

- (3) Intentionally Deleted.
- (4) The licensee shall implement and maintain in effect all provisions of the approved fire protection program, as described in the Fire Protection Plan for Rancho Seco Nuclear Generating Station, and as approved in the SER dated September 10, 1990 subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

- (5) The licensee shall perform the supplementary inservice tests and inspections of Nuclear Class 2 and Class 3 systems and components committed to in the letter of February 27, 1978, from J. J. Mattimoe to R. W. Reid.
- (6) In accordance with the provisions of 10 CFR 50.12(a) an exemption from the provisions of 10 CFR 50.55a(g)(4)(v) is granted, and the effective date for the start of the next 40-month period is extended from August 18, 1978 to October 18, 1979.

(7) Systems Integrity

The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

1. Provisions establishing preventive maintenance and periodic visual inspection requirements, and
 2. Integrated leak test requirements for each system at a frequency not to exceed refueling cycle intervals.
- (8) Intentionally Deleted.
- (9) Intentionally Deleted.

DEFINITIONS (Continued)

D1.9 LICENSEE EVENT REPORT

Defined under Administrative Controls Specification D6.9.5.

D1.10 RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM (REMP) MANUAL

The RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM (REMP) MANUAL shall contain a description of the Rancho Seco radiological environmental monitoring program. The REMP MANUAL shall also contain the REMP requirements of Specification D6.8.3b, a description of the environmental samples to be collected, sample locations, sampling frequencies, sample analysis criteria, and a description of the information to be included in the Annual Radiological Environmental Operating Report as required by Specification D6.9.2.3.

D1.11 NUCLEAR SAFETY

NUCLEAR SAFETY shall refer to those systems, components and administrative controls that have or may have an effect on the health and safety of the general public.

D1.12 INDUSTRIAL AREA

The INDUSTRIAL AREA shall be that portion of the site property, the access to which is controlled by security fencing, equipment, and personnel.

D1.13 UNRESTRICTED AREA

An UNRESTRICTED AREA shall be any area at or beyond the INDUSTRIAL AREA, access to which is not controlled by the licensee for purposes of protection of individuals from exposure to radiation and radioactive materials, or any area within the SITE BOUNDARY used for residential, industrial, commercial, institutional and/or recreational purposes.

TABLE D1-1 SURVEILLANCE INTERVALS*

Frequency	Notation	Definition
SHIFTLY	S	At least once per 12 hours.
DAILY	D	At least once per 24 hours.
WEEKLY	W	At least once per 7 days.
MONTHLY	M	At least once per 31 days.
QUARTERLY	Q	At least once per 92 days.
6 MONTHS	SY	At least once per 184 days.
ANNUALLY	A	At least once per 12 months.
18 MONTHS	18M	At least once per 18 months.

* See Specification D4.0.2.

ADMINISTRATIVE CONTROLS (Continued)

RESPONSIBILITIES

D6.5.1.6 The PRC shall be responsible for review of:

- a. The required safety evaluation of: (1) all procedures and programs required by Specification D6.8 and changes thereto, and (2) any other proposed procedures or programs or changes thereto which are as determined by the Plant Manager to affect NUCLEAR SAFETY.
- b. The safety evaluations of proposed tests and experiments that affect NUCLEAR SAFETY.
- c. Proposed changes to the Technical Specifications or the Facility Operating License (Possession Only).
- d. All safety evaluations of proposed changes or modifications to plant systems or equipment that affect NUCLEAR SAFETY. Items which are determined by a qualified reviewer as not involving an unreviewed safety question, a change of Technical Specifications, or a change in a licensing basis document need not be reviewed by the PRC.
- e. Investigations of all violations of the Technical Specifications to determine adequacy of corrective action and to detect any degrading trend.
- f. Facility operations to detect potential safety hazards.
- g. Events requiring a Licensee Event Report as defined by 10 CFR 50.73 and NUREG-1022 to determine adequacy of corrective action and to detect any degrading trend. (See Specification D6.9.5.)
- h. Special investigations and reports thereon as requested by the Plant Manager.
- i. (Intentionally deleted)
- j. The Emergency Plan and changes thereto.
- k. The Fire Protection Plan Program and changes thereto.
- l. Changes to the PROCESS CONTROL PROGRAM, the OFF-SITE DOSE CALCULATION MANUAL and the RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM MANUAL. (See Specifications D6.13 and D6.14.)
- m. Major changes to the Radioactive Waste Treatment Systems (Liquid, Gaseous and Solid).
- n. Review of any accidental, unplanned, or uncontrolled release of radioactive material to the environs, including the preparation and forwarding of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence. The PRC shall forward these reports to the Plant Manager and the MSRC.

ADMINISTRATIVE CONTROLS (Continued)

D6.5.3 TECHNICAL REVIEW AND CONTROL

Activities which affect NUCLEAR SAFETY shall be conducted as follows:

- a. Procedures, plans, manuals, and programs required by Specification D6.8 and other procedures, plans, manuals, and programs which affect plant NUCLEAR SAFETY, and changes thereto, shall be prepared, reviewed and approved. Each such procedure, plan, manual, and program or change thereto shall be reviewed by an individual(s) other than the preparer, but who may be from the same organization as the preparer of the procedure, plan, manual and program or change thereto. Programs, plans, manuals, and procedures other than plant administrative procedures will be approved as delineated in writing by the Plant Manager, but not lower than a direct report to the Plant Manager. Such procedures, plans, manuals and programs shall be reviewed periodically in accordance with administrative procedures.

The Plant Manager will approve plant administrative procedures and Emergency Plan Implementing Procedures.

Approval of temporary procedure changes which clearly do not change the intent of the approved procedure can be made by two members of the plant management staff, at least one of whom is a Certified Fuel Handler. The change shall be documented, reviewed and approved by the procedure's approval authority within 14 days of implementation.

- b. Proposed changes or modifications to plant systems or equipment that affect NUCLEAR SAFETY shall be reviewed by an individual(s) other than the individual(s) who designed the modification, but who may be from the same organization as the individual(s) who designed the modifications. Such modifications shall be approved by the Plant Manager or his designee as delineated in writing, but not lower than a direct report to the Plant Manager.
- c. Proposed tests and experiments which affect plant NUCLEAR SAFETY and are not addressed in the DSAR shall be reviewed by an individual(s) other than the individual(s) who prepared the proposed test or experiment. Such tests or experiments shall be approved by the Plant Manager or his designee as delineated in writing, but not lower than a direct report to the Plant Manager.
- d. Individuals responsible for reviews performed in accordance with Specification D6.5.3a, b and c shall meet or exceed the qualification requirements of Section 4, "Qualifications," of ANSI N18.1-1971, excluding Sections 4.5.2, "Technicians," and 4.5.3, "Repairmen." Each such review shall include a determination of whether or not additional, cross-disciplinary review is necessary. If deemed necessary, such a review shall be performed by the review personnel of the appropriate discipline. A list of qualified reviewers for the independent reviews described in D6.5.3a, b and c above shall be established by the Plant Manager.

ADMINISTRATIVE CONTROLS (Continued)

TECHNICAL REVIEW AND CONTROL (Continued)

- e. Events reportable pursuant to Specification D6.9.5 and violations of Technical Specifications shall be investigated and a report prepared which evaluates the event and which provides recommendations to prevent recurrence. Such reports shall be reviewed by the PRC and forwarded to the Plant Manager and the MSRC Chairman.

D6.5.4 AUDITS

Audits of facility activities shall be performed under the cognizance of the Quality organization supervisor and in accordance with the Rancho Seco Quality Manual (RSQM). These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions.
- b. The performance and qualifications of the entire Rancho Seco facility technical staff.
- c. The result of actions taken to correct deficiencies occurring in facility equipment, structures, systems or methods of operation that affect NUCLEAR SAFETY for those changes not previously audited.
- d. The performance of activities required by the Quality Assurance Program to meet the criteria of 10 CFR 50, Appendix B.
- e. The Facility Emergency Plan and implementing procedures.
- f. (Intentionally deleted)
- g. Any other area of facility operation considered appropriate by the Plant Manager or the Deputy AGM, Operations.
- h. Compliance with fire protection requirements and implementing procedures.
- i. An independent fire protection and loss prevention inspection and audit shall be performed using either qualified off-site licensee personnel or an outside fire protection firm.
- j. The independent fire protection and loss prevention program inspection and audit shall be performed by an outside, qualified fire protection consultant at least once every 3 years.
- k. The Radiological Environmental Monitoring Program (REMP) and the results thereof.
- l. The ODCM and implementing procedures.
- m. The PCP and implementing procedures for processing and packaging of radioactive wastes from liquid systems.

ADMINISTRATIVE CONTROLS (Continued)

AUDITS (Continued)

- n. The performance of activities required by the Quality Assurance Program for Effluent Control and Environmental Monitoring.

Audit reports of reviews encompassed by Specification D6.5.4 shall be forwarded to the Deputy AGM, Operations, the Plant Manager, the MSRC Chairman, and the management positions responsible for the areas reviewed within 30 days after completion.

The required frequencies for performance of the audits listed above are specified in the RSQM. Changes to these audit frequencies are subject to the requirements in 10 CFR 50.54(a).

D6.6 LICENSEE EVENT REPORT ACTION

The following actions shall be taken for events which are reportable as LICENSEE EVENT REPORTS:

- a. The Commission shall be notified and a report submitted per Specification D6.9.5, pursuant to the requirements of 10 CFR 20.2203, 50.73, and
- b. Each LICENSEE EVENT REPORT shall be reviewed by the PRC and the results of this review submitted to the MSRC. Each LICENSEE EVENT REPORT shall be reviewed and approved by the Plant Manager or designee.

D6.7 -NOT USED-

D6.8 PROCEDURES, PLANS, MANUALS, AND PROGRAMS

D6.8.1 Written procedures, plans, manuals, and programs shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Safety Guide 33, November 1972
- b. Irradiated Core Component Storage, Fuel Movement Operations, Decommissioning, and Decontamination
- c. Surveillance and test activities on equipment required for long-term safe storage of irradiated core components
- d. (Intentionally deleted)
- e. Emergency Plan implementation
- f. Fire Protection Program Plan implementation
- g. PCP implementation
- h. ODCM implementation
- i. REMP MANUAL implementation
- j. Quality Assurance Program for Effluent Control and Environmental Monitoring using the guidance of Regulatory Guide 4.15, Revision 1, February 1979
- k. Certified Fuel Handler Training Program implementation
- l. Quality Assurance Program implementation
- m. Radiation Protection Program implementation

ADMINISTRATIVE CONTROLS (Continued)

D6.9.3 ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

The Annual Radioactive Effluent Release Report covering the operation of the unit during the previous 12 months shall be submitted within 60 days after January 1 of each year. The report shall include a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the unit. The material provided shall be (1) consistent with the objectives outlined in the ODCM and PCP and (2) in conformance with 10 CFR 50.36a and Section IV.B.1 of Appendix I to 10 CFR 50.

D6.9.4 ANNUAL REPORT

A routine report consisting of shutdown statistics, a narrative summary of shutdown experience, major maintenance of structures, systems, and components required for long-term safe storage of irradiated fuel assemblies, and tabulations of facility changes, tests or experiments required pursuant to 10 CFR 50.59(b) shall be submitted on an annual basis to the U. S. Nuclear Regulatory Commission, Document Control Desk, Washington, D.C. 20555, with a copy to the Regional Office, postmarked no later than 30 days following the twelve month period covered by the report.

D6.9.5 LICENSEE EVENT REPORT

The types of events listed in 10 CFR 20.2203 and 50.73 shall be the subject of LICENSEE EVENT REPORTS, submitted to the U.S. Nuclear Regulatory Commission (NRC), Document Control Desk, Washington, D.C. 20555, within the time requirements of 10 CFR 20.2203, 50.73 and 73.71. An additional copy shall also be submitted to the Regional Administrator of the Region IV Office. The written report shall include a completed copy of a LICENSEE EVENT REPORT form, pursuant to 10 CFR 20.2203, 50.73 and 73.71 and the guidance of NUREG-1022, and a description of corrective actions and measures to prevent recurrence. Supplemental reports may be required to fully describe final resolution of the occurrence. For corrected or supplemental reports, a LICENSEE EVENT REPORT shall be completed and reference shall be made to the original report date, pursuant to the requirements of 10 CFR 20.2203, 50.73 or 73.71 as appropriate.

D6.9.6 ENVIRONMENTAL REPORTS

- a. When a change to the plant design or to the plant operation is planned which would have a significant adverse effect on the environment or which involves an environmental matter or question not previously reviewed and evaluated by the NRC, a report on the change will be made to the NRC prior to implementation. The report will include a description and evaluation of the change, including a supporting benefit-cost analysis.