions, the pressurizer safeties are not credited with protecting the RCS pressure boundary. Thus, the licensee proposal meets the regulatory requirements, and, we find this proposal acceptable.

In addition, the licensee proposes administrative changes to use Mircosoft Word for TS Section 3.1, renumber pages based on the word processing and the proposed revision to TS 3.1.a.3 as discussed above, and make administrative changes to TS basis page TS B3.1-2. The NRC staff has reviewed the administrative changes and finds the administrative changes acceptable. The NRC staff has reviewed the use of Microsoft Word and the NRC staff finds no objection.

The NRC staff has reviewed the TS basis changes and the NRC staff finds no objection to the TS basis changes.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Wisconsin State official was notified of the proposed issuance of the amendment. The State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluent that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding (67 FR 50957). Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

7.0 CONCLUSION

The NRC staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: K. Desai

Date: November 7, 2002

Kewaunee Nuclear Power Plant

cc:

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