FINAL AS-ADMINISTERED SCENARIOS

FOR THE DRESDEN INITIAL EXAMINATION - JUNE 2002



Dresden Generating Station

SIMULATOR EXERCISE GUIDE

ILT 01-1 NRC EXAM

SCENARIO

ILT-N-1

Rev. 01

05/02

DEVELOPED BY:

Exam Author

Date

 $\widetilde{V} \sim$

APPROVED BY:

Facility Representative

Date

Scenario Outline

Form ES-D-1

Facility:	Dresden		Scer	nario No:	<u>ILT-N-1</u>	Op-Test No: <u>ILT 01-1</u>		
Examin	ers:				Operators:			
	. <u> </u>				_			
<u>Initial Co</u> Unit 3 is	onditions: ~33% in Mode 4.	% rea	ctor powe	er; IRM ch	nannel 16 out of	service; 2B EHC Pump OOS;		
Turnove	er: Unit startup	in pro	ogress; ra	ise recirc	flow, then trans	fer FWLC to 3-element control.		
Event No.	Malf. No.	-	Event Гуре*		Event Description			
1	N/A	R	NSO SRO	raise rea	raise reactor power recirculation flow			
2	N/A	N	NSO SRO	transfer FWLC to 3-element control				
3	EHD626	I	ANSO SRO	spurious opening of main turbine bypass valve				
4	NIA5POT	I	NSO	ARPM c	hannel 5 fails de	ownscale		
5	NII15POT B15	с	NSO SRO	ARPM channel 5 companion IRM 15 spike upscale causing partial half-scram				
6	K49 T12	С	ANSO SRO	main feed breaker to Bus 23-1 trips with failure of emergency diesel generator to start automatically				
7	B12 AW4	М	ALL	failure o	f RPS to deener	gize and ARI to initiate		
8	H31, H32, H33, H44	IVI	ANSO SRO	trip of all reactor feed pumps				
' (N)orm	al, (R)eactivity	, (l)r	strument	, (C)om	oonent, (M)ajor			

_						
			••	_		
· · · ·			1 C A A			
- A T		nnc	IIV.	1 1		
		71 D.				
	~~~					

		the second se					
Op-Test No Event Desc	o: <u>ILT 01-1</u> cription: The t	Scenario No.: <u>ILT-N-1</u> Event No.: <u>1</u> Page <u>1</u> of <u>1</u> eam raises power using recirculation flow.					
Time	Position	Applicant's Actions or Behavior					
	NSO SRO	<ul> <li>Performs the following actions per DGP 01-01, Unit Startup, and DOP 0202-03, Reactor Recirculation Flow Control System Operation: Raises recirculation pump speed using the master controller potentiometer. Verifies expected power increase.</li> <li>Directs raising reactor power per DGP 01-01, Unit Startup, and DOP 0202-03, Reactor Recirculation Flow Control System Operation, by raising</li> </ul>					
		recirculation pump speed.           ROLE PLAY:           QNE: request for a ramp rate, respond "limit ramp rate to 250 MWe/hr".					
	ANSO	Monitors panels and assists as directed.					
		<ul> <li>Event 1 Completion Criteria:         <ul> <li>Annunciator 902-5 G-8, 1-Element FW Control Active at Hi Flow, received.</li> <li>Significant power increase</li> <li>AND, at the direction of the NRC chief examiner.</li> </ul> </li> </ul>					

**Operator Actions** 

Form ES-D-2

Op-Test No: <u>ILT 01-1</u>		Scenario No.: <u>ILT-N-1</u>	Event No.: <u>2</u>	Page <u>1</u> of <u>1</u>				
Event Desc	Event Description: The crew transfers Feedwater Level Control System to 3-Element control.							
Time	Position	Applicar	it's Actions or Behavior					
	NSO	Performs the following actions p Valve (FRV) Operation: Verifies the AUTO Com Verifies that annunciato Hi Flow, is in alarm. Verifies that FWLCS is Depresses AUTO push 3-ELEM white indicating	ber DOP 0600-06, Feedv trol mode pushbutton is i or 902-5 G-8, 1-Element in Master Auto. button and verifies that t g lights are lit and the 1-f Element Control per DO	water Regulating not amber. FW Control Active at he AUTO and ELEM light goes out. P 0600-06 :				
	300	Feedwater Regulating Valve (FRV) Operation. Note: RPV level may take a couple of inch swing during the transfer to 3-Element Control, so the NSO may take Manual Control of the FRV. If th crew requests Instrument Maintenance (IMD) assistance to check out FWLC, tell the crew that you are time compressing and that IMD reports th FWLC system is functioning normally. The crew should then transfer FWL back to AUTO Mode.						
	ANSO	<ul> <li>ANSO Monitors panels and assists as directed.</li> <li><u>Event 2 Completion Criteria:</u> <ul> <li>FWLC in 3-Element Control.</li> <li>AND, at the direction of the NRC chief examiner.</li> </ul> </li> </ul>						

**Operator Actions** 

Op-Test No: ILT 01-1 Scenario No.: ILT-N-1 Event No.: 3 Page <u>1</u> of <u>1</u> Event Description: The main turbine bypass valve #1 opens spuriously. The valve is closed when the ANSO takes manual action at the EHC control panel Time Position Applicant's Actions or Behavior SIMULATOR OPERATOR: At the discretion of the NRC chief examiner, activate trigger 1, which works in conjunction with trigger 2 to open the #1 turbine bypass valve until the partial open light is lit If after 2 minutes the team has not noticed the bypass valve open, activate trigger 16, which works in conjunction with trigger 17 to open the #1 turbine bypass valve until the full open light is lit and the bypass valve open alarm is received. Note: when the crew depresses the bypass valve jack decrease pushbutton, triggers 18 and 19 will activate to delete the bypass jack increase pushbutton and light overrides. ANSO Performs the following actions per DAN 902-7 G-3, Turb Byp VIv Open, and/or DOA 5650-03, Turbine Control Valve or Bypass Valve Failed Open, as directed: Verifies reactor pressure stable Verifies bypass valve should be closed Closes the bypass valve with the bypass valve jack. NSO Acknowledges and announces alarm 902-5 C-8, Main Stm-Turbine Stm 10% Mismatch. He should recognize it alarmed due to the bypass valve opening. SRO Enters and directs performance of DOA 5650-03, Turbine Control Valve or Bypass Valve Failed Open. Places startup on hold. □ Notifies Shift Manager. May refer to the site technical requirements and determine: ITS 3.3.1.1 is not applicable. ITS 3.3.6.1 is not applicable. ITS 3.7.7 is not applicable. **ROLE PLAY:** After the crew closes the bypass valve, call the control room as the IM Supervisor and report "my crew was hooking up a Fluke to take readings on the bypass valve control circuits in panel 903-31 in the AEER. The technician thinks he inadvertently shorted between two terminal points that may have caused the bypass jack to operate. Then he discovered he was in the wrong panel. He was in the 902-31 panel instead of the 903-31 panel. The Unit 2 EHC Control system should operate normally now". Respond as groups notified.

Event 3 Completion Criteria:

- Bypass valve closed
- AND, at the direction of the NRC chief examiner.

F

Op-Test N	o: <u>ILT 01-1</u>	Scenario No.: <u>ILT-N-1</u> Event No.: <u>4 &amp; 5</u> Page <u>1</u> 0f <u>1</u>						
Event Des	Event Description: APRM channel 5 fails downscale. IRM 15 spikes upscale and a partial half-scram occurs. The NSO inserts a complete half-scram. The SRO addresses the technical specification requirements for the ARPM channel. The APRM channel is bypassed but the half-scram should not be reset by the NSO.							
Time	Position	Applicant's Actions or Behavior						
		SIMULATOR OPERATOR:						
		At the discretion of the NRC chief examiner, activate trigger 3, which fails APRM 5 downscale and 5 seconds later spikes IRM 15 upscale.						
		NOTE: trigger 4 activates on the half scram signal to remove IRM 15 upscale so it appears to be a spike.						
		NOTE: If the team attempts to reset the half scram, trigger 5 will activate to fail the fuses for scram groups B2 and B3.						
	NSO	Performs the following actions per DOA 0500-02, Partial 1/2 or Full Scram Actuation:						
		Recognizes and announces partial half scram. Manually scrams RPS channel B. (immediate action) Determines all channel B scram solenoid lights are off.						
	SRO	Enters and directs performance of DOA 0500-02, Partial 1/2 or Full Scram Actuation.						
	NSO	Performs the following actions per DAN 902-5 C-6, APRM Downscale, as directed:						
		Compares to other APRMs. Notifies SRO Bypasses APRM channel 5.						
	Performs the following actions per DAN 902-5 C-6, APRM Downscale, as directed: Checks APRM parameters on 902-37 panel.							
	SRO	<ul> <li>References plant Technical documents:</li> <li>ITS Table 3.3.1.1.A-1, verifies sufficient RPS APRM trip channels (2 available / 2 required for B channel)</li> <li>TRM Table T3.3.a-1, verifies sufficient APRM rod block channels available (5 available / 4 required)</li> </ul>						
		Directs bypassing APRM 5.						
		Notifies the Shift Manager and IMD.						
		Events 4 & 5 Completion Criteria: - RPS channel B manually scrammed. - APRM 5 bypassed. - AND, at the direction of the NRC chief examiner.						

Op-Test N	o: <u>ILT 01-1</u>	Scenario No.: <u>ILT-N-1</u> Event No.: <u>6</u> Page <u>1</u> of <u>2</u>							
Event Des	Event Description: The main feed breaker to Bus 23-1 then trips and the 2/3 EDG fails to automatically start. The 2/3 EDG does run when manually started by the ANSO and power is restored to Bus 23-1.								
Time	Position	n Applicant's Actions or Behavior							
		SIMULATOR OPERATOR:							
		At the discretion of the NRC chief examiner, activate trigger 6, which inserts failure of RPS to deenergize, pulls ARI fuses, and main feed breaker to MCC 23-1 trip with failure of 2/3 EDG to start automatically							
	ANSO	Diagnoses that Bus 23-1 has lost power and that the 2/3 EDG failed to auto start to pick up the bus							
		Should perform the following actions of DGA 12, Partial or Complete Loss of AC Power, and DOA 6600-01, Diesel Generator Failure: Places the 2/3 EDG control switch to start. Dispatches an operator to the 2/3 EDG with Attachment A. Verifies power restored to Bus 23-1 and Bus 28. Resets annunciators and verifies the ones remaining are expected. Dispatches an operator to check the U2 EDG trouble alarm							
		May perform the following actions of DOA 6500-10, 4KV Circuit Breaker Trip if time permits: Dispatches NLO to check for signs of fault or overload. Verifies no other reason for trip.							
	NSO	Monitors panels and assists as directed.							
		ROLE PLAY:							
		NLO to check Bus 23-1 feed breaker at Bus 23 (wait 3 min):							
		Report "The Bus 23-1 feed breaker at Bus 23 is open and there are no targets up at the breaker".							
		NLO to check Bus 23-1 feed breaker at Bus 23-1 (wait 3 min):							
		Report "The Bus 23-1 feed breaker at Bus 23-1 is open and there are no targets up at the breaker".							
		NLO to perform Attachment A of DOA 6600-01:							
		Respond "I will get a copy of Attachment A of DOA 6600-01 and complete it for the 2/3 EDG".							
		SIMULATOR OPERATOR/ROLE PLAY:							
		NLO to check U2 EDG trouble alarm (wait 3 min):							
		Activate trigger 12 then report "the alarms were circulating lube oil and turbo charger lube oil pumps trouble alarms. The pumps are operating normally and the alarms have reset".							

Op-Test No: <u>ILT 01-1</u>		Scenario No.: <u>ILT-N-1</u>	Event No.: 6	Page <u>2</u> of <u>2</u>				
Event Des	Event Description: The main feed breaker to Bus 23-1 then trips and the 2/3 EDG fails to automatically start. The 2/3 EDG does run when manually started by the ANSO and power is restored to Bus 23-1.							
Time	Position	Applicar	nt's Actions or Behavior					
	SRO	Enters and directs actions for the following as time permits: DGA 12, Partial or Complete Loss of AC Power. DOA 6600-01, Diesel Generator Failure. DOA 0500-05, Loss of Reactor Protection System Bus. DOA 6500-01, 4KV Bus Failure. DOA 6500-10, 4KV Circuit Breaker Trip.						
		The major actions to direct are: Starting the 2/3 EDG Verifying major loads reenergize.						
	SRO	Notifies Shift Manager and EMI	Э.					
		Role Play: Acknowledge requests for actions not already addressed earlier. If the crew request status of previous requests, report "I am working on it as fast as I can".						
		QNE to check thermal limits.						
	Report "No thermal limits where violated".							
		<b>NOTE:</b> The intent here is to proceed to the next event soon after the 2/3 EDG is started and the crew has begun to address recovery of equipment. Equipment recovery is not necessary for the remainder of the scenario.						
	Event 6 Completion Criteria: - 2/3 EDG supplying Bus 23-1. - AND, at the direction of the NRC chief examiner.							

Op-Test N	lo: <u>ILT 01-1</u>	Scenario No.: <u>ILT-N-1</u> Event No.: <u>7 &amp; 8</u> Page <u>1</u> of <u>4</u>						
Event Description: A spurious RPS Channel A scram begins an ATWS condition. The team should perform the ATWS DEOP and insert the control rods. The RFPs trip and the cr should use HPCI for RPV level control.								
Time	Position	Applicant's Actions or Behavior						
		SIMULATOR OPERATOR: At the discretion of the NRC chief examiner, activate trigger 7, which causes a spurious RPS channel A half scam and trips the RFPs.						
	NSO	Performs the following actions per DGP 02-03, Reactor Scram: Presses scram pushbuttons Places mode switch in shutdown Check rods inserted; discovers rods not inserted. Initiates ARI, checks rods, announces ATWS condition.						
		Performs the following actions per DGP 02-03, Reactor Scram: Verifies turbine and generator tripped (only if Group 1 has occurred or reactor power has dropped low enough that this should occur) Inserts SRM/IRMs						
	NSO / ANSO	Performs DEOP 400-5, Failure to Scram, actions as directed: Places ADS to inhibit Places both CS pumps in PTL Power Leg						
		Inserts IRMs & SRMs $\sqrt{2}$ Performs Alternate Rod Insertion. (see specific actions below) $\sqrt{2}$ May Inject SBLC.						
		Level Leg       Rx Power >6%       Rx Power ≤6%         √ Terminates and Prevents       Maintains level between -164         injection except boron and       and 48 in.         CRD until RPV/L is ≤ -35 in.       √ Maintains RPV/L         between -164 in. and the       level lowered to.						
		Pressure Leg Verifies turbine/bypass valves maintaining RPV/P <1060 psig. If MSIVs isolate, maintains <1060 psig using IC, ADSVs and / or HPCI.						
		SIMULATOR OPERATOR / ROLE PLAY: Operator to jumper the MSIV Group1 –59 in. and offgas hi hi radiation isolations (wait 5 min):						
		Activate trigger 8 (jumpers the MSIV Group1 –59 in. and offgas hi hi radiation isolations) and report "the MSIV Group1 –59 in. and offgas hi hi radiation isolations are jumpered".						

Op-Test No: <u>ILT 01-1</u>

Scenario No.: <u>ILT-N-1</u>

Event No.: <u>7 & 8</u> Page <u>2</u> of <u>4</u>

Event Description: A spurious RPS Channel A scram begins an ATWS condition. The team should perform the ATWS DEOP and insert the control rods. The RFPs trip and the crew should use HPCI for RPV level control. Time Position Applicant's Actions or Behavior SRO Enters and directs performance of DGP 02-03, Reactor Scram, and DEOP 100, RPV Control. Due to report of ATWS condition, exits DEOP 100 and enters and directs performance of DEOP 400-05, Failure to Scram. Placing ADS to inhibit Placing both CS pumps in PTL Power Lea  $\sqrt{1}$  Directs any of the following control rod insertion methods (first two most preferred) per DEOP 500-05, Alternate Insertion of Control Rods: step G.2: pulling scram channel A solenoid power supply fuses. step G.4: venting the scram air header. step G.5: using the scram test switches. step G.6: manually driving of withdrawn control rods.  $\sqrt{M}$  May direct Injecting SBLC Level Leg Verification water level instruments are accurate Verification any required automatic actions have occurred Directing jumpers installed for MSIV low level and Off Gas high Rad isolations Rx Power >6% Rx Power ≤6%  $\sqrt{1}$  Terminating and Maintaining level between Preventing injection -164 and 48 in. except boron and CRD until RPV/L is  $\leq$  -35 in.  $\sqrt{Maintaining RPV/L}$ 

between -164 in, and the

Maintaining <1060 psig using turbine/bypass valves.

If MSIVs isolate, maintaining <1060 psig using IC, ADSVs or HPCI.

level lowered to.

Pressure Leg

Op-Test No: ILT 01-1 Scenario No.: ILT-N-1 Event No.: 7 & 8 Page 3 of 4 Event Description: A spurious RPS Channel A scram begins an ATWS condition. The team should perform the ATWS DEOP and insert the control rods. The RFPs trip and the crew should use HPCI for RPV level control. Time Position Applicant's Actions or Behavior SIMULATOR OPERATOR / ROLE PLAY: Note: For the first two requested actions below, wait about 10 minutes and / or at the discretion of the NRC chief examiner 1. Operator to pull scram channel A solenoid power supply fuses: Activate trigger 10 (pulls Channel A RPS fuses and trips reactor feed pumps) and then after the fuses are all pulled report "the scram channel A solenoid power supply fuses are pulled". 2. Operator to vent the scram air header: Activate trigger 11 (vents the scram air header) and report "I have vented the scram air header". 3. Operator to use the individual scram test switches (wait 2 min); Go to Instructor Station screen ROD5 and begin simulating flipping the scram test switches. Follow the guidance in DEOP 500-05 for selecting rods. NSO If directed, attempts to drive withdrawn control rods in per step G.6 of DEOP 500-05, Alternate Insertion of Control Rods: May close CRD 25 charging water valve. Throttles open the CRD FCV. May start a second CRD pump. May maximize drive water pressure using the CRD 8 valve Bypasses the RWM. Applies an insert signal to insert rods in the following order: intermediate 16-32 deep 4-14 shallow 34-48 SIMULATOR OPERATOR / ROLE PLAY: NLO to close CRD 25 valve (wait 3 min): Activate trigger 9 (closes the CRD 25 valve) and report "the CRD 25 valve is closed". NSO Reports that all rods are inserted. ANSO Restores RPV level to +8 to +48 inches as directed. Begins cooldown as directed. SRO When receives report that all rods are inserted, exits DEOP 400-05. Failure to Scram and enters DEOP 100 and directs: Controlling RPV water level between +8 to +48 inches. May direct depressurizing the RPV at <100 °F/hr.

Op-Test No: <u>ILT 01-1</u>		Scenario No.: <u>ILT-N-1</u>	Event No.: <u>7 &amp; 8</u>	Page <u>4</u> of <u>4</u>			
Event Desc	cription: A spu perfc shou	urious RPS Channel A scram begin form the ATWS DEOP and insert th and use HPCI for RPV level control.	ious RPS Channel A scram begins an ATWS condition. The team should m the ATWS DEOP and insert the control rods. The RFPs trip and the crew d use HPCI for RPV level control.				
Time	Position	Applicant's Actions or Behavior					
		<ul> <li>Critical Tasks: (identified by √ With a reactor scram req ACTION TO REDUCE P control rods, to prevent e limits.</li> <li>During an ATWS with co TERMINATE AND PREV and CRD, into the RPV u injection. (May not apply requiring this action is rea When conditions are met systems to MAINTAIN R</li> <li>Scenario Completion Criteria: - Control rods inserted.</li> <li>Plant stabilized.</li> <li>AND, at the direction of the N</li> </ul>	in guide) Juired and the reactor no <b>POWER</b> by injecting boro exceeding the primary co nditions met to perform p <b>VENT INJECTION</b> , with e intil conditions are met to if power becomes <6% b ached) t to establish injection us PV water level above -16	t shutdown, <b>TAKE</b> n and/or inserting ntainment design power/level control exception of boron o re-establish pefore the step e available injection 54".			

# **Dresden Generating Station**

SIMULATOR EXERCISE GUIDE

ILT 01-1 NRC EXAM

**SCENARIO** 

ILT-N-2

Rev. 01

05/02

**DEVELOPED BY:** 

Exam Author

Date

APPROVED BY:

Facility Representative

Date

Scenario Outline

Form ES-D-1

Facility:	Facility:         Dresden         Scenario No:         ILT-N-2         Op-Test No:         ILT 01-1						
Examin	Examiners: Operators:						
Initial Conditions: Unit in Mode 2 at approximately 2% reactor power; IRM channel 16 out of service; 2B EHC Pump OOS; Unit 3 is in Mode 4. <u>Turnover</u> : Unit startup in progress; return TBCCW pump 2B to service following maintenance, then continue power ascension							
Event No.	Malf. No.	[ -	Event Type*	Event Description			
1	N/A	N	ANSO SRO	swap TBCCW pumps			
2	N/A	R	NSO SRO	raise reactor power by withdrawing control rods			
3	RODC13DO	с	NSO SRO	control rod double notches during withdrawal			
4	NII12POT	1	NSO SRO	IRM channel fails upscale			
5	PCPDWTOR	I	ANSO SRO	drywell to torus differential pressure controller failure			
5a	AT23	I	ANSO SRO	fails 2A Rx Bldg to Torus vacuum breaker open			
6	HP8	с	ANSO SRO	circulating water pump trip			
7	CSBRKSEV	м	ANSO SRO	lowering torus level from ECCS suction line break			
8	J33	с	ANSO SRO	loss of EHC system			

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

Form ES-D-2

Op-Test No: <u>ILT 01-1</u>		Scenario No.: <u>ILT-N-2</u>	Event No.:	1	Page	<u>1</u> of <u>1</u>		
Event Desc	Event Description: Maintenance has been completed on TBCCW pump 2B and the SRO directs the ANSO to switch running pumps and place the TBCCW pump 2B in service.							
Time	Position	Applicar	t's Actions or Be	havior				
	ANSO	Performs the following actions p Cooling Water System(TBCCW Directs NLO to verify 21 valves open. Directs NLO to verify 21 Starts 2B TBCCW pum Stops 2A TBCCW pum Verifies system parame	ber DOP 3800-01 (): 3 TBCCW pump 3 TBCCW pump p and verifies pro p. eters normal.	, Turbine I suction an oil levels. oper opera	Building ( d discha tion.	Closed rge		
		ROLE PLAY:		tina. spa		tury sa Aline ta		
	SBO	NLO to verify 2B TBCCW pump min): Report "2B TBCCW pump suct NLO to verify 2B TBCCW pump Report "2B TBCCW pump oil le NLO to report on operation of 2 Report "2B TBCCW pump is op NLO to check TBCCW system min) Report "TBCCW system param	o suction and disc on and discharge o oil levels (wait 1 vels are normal". B TBCCW pump erating normally" parameters after eters are normal	charge valv e valves an min): 24 pump i 29 pump i	ves open". e open". s secure	(wait 1 d: (wait 1		
	SRU	3800-01, Turbine Building Closed Cooling Water System (TBCCW).						
	NSO	Monitors panels and assists as <u>Event 1 Completion Criteria:</u> – 2B TBCCW pump running a – AND, at the direction of the	directed. and 2A TBCCW p NRC chief exam	oump stopp iner.	oed.			

Г

Op-Test No: <u>ILT 01-1</u>		Scenario No.: <u>ILT-N-2</u>	Event No.: 2	Page <u>1</u> of <u>1</u>				
Event Desc	Event Description: The NSO, as directed by the SRO, then continues the power ascension for unit startup by control rod withdrawal.							
Time	Position	Applicant's	s Actions or Beha	avior				
	NSO	Performs the following actions per DOP 0400-01, Reactor Manual Control System Operation, and DGP 03-04, Control Rod Movements, as directed <u>Verifies the following prior to moving any control rod:</u> Control rod selected on the select matrix is correct rod. Second Verification requirements satisfied. Rod Out Permit light is illuminated. Drive water pressure at nominal 260 psid. <u>Withdraws rods as follows:</u> Moves Rod Out Notch Override (RONOR) Switch to NOTCH OVERRIDE position (use of RONOR switch is optional) and the Rod Movement Control switch to ROD OUT.						
		Verifies ON light illuminated and proper Control Rod Timer operation. Releases switches before target position is reached. Verifies rod settles to target position and proper response of nuclear instrumentation.						
	ANSO / or Surrogate	Performs second verification check <u>For first rod in a step:</u> Verifies correct control roverifies correct step and a Verifies RWM rod blocks <u>For all rods moved:</u> Verifies correct control roverifies planned control rowerifies planned control rowerifies co	forms second verification checks. <u>first rod in a step:</u> Verifies correct control rod pattern Verifies correct step and array. Verifies RWM rod blocks enabled <u>all rods moved:</u> Verifies correct control rod selected. Verifies planned control rod motion is correct. Immediately notify the NSO of errors during rod motion.					
-	SRO	Directs pulling control rods. Reviews REMA. Designates second verifier. Directs NSO to pulls rods. Event 2 Completion Criteria: - Sufficient power increase. - AND, at the direction of the NRC chief examiner.						

Op-Test No	o: <u>ILT 01-1</u>	Scenario No.: <u>ILT-N-2</u> Event No.: <u>3</u> Page <u>1</u> of <u>1</u>				
Event Description: During the control rod withdrawal, a control rod double notches beyond the withdraw limit and must be repositioned.						
Time	Position	Applicant's Actions or Behavior				
	NSO	Two possible paths can occur:				
		Either: Observes and announces that control rod N-13 has moved to position 14 (1 notch beyond its target position) and responds to alarms: 902-5 B-3, Rod Worth Min Block 902-5 C-3, Rod Out Block				
		Should perform the following actions per DOA 0300-12, Mispositioned Control Rod: Discontinues rod movement and notifies SRO. Moves the mispositioned rod to its target position. Records in DOS-0300-06, Control Rod Abnormality Record.				
		<u>Or:</u> When observes rod N-13 moving past its target position of 12, then: Uses the Emerg Rod In position of the Rod Out Notch Override switch to move the rod back to its target position prior to it latching in to position 14. Notifies the SRO. Records in DOS-0300-06, Control Rod Abnormality Record.				
	SRO	May enter and direct performance of DOA 0300-12, Mispositioned Control Rod.				
		Notifies the Shift Manager. May refer to ITS 3.1.3 and determine that no actions are required.				
		May enter DOA 0300-05, Inoperable or Failed Control Rod Drives.				
		ROLE PLAY: Respond as the Shift Manager. QNE: If crew requests guidance from the QNE, respond "I recommend inserting rod N-13 to position 12.				
	ANSO	Monitors panels and assists as directed.				
		Event 3 Completion Criteria: - Rod returned to target position - AND, at the direction of the NRC chief examiner.				

Op-Test N	o: <u>ILT 01-1</u>	Scenario No.: <u>ILT-N-2</u> Event No.: <u>4</u> Page <u>1</u> of <u>1</u>			
Event Description: IRM channel 12 then fails upscale and a half-scram occurs on the RPS "A" channel The NSO bypasses the failed IRM channel and the SRO addresses the technical specification requirements for the failure.					
Time	Position	Applicant's Actions or Behavior			
		SIMULATOR OPERATOR:			
		At the discretion of the NRC chief examiner, activate trigger 1, fails IRM 12 channel upscale			
	NSO	<ul> <li>Should perform the following actions per DAN 902-5 C-15: If not in the RUN Mode, verifies the following occurred: <ul> <li>Channel A half scram</li> <li>Rod Block.</li> </ul> </li> <li>Verifies IRM 12 readings against other IRMs on 902-5 panel. Verifies IRM range switch in correct position Bypasses IRM 12 after T. S. compliance verified by SRO. Resets RPS channel A per DOP 0500-07, Insertion/Reset of Manual Half Scram as follows:</li> </ul>			
		<ul> <li>Verifies half scram no longer required</li> <li>Turns the Scram Reset switch in each direction and verifies all eight white group solenoid lights are lit.</li> <li>Verifies alarm 902-5 A-10, Channel A Manual Trip, resets.</li> </ul>			
	ANSO	Should perform the following actions per DAN 902-5 C-10: Verifies IRM 12 readings against other IRMs on 902-36 panel. Verifies IRM 12 function switch in operate. Verifies power supplies operating properly.			
	SRO	<ul> <li>Should references plant technical documents:</li> <li>ITS Table 3.3.1.1.A-1, verifies sufficient RPS IRM trip channels (3 available / 3 required for A channel) for Mode 2.</li> <li>TRM Table T3.3.a-1, verifies sufficient APRM rod block channels available (7 available / 6 required) for Mode 2.</li> </ul>			
		Should direct IRM 12 bypassed and the half scram reset per DOP 0500-07, Insertion/Reset of Manual Half Scram.			
		Notifies the Shift Manager and IMD.			
		Role Play: Respond as persons notified.			
		Event 4 Completion Criteria: - IRM 12 bypassed. - Half scram reset. - AND, at the direction of the NRC chief examiner.			

Op-Test No	o: <u>ILT 01-1</u>	Scenario No.: <u>ILT-N-2</u> Event No.: <u>5</u> Page <u>1</u> of <u>1</u>				
Event Desc	cription: Dryw is re diffe	rell to Torus differential pressure then begins to decrease and pressure control gained when the ANSO takes manual control of the Drywell to Torus rential pressure controller.				
Time	Position	Applicant's Actions or Behavior				
		SIMULATOR OPERATOR: At the discretion of the NRC chief examiner, activate trigger 2, which causes drywell to torus differential pressure controller input to fail high causing the demand to fail to 0%.				
	ANSO	Acknowledges and announces alarm 902-4 B-15, DW to Torus DP Hi/Lo, and performs the following: Should diagnose failure of the drywell to torus differential pressure controller AUTO mode of operation.				
		<ul> <li>Performs any of the following as directed:</li> <li>Places the drywell to torus differential pressure controller to MAN and controls the drywell to torus differential pressure manually;</li> <li>AND / OR, vents the torus to Reactor Building Ventilation per DOP 1600-01, Normal Pressure Control of the Drywell or Torus:</li> <li>Verifies atmospheric sample results allow venting.</li> <li>Verifies U2 Reactor Building Ventilation operating.</li> <li>Verifies AO 2-1601-91 open.</li> <li>Opens AO 2-1601-24. (closes when desired to stop venting)</li> <li>Opens AO 2-1601-61. (closes when desired to stop venting)</li> </ul>				
		May close AO 2-1601-58 per DOP 1600-05, Primary Containment Inerting and Atmosphere Control.				
SRO		To maintain Primary Containment pressures to those specified in DOP 1600- 05, Primary Containment Inerting and Atmosphere Control, should direct performance of any of the following: Placing the drywell torus differential pressure controller to MAN and controlling Drywell to Torus differential pressure manually; AND / OR, venting the torus to Reactor Building Ventilation per DOP 1600-01, Normal Pressure Control of the Drywell or Torus, to maintain the required differential pressure. May direct closing AO 2-1601-58 per DOP 1600-05, Primary Containment Inerting and Atmosphere Control. May notify IMD.				
	NSO	Monitors panels and assists as directed.				
		BOLE PLAY:         IMD to investigate drywell pressure controller:         Respond "I will send a technician to investigate".         Event 5 Completion Criteria:         – Drywell to Torus differential pressure control in progress.         – AND, at the direction of the NRC chief examiner.				

Op-Test No: <u>ILT 01-1</u>		Scenario No.: <u>ILT-N-2</u> Event No.: <u>5.a</u> Page <u>1</u> of <u>1</u>			
Event Description: A drifting Rx Bldg to Torus differential pressure instrument will cause the 2A Rx Bldg to Torus Vacuum Breaker to fail open. The Unit Supervisor should reference the Technical Specifications.					
Time	Position	Applicant's Actions or Behavior			
		SIMULATOR OPERATOR:			
		At the discretion of the NRC chief examiner, activate trigger 6, which causes 2A Rx Bldg to Torus Vacuum Breaker to fail open.			
	ANSO	Acknowledges and announces alarms 902-3 B-14, and 902-3 C14, and performs the following:			
		<ul> <li>Evaluates Rx Bldg to Torus DP to determine if valve should have opened.</li> <li>Directs a NLO to check the AO 2-1601-20A valve.</li> <li>Notifies IMD to perform DIS 1600-3, Torus to Reactor Building Vacuum Breakers Trip Unit Calibration.</li> </ul>			
	SRO	References Technical Specifications and determines:			
		<ul> <li>ITS 3.6.1.7.A, close the open vacuum breaker within 7 days.</li> <li>ITS 3.0.4, cannot change mode.</li> </ul>			
		Notifies Shift Manager and IMD.			
	NSO	Monitors panels and assists as directed.			
		ROLE PLAY:			
		NLO to investigate 2-1601-20A, 2A Rx Bldg to Torus Vacuum Breaker: (wait 3 min)			
		Report "the 2-1601-20A, 2A Rx Bldg to Torus Vacuum Breaker is open. I see nothing abnormal"			
		Respond as groups notified.			
		Event 5a Completion Criteria: - Unit Supervisor has addressed Technical Specifications. - AND, at the direction of the NRC chief examiner.			

Appendix D	)	Operator Action	S	Form ES-D-2
Op-Test N Event Des	lo: <u>ILT 01-1</u> cription: Circu circu	Scenario No.: <u>ILT-N-2</u> ulating water pump 2C then trips on ulating water pump 2A to maintain c	Event No.: <u>6</u> overload and the ANS ondenser vacuum.	Page <u>1</u> of <u>1</u> O manually starts
Time	Position	Applicant	s Actions or Behavior	
		SIMULATOR OPERATOR: At the discretion of the NRC chief Circulating Water pump.	f examiner, activate trig	ger 3, which trips 2C
	ANSO	Performs the following actions per 4400-01, Circulating Water Syste Breaker Trip, as directed: Starts 2B Circulating Water Verifies condenser vacuu Verifies 2C Circulating W Sends NLO to check 2C operation of 2B Circulating Places 2C Circulating Water Verifies Circulating Water May send NLO to check of	r DAN 902-7 A-15, Circ m Failure, and DOA 65 ter pump. (immediate a um returning to normal. ater pump discharge v Circulating Water pump ig Water pump. ater pump control switc r Flow reversal valves I cribhouse bar racks an	c Wtr PP Trip, DOA 500-10, 4KV Circuit action) alve closes. p breaker and h in PTL. ined up normally. d traveling screens.
	SRO	ROLE PLAY: NLO to 2C Circulating Water pur Report "2C Circulating Water pur NLO to check 2B Circulating Water Report "2B Circulating Water pur NLO to check cribhouse bar racks Report "the cribhouse bar racks a Enters and directs performance o	ip breaker (wait 3 min) np breaker has an over er pump operation (wai ip is operating normally s and traveling screens and traveling screens ar f DOA 4400-01, Circula	rcurrent target up". it 2 min) y". • (wait 3 min) re clear". ating Water System
	NSO	Pailure, and DOA 6500-10, 4KV C Notifies the Shift Manager and EM <u>Role Play:</u> Respond as persons notified. Monitors panels and assists as directly	Arcuit Breaker Trip. AD. rected.	
		<ul> <li>Event 6 Completion Criteria:</li> <li>2B Circulating Water pump st</li> <li>AND, at the direction of the N</li> </ul>	arted. RC chief examiner.	

Op-Test No	o: <u>ILT 01-1</u>	Scenario No.: <u>ILT-N-2</u> Event No.: <u>7 &amp; 8</u> Page <u>1</u> of <u>3</u>			
Event Description: An ECCS suction line break occurs resulting in a lowering torus water level. HPCI spuriously initiates. HPCI should be secured, the reactor should be scrammed and an emergency depressurization should be performed as directed by the DEOP for primary containment control.					
Time	Position	Applicant's Actions or Behavior			
		SIMULATOR OPERATOR: At the discretion of the NRC chief examiner, activate trigger 4, which starts an ECCS suction line break in the torus basement and spuriously initiates HPCI after 5 minutes.			
		When the unit is manually scrammed, verify trigger 5 automatically activates to trip 2A EHC Pump. Note: It takes about 30 minutes for torus level to reach 12 feet. At the discretion of the lead NRC examiner, use the cues later in this event to jump ahead in time to expedite the level drop if desired.			
	ANSO	<ul> <li>Reports the following alarms:</li> <li>902-4 C-23 Torus Narrow Range Wtr Lvl Lo</li> <li>923-4 A-3 U2 E RBFD Sump Lvl Hi Hi</li> <li>923-4 B-2 U2 W RBFD Sump Lvl Hi Hi</li> <li>Checks the torus narrow range level indicator. Reports level dropping.</li> <li>Directs NLO to perform DOS 1600-02, Torus Level Verification Using Local Sight Glass.</li> <li>Directs NLO to investigate leakage to torus basement.</li> <li>Verifies proper operation of the RBFD Sump pumps. (will require resetting the Group 2 isolation at both the 902-5 panel and the 923-4 panel for the sump pumps to operate if a Group 2 Isolation occurs)</li> </ul>			
		<b>BOLE PLAY:</b> NLO to perform DOS 1600-02, Torus Level Verification Using Local Sight Glass: (wait 5 min)Report "Local Torus level is (use value from variable ppc232, unless it is <20", then report it is below the sightglass)".			

Op-Test N	o: <u>ILT 01-1</u>	Scenario No.: <u>ILT-N-2</u> Event No.: <u>7 &amp; 8</u> Page <u>2</u> of <u>3</u>
Event Des	cription: An E spuri an ei prim	CCS suction line break occurs resulting in a lowering torus water level. HPCI iously initiates. HPCI should be secured, the reactor should be scrammed and mergency depressurization should be performed as directed by the DEOP for ary containment control.
Time	Position	Applicant's Actions or Behavior
	ANSO	Performs the following actions per DEOP 200-01, Primary Containment Control, as directed:
		May attempt to add water to the torus by opening the HPCI 14 valve. Monitors/Reports DEOP 200-01 entry parameters. $\sqrt{Prevents HPCI operation}$ by placing the HPCI 4 valve in PTI
	ANSO	Performs the following actions per DOA 0040-02, Localized Flooding in Plant, as directed: Makes PA announcement. Directs NLO to investigate leakage to torus basement.
		Notifies Radiation Protection and Security as time permits.
		Cue: (if desired for time compression) (to be handled by Lead Examiner) When torus level is < 14.5 feet and/or at the discretion of the lead NRC examiner, cue the crew that we are taking a time jump and that both torus wide range level meters indicate 12.5 feet and are dropping at about 0.1 ft per minute.
	SRO	Enters and directs performance of DEOP 0200-01, Primary Containment Control:
		<ul> <li>May attempt to add water to the torus by opening the HPCI 14 valve. May decide to anticipate RPV Blowdown:</li> <li>directs a manual scram per DGP 02-03, Reactor Scram.</li> <li>enters DEOP 100, RPV Control.</li> <li>directs opening turbine bypass valves.</li> <li>√ Directs ANSO to secure HPCI by placing the HPCI 4 valve in PTL before torus level reaches 12 feet.</li> </ul>
		Enters DEOP 0300-01, Secondary Containment Control, and directs: If Reactor Building Ventilation Isolates when unit is scrammed, directs restarting Reactor Building Ventilation.
	ANSO	Performs DEOP 300-01, Secondary Control, as directed: Time permitting, restarts Reactor Building Ventilation (if it isolates when the reactor is scrammed).
	NSO	√ Performs DGP 02-03, Reactor Scram, as directed: Presses scram pushbuttons Places mode switch in shutdown Checks rods inserted Maintain RPV/L between +25 and +35 inches or as directed by DEOPs

Op-Test N	o: <u>ILT 01-1</u>	Scenario No.: <u>ILT-N-2</u> Event No.: <u>7 &amp; 8</u> Page <u>3</u> of <u>3</u>
Event Des	cription: An E spur an e prim	CCS suction line break occurs resulting in a lowering torus water level. HPCI iously initiates. HPCI should be secured, the reactor should be scrammed and mergency depressurization should be performed as directed by the DEOP for ary containment control.
Time	Position	Applicant's Actions or Behavior
	ANSO	When attempts to open bypass valves (if directed), reports that the 2B EHC pump tripped.
		Cue: (if time compression was used above)
		10 minutes after the initial time compression cue and/or at the discretion of the lead NRC examiner, cue the crew that both torus wide range level meters indicate 11.5 feet and are dropping at about 0.1 ft per minute.
	SRO	When informed that torus level is approaching 11 feet: $\sqrt{\text{Directs a manual scram}}$ (if not already directed above) per DGP 02-03, Reactor Scram.
		<ul> <li>Enters DEOP 0400-02, Emergency Depressurization, and directs:</li> <li>Initiation of Iso Condenser to maximum flow</li> <li>Verification that SP/L &gt;6 feet.</li> <li>√ Opening all ADS valves</li> <li>Verification all relief valves are open.</li> </ul>
	ANSO	<ul> <li>Performs DEOP 0400-02, Emergency Depressurization, actions as directed: Initiates Iso Condenser to maximum flow</li> <li>Verifies that SP/L &gt;6 feet.</li> <li>√ Opens all ADS valves</li> <li>Verifies all relief valves are open.</li> </ul>
		<ul> <li>Critical Tasks: (identified by √ in guide)</li> <li>With reactor at power and suppression pool water level cannot be maintained in the safe region of the heat capacity temperature limit,</li> <li>MANUALLY SCRAM the reactor.</li> <li>When it is determined that suppression pool water level cannot be held above 12 feet wide range (level of HPCI exhaust), TRIP AND PREVENT HPCI operation irrespective of adequate core cooling. When it is determined that suppression pool water level cannot be held above 11 feet wide range (level of the downcorners), INITIATE emergency depressurization.</li> </ul>
		Scenario Completion Criteria:         -       HPCI secured.         -       Reactor scrammed.         -       RPV depressurization in progress.         -       AND, at the direction of the NRC chief examiner.

**Dresden Generating Station** 

SIMULATOR EXERCISE GUIDE

ILT 01-1 NRC EXAM

#### **SCENARIO**

ILT-N-3

Rev. 01

05/02

**DEVELOPED BY:** 

Exam Author

Date

APPROVED BY:

Facility Representative

Date

Scenario Outline

Form ES-D-1

Facility:	Facility:         Dresden         Scenario No:         ILT-N-3         Op-Test No:         ILT 01-1				
Examin	Examiners: Operators:				
Initial Co Pump o	onditions: Appr out of service; U	oxima nit 3 i	ately 78% s in Mode	reactor power; IRM channel 16 out of service; 2B EHC 4.	
<u>Turnove</u> 2B, ther	<u>هr</u> : Unit shutdo ۱ continue powe	wn in er redi	progress uction for	for forced outage; shutdown reactor condensate pump unit shutdown	
Event No.	Malf. No.		Event Fype*	Event Description	
1	N/A	N	ANSO SRO	shutdown condensate pump for unit shutdown	
2	N/A	R	NSO SRO	lower reactor power by reducing recirculation flow	
3	SER1375 FWDOP2 FWDOP5	С	NSO SRO	reactor feed pump 2B failure	
4	ADS3ESD	1	ANSO SRO	spurious ADS valve opening	
5	K11 MGDSCBTR	с	ANSO SRO	stator cooling water pump trips on overload and standby pump fails to start automatically	
6	RLLMLS	1	NSO SRO	feedwater level control system setpoint drifts high	
7	F44	м	ALL	small recirculation loop break	
8	ACDTP21 ACDCL21		ANSO SRO	TR 22 feed to Bus 21 fails to close	
9	HP8VBKR HPLCL8		ANSO SRO	HPCI injection valve failure	

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

**Operator Actions** 

Op-Test No: <u>ILT 01-1</u>		Scenario No.: <u>ILT-N-3</u> Event No.: <u>1</u> Page <u>1</u> of <u>1</u>			
Event Desc	cription: The t	eam continues the shut down by securing 2B Condensate Pump.			
Time	Position	Applicant's Actions or Behavior			
	SRO	Directs ANSO to secure 2B Condensate Pump per DOP 3300-03, Condensate System Shutdown.			
	ANSO	<ul> <li>Performs the following actions per DOP 3300-03, Condensate System</li> <li>Shutdown, to shutdown 2B Condensate Pump as directed: <ul> <li>Verifies Reactor Feed Pump Suction Pressure &gt;230 psig.</li> <li>Verifies closed the hydrogen isolation valves. (on turnover)</li> <li>Stops 2B Condensate Pump</li> <li>Selects 2B Condensate Pump for standby.</li> <li>Directs NLO to verify Condensate Pre-filter operating parameters.</li> </ul> </li> </ul>			
	NSO	Monitors panels and assists as directed.			
		ROLE PLAY:			
		NLO to verify 2B Condensate Pump hydrogen isolation valves closed: Report "hydrogen isolation valves are closed".			
		NLO to verify 2B Condensate Pump stopped rotating:			
		Report "2B Condensate Pump has stopped rotating".			
		NLO to verify Condensate Pre-filter operating parameters (wait 2 min):			
		Report: Condensate Pre-filter operating parameters are normal."			
		Event 1 Completion Criteria: - 2B Condensate Pump secured. - AND, at the direction of the NRC chief examiner.			

Op-Test No: <u>ILT 01-1</u>		Scenario No.: <u>ILT-N-3</u> Event No.: <u>2</u> Page <u>1</u> of <u>1</u>
Event Desc	cription: The t	eam reduces power using recirculation flow.
Time	Position	Applicant's Actions or Behavior
	NSO	<ul> <li>Performs the following actions per DGP 02-01, Unit Shutdown, and DOP 0202-03, Reactor Recirculation Flow Control System Operation: <ul> <li>Lowers recirculation pump speed using the master controller potentiometer.</li> <li>Verifies expected power reduction.</li> </ul> </li> </ul>
	SRO	Directs reducing reactor power per DGP 02-01, Unit Shutdown, and DOP 0202-03, Reactor Recirculation Flow Control System Operation, by lowering recirculation pump speed.
	ANSO	Monitors panels and assists as directed.
		Event 2 Completion Criteria: - Significant power reduction - AND, at the direction of the NRC chief examiner.

Op-Test No: ILT 01-1 Scenario No.: ILT-N-3

Event No.: <u>3</u> Page <u>1</u> of <u>3</u>

Event Description: A 2B RFP low lube oil condition will require the team to start 2C RFP and secure 2B RFP.

Time	Position	Applicant's Actions or Behavior		
		SIMULATOR OPERATOR:		
		At the discretion of the NRC chief examiner, activate trigger 1, which inserts 2B RFP low oil pressure and causes failure of its auxiliary oil pump to start.		
	NSO	<ul> <li>Performs the following actions per DAN 902-6 H-8, 2B RFP Brg Oil Press Lo:</li> <li>Attempts to start 2B RFP Auxiliary Oil Pump.</li> <li>Directs NLO to report 2B RFP oil pressure, oil reservoir level and check for oil leaks.</li> <li>Informs SRO 2B RFP is running with low oil pressure and the auxiliary oil pump will not start.</li> </ul>		
		ROLE PLAY:		
		NLO to check 2B RFP (wait 2 min):		
		Report "2B RFP oil pressure is 16 psig. Oil reservoir level is low. There is a large amount of oil on the pump base-plate The oil had not reached the base plate drain yet. I plugged the base plate drain".		
		NLO to check 2B RFP Aux Oil pump breaker: (wait 2 min)		
		Report "I see no problems with 2B RFP Aux Oil pump breaker".		
	SRO	Directs starting 2C RFP and securing 2B RFP.		
		Contacts the Shift Manager, and appropriate maintenance departments.		
	ANSO	Monitors panels and assists as directed.		
	NSO	<ul> <li>Starts 2C RFP per DOP 3200-03, Startup of second Reactor Feed Pump or Shifting to Alternate Reactor Feed Pump, as directed:</li> <li>May direct NLO to perform pre-startup checks.</li> <li>Places RFPs Standby selector switch in OFF.</li> <li>Closes the discharge valve.</li> <li>Opens the recirculation valve.</li> <li>Verifies RPV level stable.</li> <li>Starts discharge valve opening.</li> <li>Starts the RFP. (should start it on Bus 22 per procedure)</li> <li>Verifies RPV level stable.</li> <li>Closes the recirculation valve.</li> <li>Directs NLO to perform post-startup checks.</li> <li>Verifies auxiliary oil pump stops.</li> </ul>		

Op-Test No: ILT 01-1 Scenario No.: ILT-N-3

Event No.: <u>3</u> Page <u>2</u> of <u>3</u>

Event Description: A 2B RFP low lube oil condition will require the team to start another RFP and secure 2B RFP.

Time	Position	Applicant's Actions or Behavior		
		ROLE PLAY:NLO to perform RFP pre-startup checks (wait 5 min):Respond "I have completed the requested RFP startup pre-checks per stepG.1 through G.11 of DOP 3200-03".NLO to perform post-startup checks: (wait 5 min)Respond "The RFP post-startup checks are completed per G.31 of DOP3200-03".		
	NSO	<ul> <li>Performs the following actions per DOP 3200-05, Reactor Feed Pump Shutdown, to shutdown 2B RFP as directed: <ul> <li>Verifies zinc injection is not lined up to 2B RFP (may not wait on this).</li> <li>Places RFPs standby selector switch to OFF.</li> <li>Opens the recirculation valve.</li> <li>Verifies RPV level stable.</li> <li>Closes the discharge valve.</li> <li>Verifies RPV level stable.</li> <li>Stops the RFP.</li> <li>Verifies the running RFP amps below 1115 amps.</li> <li>Closes the recirculation valve.</li> <li>Has NLO verify the RFP has come to rest.</li> <li>Has NLO adjust zinc injection to ~20 gpm.</li> <li>Has NLO verify 2-5772-48B closed.</li> </ul> </li> </ul>		
		ROLE PLAY:         NLO to verify lineup of zinc injection (wait 1 min):         Respond "Zinc injection is lined up to 2A RFP".         NLO to verify 2B RFP has come to rest (wait 1 min):         Respond "2B RFP has stopped rotating".         NLO to verify 2B RFP is not rotating in reverse direction:         Respond "2B RFP is not rotating in reverse direction".         NLO to adjust zinc injection to ~20 gpm (wait 2 min):         Respond "1 adjusted zinc injection to ~20 gpm".         NLO to verify 2-5772-48B closed (wait 1 min):         Respond "5772-48B is closed".		

Op-Test No: ILT 01-1 Scenario No.: ILT-N-3

Event No.: <u>3</u> Page <u>3</u> of <u>3</u>

Event Description: A 2B RFP low lube oil condition will require the team to start another RFP and secure 2B RFP.

Time	Position	Applicant's Actions or Behavior
		Event 3 Completion Criteria: - 2C RFP started - 2B RFP shutdown - AND, at the direction of the NRC chief examiner.

Op-Test N	o: <u>ILT 01-1</u>	Scenario No.: <u>ILT-N-3</u> Event No.: <u>4</u> Page <u>1</u> of <u>1</u>			
Event Description: The E ADSV setpoint will drift low causing it to open. The team should close it by placing its control switch in OFF.					
Time	Position	Applicant's Actions or Behavior			
		SIMULATOR OPERATOR:			
		At the discretion of the NRC chief examiner, activate trigger 2, which sets the E ADS valve's setpoint to 890 psig causing it to open.			
	ANSO	<ul> <li>Performs the following actions per DAN 902-3 E-12, 2E Electromatic Relief Vlv Open:</li> <li>Verifies valve actually open by checking any of the following: <ul> <li>Valve position indication</li> <li>Generator output decrease.</li> <li>Acoustic monitor tripped.</li> <li>Increasing tailpipe temperature.</li> <li>Torus temperature increasing</li> </ul> </li> <li>Notifies SRO the valve is open.</li> </ul>			
	SRO	Enters and directs performance of DOA 0250-01, Relief Valve Failure.			
	ANSO	<ul> <li>Performs DOA 0250-01, Relief Valve Failure, as directed:</li> <li>Places the 2E Electromatic Relief valve control switch to OFF. (immediate action)</li> <li>Verifies RPV level stable. (immediate action)</li> <li>Verifies and reports the valve closed.</li> <li>Resets the acoustic monitor.</li> </ul>			
	SRO	<ul> <li>References Technical Specifications and determines:</li> <li>ITS 3.6.2.1, Suppression Pool Average Temperature, is not applicable.</li> <li>ITS 3.5.1.G, ECCS, if decides the failure does affect its ADS function, then restore the ADS valve to OPERABLE status within 14 days, otherwise, this is not applicable.</li> <li>ITS 3.4.3.A, Safety and Relief Valves, restore the relief valve to OPERABLE status within 14 days.</li> <li>ITS SR 3.6.1.8.2, Suppression Chamber to Drywell Vacuum Breakers, perform a functional test of each required vacuum breaker within 12 hours.</li> </ul>			
		<ul> <li>Event 4 Completion Criteria:</li> <li>2E Electromatic Relief valve closed.</li> <li>Referenced Technical Specifications.</li> <li>AND, at the direction of the NRC chief examiner.</li> </ul>			

F

Op-Test N	o: <u>ILT 01-1</u>	Scenario No.: <u>ILT-N-3</u> Event No.: <u>5</u> Page <u>1</u> of <u>1</u>			
Event Desestart auton	Event Description: The 2A stator cooling water pump trips on overload and the standby pump fails to start automatically. The team should manually start the 2B stator cooling water pump.				
Time	Position	Applicant's Actions or Behavior			
		SIMULATOR OPERATOR: At the discretion of the NRC chief examiner, activate trigger 3, which causes 2A stator cooling water pump to trip on overload and 2B to fails to auto start.			
	ANSO	<ul> <li>Announces the following alarms:</li> <li>DAN 902-7 B-10, Stator Clg PP Trip</li> <li>DAN 902-7 C-3, Turb Stator Coolant Runback</li> <li>Performs appropriate actions per DOA 7400-01, Failure of the Stator Coolant System: <ul> <li>Starts 2B Stator Cooling Water Pump (Immediate Action)</li> <li>Verifies Runback condition clears.</li> <li>Sends NLO to verify 2B Stator Cooling Water Pump operating normally.</li> </ul> </li> </ul>			
		<ul> <li>Performs appropriate actions per DOA 6700-06, 480V Circuit Breaker Trip:</li> <li>Sends NLO to check breaker and 2A Stator Cooling Water Pump for cause of trip.</li> <li>Places 2A Stator Cooling Water Pump control switch in PTL.</li> </ul>			
		SIMULATOR OPERATOR / ROLE PLAY: NLO to acknowledge stator cooling water trouble alarm: (wait 2 min)			
		Activate trigger 9 and report "I have acknowledged stator cooling water trouble alarm. The alarms were Inlet Pressure Low and Turbine Runback".			
		NLO to check operation of 2B Stator Cooling Water Pump: (wait 2 min) Report "2B Stator Cooling Water Pump is operating normally".			
		NLO to check cause of 2A Stator Cooling Water Pump trip (wait 2 min) Report "2A Stator Cooling Water Pump trip on overload".			
	SRO	Enters and directs performance of DOA 7400-01, Failure of the Stator Coolant System.			
		Enters and directs performance of DOA 6700-06, 480V Circuit Breaker Trip.			
		Notifies Work Week Manager, IMD and/or EMD			
	NSO	Monitors panels and assists as directed.           ROLE PLAY:           Respond to calls for assistance.			
		Event 5 Completion Criteria: - 2B Stator Cooling Water Pump started - AND, at the direction of the NRC chief examiner.			

Op-Test No	o: <u>ILT 01-1</u>	Scenario No.: <u>ILT-N-3</u> Event No.: <u>6</u> Page <u>1</u> of <u>1</u>				
Event Desc control of th	Event Description: The FWLC system setpoint will begin drifting up. The team should take manual control of the FWLC system.					
Time	Position	Applicant's Actions or Behavior				
		SIMULATOR OPERATOR:				
		At the discretion of the NRC chief examiner, activate trigger 7, which causes feedwater level control system setpoint to drift high (to 60") over 10 min.				
		If the crew scrams the unit, begin the next event.				
	NSO	Observes and announces RPV level rising or responds to alarm 902-5 E-8, RPV LvI Hi.				
		<ul> <li>Performs the following actions per DOA 0600-01 Transient Level Control:</li> <li>Takes manual control of the Feedwater Regulating Valves.</li> <li>Restores level to within band specified by the SRO</li> </ul>				
	SRO	Enters and directs performance of DOA 0600-01 Transient Level Control.				
		Notifies Shift Manager and IMD.				
		ROLE PLAY:				
		Respond to calls for assistance.				
		<u>Cue</u> :				
		If ANSO goes to check OIS screen, prompt him that the OIS Monitor is blank.				
	ANSO	Monitors panels and assists as directed.				
		Event 6 Completion Criteria: - Feedwater level control system under manual control. - AND, at the direction of the NRC chief examiner.				

Scenario No.: <u>ILT-N-3</u>

Event No.: 7,8 & 9 Page 1 of 4

Event Description: A small recirculation loop break occurs with a loss of high pressure injection. The team should perform the RPV Control and Primary Containment Control DEOPs. Due to the loss of high pressure injection and RPV level dropping to TAF, the team should Emergency Depressurize and restore RPV level with low pressure injection systems.

Time	Position	Applicant's Actions or Behavior		
		SIMULATOR OPERATOR: At the discretion of the NRC chief examiner, activate trigger 8, causes a 2% recirc loop leak and failure of the HPCI 8 viv to open.		
	NSO	Announces Reactor Scram on high Drywell pressure.		
		<ul> <li>Performs the following actions per DGP 02-03, Reactor Scram, and DEOP 100, RPV Control, as directed: <ul> <li>Places Mode Switch to Shutdown and depresses the Scram pushbuttons.</li> <li>Checks rods inserted.</li> <li>Maintains RPV level as directed by SRO.</li> <li>Checks turbine and generator tripped.</li> <li>Checks recirc pumps run back to minimum speed.</li> <li>Inserts SRMs and IRMs</li> </ul> </li> </ul>		
	ANSO	Should inform the SRO that High Pressure Feedwater and HPCI are not available.		
	SRO	<ul> <li>Enters DEOP 100, RPV Control, due to high PC/P and/or low RPV/L and performs/directs:</li> <li>Entering DGP 2-3</li> <li>Verification of water level instrument accuracy</li> <li>Verification of all isolations, ECCS and EDGs starts</li> <li>Holding RPV/L +8 to +48 inches</li> <li>Maintaining RPV/P &lt;1060 psig</li> </ul>		
		When informed no high pressure feed is available other than CRD, then directs: <ul> <li>Inhibiting ADS before59 inches.</li> <li>Initiating the isolation condenser</li> </ul>		
		<ul> <li>Directing use of high pressure Alternate Injection systems. (SBLC and CRD Crosstie)</li> <li>Verifies at least two low pressure injection systems available.</li> <li>Waits until RPV level drops to TAF.</li> <li>Verifies any low pressure system lined up with a pump running</li> </ul>		
		SIMULATOR OPERATOR / ROLE PLAY:		
		NLO to lineup CRD crosstie (wait 5 min.):		
		Verify trigger 10 is activated then report "the CRD crosstie is lined up".		

Scenario No.: ILT-N-3

Event No.: 7,8 & 9 Page 2 of 4

Event Description: A small recirculation loop break occurs with a loss of high pressure injection. The team should perform the RPV Control and Primary Containment Control DEOPs. Due to the loss of high pressure injection and RPV level dropping to TAF, the team should Emergency Depressurize and restore RPV level with low pressure injection systems.

Time	Position	Applicant's Actions or Behavior		
		SIMULATOR OPERATOR / ROLE PLAY: NLO to lineup makeup to SBLC Boron tank (wait 5 min.): Verify trigger 11 is activated then report "makeup is lined up to SBLC Boron tank".		
	SRO	<ul> <li>Before RPV level reaches -164 inches, enters DEOP 400-02, Emergency Depressurization, and directs:</li> <li>Initiation of Iso Condenser to maximum flow</li> <li>Verification that SP/L &gt;6 feet.</li> <li>√ Opening all ADS valves</li> <li>Verification all relief valves are open</li> </ul>		
	ANSO	<ul> <li>Performs DEOP 400-02, Emergency Depressurization, as directed:</li> <li>Prevents injection from LPCI/CS pumps not needed for core cooling per Hard Card, LPCI INJ/CC CONTROL/SHUTDOWN</li> <li>Initiates Iso Condenser to maximum flow per Hard Card, ISOLATION CONDENSER</li> <li>Verifies SP/L &gt;6 feet</li> <li>√ Opens all ADS valves</li> <li>Verifies all relief valves are open</li> </ul>		
	SRO	<ul> <li>√ Directs NSO/ANSO to control RPV level above TAF using any of the preferred injection systems listed below:</li> <li>Condensate</li> <li>Core Spray</li> <li>LPCI</li> </ul>		
	NSO / ANSO	$\sqrt{\text{Restores RPV}}$ level to that directed by the SRO (above TAF) using the systems specified by the SRO.		

Scenario No.: <u>ILT-N-3</u>

Event No.: 7,8 & 9 Page 3 of 4

Event Description: A small recirculation loop break occurs with a loss of high pressure injection. The team should perform the RPV Control and Primary Containment Control DEOPs. Due to the loss of high pressure injection and RPV level dropping to TAF, the team should Emergency Depressurize and restore RPV level with low pressure injection systems.

Time	Position	Applicant's Actions or Behavior		
- -	SRO	<ul> <li>Enters DEOP 200-1, Primary Containment Control, when PC/P reaches 2 psig and performs/directs:</li> <li>Monitoring of PC/P</li> <li>Initiation of torus sprays before PC/P of 9 psig</li> <li>When PC/P is above 9 psig or before DW/T reaches 281°F: <ul> <li>Verification of DSIL</li> <li>Tripping of recirc pumps</li> <li>Tripping of DW coolers</li> <li>Initiation of DW sprays</li> </ul> </li> <li>Monitoring of DW/T (drywell sprays may be initiated for temperature control)</li> <li>Monitoring of SP/T and initiation of torus cooling</li> <li>Monitors SP/L</li> <li>Verifies initiation of drywell and torus H₂/O₂ monitors</li> </ul>		
	ANSO	<ul> <li>Performs DEOP 200-1, Primary Containment Control, actions as directed:</li> <li>Monitors PC/P and initiates torus sprays and drywell sprays per Hard Card LPCI/CCSW OPERATION, as directed</li> <li>Monitors DW/T</li> <li>Monitors SP/T and initiates torus cooling per Hard Card LPCI/CCSW OPERATION as directed</li> <li>Monitors SP/L</li> <li>Verifies initiation of drywell and torus H₂/O₂ monitors</li> </ul> <b>ROLE PLAY:</b> NLO to check EDG operation (wait 3 min) Report "Both EDGs are operating normally"		
		Critical Tasks: (identified by √ in guide) With Reactor pressure greater than shutoff head of the low pressure systems and when RPV water level reaches TAF, <i>INITIATE</i> emergency depressurization, before level reaches Minimum Zero- Injection RPV Water Level. Action is taken to restore RPV water level above TAF by <i>OPERATING</i> available low pressure systems, when RPV pressure decreases below the shutoff head of low pressure systems.		
		Scenario Completion Criteria: - Performed Emergency Depressurization - Restored RPV level above TAF - AND, at the direction of the NRC chief examiner.		

# Dresden Generating Station

### SIMULATOR EXERCISE GUIDE

ILT 01-1 NRC EXAM

SCENARIO

ILT-N-5

Rev. 01

05/02

DEVELOPED BY:

Exam Author

APPROVED BY:

Facility Representative

Date

Date

ئى ا

Appendix	( D			Scenario Outline		Form ES-D-1
						]
Facility:	Dresden		Sce	nario No: <u>ILT-N-5</u>	Op-Tes	st No: <u>ILT 01-1</u>
Examin	ers:			Operators:		
	<u> </u>					
Initial Co	onditions: Appro	oxima	tely 78% r	eactor power; IRM channel 16	6 out of servi	ce; 2B EHC
Pump C	OS; Unit 3 is in	Mod	ə 4.	• •		
Turnove	er: Power reduc	tion i	n progress	s for drywell entry.		
				tion the second s	· • ·	
Event	Malf.	E	Event	Eve	ent	
110.	100.		ype			
1	N/A	Ν	SRO	rotating idle SDC pumps		
2	N/A	R	NSO SRO	lower reactor power by redu	cing recircula	ition flow
3	MGGH2CON	1	ANSO SRO	main generator hydrogen ter fails low	mperature co	ntroller output
4	ICTUBLK	С	ANSO SRO	isolation condenser tube lea	k	
5	N/A	с	NSO SRO	CRD pump failure		
6	RRMAFDBK	I	NSO SRO	recirculation pump controller	r speed signa	Il failure
7	CIGP1I		ALL	spurious group 1 actuation a	and reactor s	cram
8	RDHLVFPA RDHLVFPB RDHLDEGA RDHLDEGB	М	ALL	SDV partial hydraulic lock (A	ATWS)	
9	SCRLFVAD SCRLFVBD		NSO SRO	SBLC pump relief valves fail	lopen	

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

Appendix D		Operator Actions	Form ES-D-2
Op-Test No: <u>ILT 01-1</u> Event Description: The Pum		Scenario No.: <u>ILT-N-5</u> Event No.: <u>1</u> ANSO, as directed by the SRO, performs DOP 1000-06, ips.	Page <u>1</u> of <u>1</u> Rotating Idle SDC
Time	Position	Applicant's Actions or Behavior	
	SRO	Directs ANSO to perform DOP 1000-06, Rotating Idle S	SDC Pumps.
	ANSO	<ul> <li>Performs the following actions per DOP 1000-06, Rotat</li> <li>Verifies the following valves closed: <ul> <li>MO 2-1001-2A, B &amp; C.</li> <li>MO 2-1001-4A, B &amp; C.</li> </ul> </li> <li>Places each SDC pump control switch to start a places each pump control switch to stop. <ul> <li>Requests operator to confirm rotation for each</li> </ul> </li> <li>Monitors panels and assists as directed.</li> </ul>	ting Idle SDC Pumps: and then within 3 sec pump.
		ROLE PLAY:         NLO to confirm SDC pump rotation:         Report "the (requested pump) rotated".         Event 1 Completion Criteria:         - DOP 1000-06 completed.         - OR, at the direction of the NRC chief examiner.	

Op-Test No	o: <u>ILT 01-1</u>	Scenario No.: <u>ILT-N-5</u> Event No.: <u>2</u> Page <u>1</u> of <u>1</u>
Event Desc	cription: The N the S	NSO lowers reactor power by reducing recirculation flow following direction by SRO.
Time	Position	Applicant's Actions or Behavior
	NSO	<ul> <li>Performs the following actions per DGP 03-01, Routine Power Changes, and DOP 0202-03, Reactor Recirculation Flow Control System Operation: <ul> <li>Lowers recirculation pump speed using the master controller potentiometer.</li> <li>Verifies expected power reduction.</li> </ul> </li> </ul>
	SRO	Directs reducing reactor power per DGP 03-01, Routine Power Changes, and DOP 0202-03, Reactor Recirculation Flow Control System Operation, by lowering recirculation pump speed.
	ANSO	Monitors panels and assists as directed.
		Event 2 Completion Criteria: - Sufficient power reduction. - OR, at the direction of the NRC chief examiner.

Form ES-D-2

Op-Test No	o: <u>ILT 01-1</u>	Scenario No.: <u>ILT-N-5</u> Event No.: <u>3</u> Page <u>1</u> of <u>1</u>
Event Description: Alarms are then received due to high main generator hydrogen temperature resulting from a failed controller. Hydrogen temperature is restored after the controller is placed in manual and adjusted by the ANSO.		
Time	Position	Applicant's Actions or Behavior
		SIMULATOR OPERATOR: At the discretion of the NRC chief examiner, activate trigger 2, which fails the main generator hydrogen temperature controller output. Verify trigger 3 automatically activates when the main generator hydrogen temperature controller is placed to MAN.
	ANSO	<ul> <li>Performs the following actions per DAN 902-7 E-11, H2 Seal Oil &amp; Alterrex</li> <li>Pnl Trouble and DAN 2252-7 A-8,.</li> <li>Directs NLO to local panel 2252-7 to determine alarm received.</li> <li>Diagnosis that the main generator hydrogen cooler temperature controller auto mode has failed and places in MAN mode and restores temperature to normal band.</li> </ul>
		SIMULATOR OPERATOR / ROLE PLAY: NLO to panel 2252-7 (wait 2 min): Activate trigger 4 then Report "the alarm is A-8, Machine Gas Temperature High, and I have acknowledged it".
		NLO to check hydrogen cooler TCV operation: (wait 1 min) Report "the hydrogen cooler TCV appears to be operating normally".
	SRO	Directs ANSO to take manual control of the main generator hydrogen temperature controller.
		Notifies Shift Manager and IMD of controller problem.           ROLE PLAY:           Respond as individual notified.
	NSO	Monitors panels and assists as directed.
		<ul> <li>Event 3 Completion Criteria:</li> <li>Takes Manual control of the main generator hydrogen temperature controller.</li> <li>OR, at the direction of the NRC chief examiner.</li> </ul>

Op-Test No: <u>ILT 01-1</u>		Scenario No.: <u>ILT-N-5</u> Event No.: <u>4</u> Page <u>1</u> of <u>2</u>
Event Description: Alarm conde		ns are received due to an isolation condenser tube leak. The isolation enser is manually isolated by the ANSO. The SRO addresses the technical
	spec	fication requirements for the inoperable isolation condenser.
Time	Position	Applicant's Actions or Behavior
		SIMULATOR OPERATOR: At the discretion of the NRC chief examiner, activate trigger 5 which inserts an Isolation Condenser tube leak (0.5%).
	ANSO	Performs the following actions per DAN 902-3 C-4, Isol Condr Temp Hi, and / or DOA 1300-01, Isolation Condenser Tube Leak: May try to reseat the MO 2-1301-3 valve: Closes MO 2-1301-4 Opens MO 2-1301-3 Closes MO 2-1301-3 Opens MO 2-1301-4 Opens AO 2-1301-17 & 20 Checks indication that leak has stopped Isolates the Isolation Condenser by closing: MO 2-1301-1 MO 2-1301-2 MO 2-1301-4 AO 2-1301-4 AO 2-1301-4
	SRO	<ul> <li>MO 2-4399-74</li> <li>Enters DOA 1300-01, Isolation Condenser Tube Leak, and directs: <ul> <li>May try reseating the MO 2-1301-3</li> <li>Isolating the Isolation Condenser.</li> <li>Radiation Protection to survey below the IC vent.</li> <li>Security to limit access below IC vent.</li> <li>Chemistry Department to sample IC shell side for activity.</li> <li>Declares IC inoperable.</li> </ul> </li> </ul>
		References Technical Specifications and determines: <ul> <li>ITS 3.5.3.A.1; verifies HPCI system operable immediately.</li> <li>ITS 3.5.3.A.2; restore IC system to OPERABLE within 14 days</li> </ul>
		Notifies Shift Manager and MMD of IC tube leak.
-	NSO	Monitors panels and assists as directed.

Op-Test No: <u>ILT 01-1</u>		Scenario No.: <u>ILT-N-5</u>	Event No.: <u>4</u>	Page <u>2</u> of <u>2</u>
Event Des	cription: Alar con spe	ms are received due to an isolation denser is manually isolated by the cification requirements for the ino	on condenser tube leak. T e ANSO. The SRO addres perable isolation condens	The isolation sses the technical er.
Time Position		Applicant's Actions or Behavio	r	
		ROLE PLAY: Respond as departments cont	acted.	
		When Isolation Condenser she the control room as Security an on the south side of the reacto	ell side temperature reach nd report "there is steam o r building".	es 212 deg. F, phone coming out of a pipe
		After the above report, phone t report "radiation levels are slig in the reactor building.	he control room as Radia htly elevated on the isolati	tion Protection and on condenser level
		Event 4 Completion Criteria: – Isolation Condenser isolate – Technical Specifications re – OR, at the direction of the	ed. eferenced. NRC chief examiner.	

**Operator Actions** 

Form ES-D-2

Op-Test No	o: <u>ILT 01-1</u>	Scenario No.: <u>ILT-N-5</u> Event No.: <u>5</u> Page <u>1</u> of <u>1</u>
Event Description: A field report is received that the 2B CRD pump is failing due to rapid oil loss from a leak. The NSO shutdowns the 2B CRD pump and starts the 2A CRD pump.		
Time	Position	Applicant's Actions or Behavior
		<b>ROLE PLAY:</b> At the discretion of the NRC chief examiner, report as NLO that "the 2B CRD pump is rapidly losing oil from the pump outboard bearing".
	NSO	<ul> <li>Performs the following actions per DOP 0300-01, Control Rod Drive System Startup and Operation:</li> <li>May direct the NLO to perform pre-startup checks per the procedure.</li> <li>Verifies 2A CRD pump discharge valve MO 2-0301-2A open.</li> <li>Starts 2A CRD pump.</li> <li>Stops 2B CRD pump</li> <li>Verifies charging water pressure between 1450 to 1500 psig.</li> <li>Directs NLO to perform post-startup checks per the procedure.</li> </ul>
		ROLE PLAY: NLO to perform 2A CRD pump pre-startup checks (wait 3 min): Report "I have completed the pre-startup checks for 2A CRD pump". NLO to perform 2A CRD pump post-startup checks (wait 3 min): Report "I have completed the post-startup checks for 2A CRD pump".
	SRO	Directs NSO to swap CRD pumps per DOP 0300-01, Control Rod Drive System Startup and Operation;
		Control Rod Drive System Failure.
	ANSO	Monitors panels and assists as directed.
		Event 5 Completion Criteria: - CRD pumps swapped. - OR, at the direction of the NRC chief examiner.

Op-Test No	): <u>ILT 01-1</u>	Scenario No.: <u>ILT-N-5</u> Event No.: <u>6</u> Page <u>1</u> of <u>2</u>	
Event Desc	Event Description: The speed control signal fails low for recirculation pump 2A and the pump flow increase is stopped when the NSO locks out the scoop tube.		
Time	Position	Applicant's Actions or Behavior	
		SIMULATOR OPERATOR: At the discretion of the NRC chief examiner, activate trigger 6, which fails the 2A Recirculation Pump speed feedback signal.	
	NSO	<ul> <li>Performs the following actions per DAN 902-4 E-6, 2A/B Recirc PPs Speed Mismatch, and DOA 0202-03, Reactor Recirculation System Flow Control Failure:</li> <li>Places the 2A M-G Set Scoop Tube Power Lockout Reset Switch in the Lockout position.</li> <li>Verifies Core thermal power &lt;2927 MWth.</li> <li>Completes actions of 2A Recirc M-G Lockout in DOP 0202-12, Recirculation Pump Motor Generator Set Scoop Tube Operation.</li> <li>Places both recirc pump speed control transfer stations to manual.</li> <li>Runs 2A Recirc M-G Set speed demand to minimum.</li> <li>Places caution card on its Lockout Reset switch.</li> <li>Coordinates with licensed operator at the 2A Recirc M-G Set Scoop Tube to lower its speed to match 2B Recirc M-G Set per DOP 0202- 12, Recirculation Pump Motor Generator Set Scoop Tube Operation.</li> </ul>	
	SRO	SIMULATOR OPERATOR / ROLE PLAY:         Licensed Operator to lower 2A Recirc M-G Set speed locally (wait 3 min)         Report "I am ready to begin lowering 2A Recirc M-G Set speed". When directed to lower the speed, toggle trigger 7 active and then inactive about every 10 seconds to "bump" the speed down until the NSO directs stopping the speed drop.         Enters and directs performance of DOA 0202-03, Reactor Recirculation System Flow Control Failure.         Enters and directs performance of DGA 7, Unpredicted Reactivity Addition.         Contacts QNE.         May request NLO to take local speed readings         Directs NSO to coordinate with a licensed operator at the 2A Recirc M-G Set Scoop Tube to lower its speed to match 2B Recirc M-G Set.         Notifies Shift Manager and IMD of controller problem.	
	ANSO	<ul> <li>Begins working through the steps of DGA 7, Unpredicted Reactivity Addition, but will not have time to complete the required actions.</li> </ul>	

Op-Test No: <u>ILT 01-1</u> Event Description: The		Scenario No.: <u>ILT-N-5</u> Event No.: <u>6</u> Page <u>2</u> of <u>2</u> speed control signal fails low for recirculation pump 2A and the pump flow
		ease is stopped when the NSO locks out the scoop tube.
Time	Position Applicant's Actions or Behavior	
		ROLE PLAY:
		NLO to obtain local speed of the 2A Recirc MG Set: (Wait 5 min for initial reading, 1 min for subsequent)
		Take variable RRNMGGEN(1) times 1150 RPM and report the result as the 2A Recirc MG Set speed.
		NLO to obtain local speed of the 2B Recirc MG Set: (Wait 1 min)
		Take variable RRNMGGEN(2) times 1150 RPM and report the result as the 2B Recirc MG Set speed.
		Event 6 Completion Criteria: - Efforts in progress to lower 2A Recirc M-G Set speed. - OR, at the direction of the NRC chief examiner.

Op-Test No: <u>ILT 01-1</u>

Scenario No.: <u>ILT-N-5</u>

Event No.: 7,8 & 9 Page 1 of 4

Event Description: A spurious group 1 isolation and a reactor scram occurs. A partial hydraulic lock of the scram discharge volume results in an ATWS. The SBLC system fails due pump relief valves failing open. The crew initiates alternate SBLC injection. Time Position Applicant's Actions or Behavior SIMULATOR OPERATOR: At the discretion of the NRC chief examiner, activate trigger 8, which inserts a spurious Group 1 Isolation and SBLC relief valve setpoint drift. ROLE PLAY: Five minutes after the start of this event, call the Control Room as an IMD Supervisor and report "While checking the Main Steam Line Flow instruments in the LPCI corner room, I slipped and fell into the instrument rack. I do not require medical attention". NSO Performs the following actions per DGP 02-03, Reactor Scram: Presses scram pushbuttons Π Π Places mode switch in shutdown Π Checks rods inserted: discovers ATWS condition Π Initiates ARI Π Verifies recirc pump speed at minimum. SRO Enters DEOP 0100, RPV Control. When receives report that ATWS condition exists, exits DEOP 100 and enters DEOP 400-5, Failure to Scram, and directs the following:  $\Box = \sqrt{Placing ADS}$  to inhibit Placing both CS pumps in PTL Power Lea Π Running back recirc to minimum. Π Tripping the recirc pumps  $\square$   $\sqrt{\text{Directing Alternate Rod Insertion per DEOP 500-05:}}$ manually driving rods. repeated scram/resets. Initiating SBLC before SP/T of 110°F. Level Leg Verification water level instruments are accurate Π Verification any required automatic actions have occurred Directing jumpers installed for MSIV low level and Off Gas high Rad isolations Π  $\sqrt{1}$  Terminating and Preventing injection except boron and CRD until RPV/L is  $\leq$  -35 inches. Π If SP/T is above 110°F, lets level drop until: Power is below 6%, OR Level drops to -143 in. (TAF), OR All ADSVs stay closed and PC/P stays below 2 psig Π  $\sqrt{Re}$ -establishing injection to *MAINTAIN* RPV water level above -164 inches. Pressure Leg Dropping RPV/P to 945 psig using ADSVs Maintaining <1060 psig using HPCI/ADSVs</p>

Op-Test No: <u>ILT 01-1</u>

Scenario No.: <u>ILT-N-5</u>

Event No.: <u>7,8 & 9</u> Page <u>2</u> of <u>4</u>

Event Description: A spurious group 1 isolation and a reactor scram occurs. A partial hydraulic lock of the scram discharge volume results in an ATWS. The SBLC system fails due pump relief valves failing open. The crew initiates alternate SBLC injection.

Time	Position	Applicant's Actions or Behavior	
	NSO / ANSO	<ul> <li>Performs DEOP 400-5, Failure to Scram, actions as directed: <ul> <li>√ Places ADS to inhibit</li> <li>Places both CS pumps in PTL</li> </ul> </li> <li>Power Leg <ul> <li>Trips recirculation pumps</li> <li>√ Performs Alternate Rod Insertion. (see below for specific actions)</li> <li>Initiates boron injection. Reports SBLC has failed to inject.</li> </ul> </li> <li>Level Leg <ul> <li>√ Terminates and Prevents injection except boron and CRD until RPV/L is ≤ -35 inches.</li> <li>If SP/T is above 110°F, lets level drop until: <ul> <li>Power is below 6%, OR</li> <li>Level drops to -143 in. (TAF), OR</li> <li>All ADSVs stay closed and PC/P stays below 2 psig</li> <li>√ Re-establishes injection to MAINTAIN RPV water level above -164 inches.</li> </ul> </li> <li>Pressure Leg <ul> <li>Drops RPV/P to 945 psig ADSVs</li> <li>Maintains &lt;1060 psig using HPCI/ADSVs</li> </ul> </li> </ul></li></ul>	
	NSO	<ul> <li>√ Performs manual control rod insertion per DEOP 500-05, Alternate Insertion of Control Rods, as directed:         <ul> <li>Bypasses the RWM</li> <li>Starts the second CRD pump</li> <li>Maximizes drive water pressure using one or more of the methods in DEOP 500-05.</li> <li>Inserts rods using RONOR in EMERG IN or the normal rod movement control switch</li> </ul> </li> </ul>	
	NSO	<ul> <li>√ Performs repeated scram/resets per DEOP 500-05, Alternate Insertion of Control Rods, as directed:</li> <li>Depresses close pushbuttons for SDV vent and drain valves</li> <li>If RPV/L &lt;-59 inches, directs pulling ARI fuses.</li> <li>Attempts to reset scram</li> <li>Directs scram jumpers installed.</li> <li>Resets the scram</li> <li>Verifies all scram valves closed</li> <li>Opens the SDV vent and drains</li> <li>When 902-5 C-1 clears, scrams reactor</li> <li>Repeats as necessary</li> </ul>	

Scenario No.: <u>ILT-N-5</u>

Event No.: <u>7,8 & 9</u> Page <u>3</u> of <u>4</u>

Event Description: A spurious group 1 isolation and a reactor scram occurs. A partial hydraulic lock of the scram discharge volume results in an ATWS. The SBLC system fails due pump relief valves failing open. The crew initiates alternate SBLC injection. Time Position Applicant's Actions or Behavior SIMULATOR OPERATOR / ROLE PLAY: Operator to jumper the MSIV Group1-59 in. and offgas hi hi radiation isolations (wait 5 min); Activate trigger 9 and report "the MSIV Group1-59 in. and offgas hi hi radiation isolations are jumpered". SIMULATOR OPERATOR / ROLE PLAY: Operator to pull ARI fuses (wait 5 min); Verify trigger 10 activated and report "the ARI fuses are pulled". Operator to install scram jumpers (wait 5 min): Verify trigger 11 activated and report "the scram jumpers are installed". SRO Π Based on failure of SBLC to inject, directs performance of DEOP 0500-01, Alternate Standby Liquid Control Injection. SRO Enters DEOP 200-1, Primary Containment Control, when PC/P reaches 2 psig and performs/directs: May direct starting Drywell Coolers per DEOP 0500-02. Initiation of torus sprays before PC/P of 9 psig П When PC/P is above 9 psig or before DW/T reaches 281°F: (may not reach these levels) Verification of DSIL Tripping of recirc pumps Tripping of DW coolers Initiation of DW sprays Initiation of torus cooling Π Π Verifies initiation of drywell and torus H₂/O₂ monitors ANSO Performs DEOP 200-1, Primary Containment Control, actions as directed: May start Drywell Coolers after jumpers installed per DEOP 0500-02. Π May initiate torus sprays and drywell sprays per Hard Card LPCI/CCSW OPERATION, as directed Π Initiates torus cooling per Hard Card LPCI/CCSW OPERATION as directed Π Verifies initiation of drywell and torus H₂/O₂ monitors SIMULATOR OPERATOR / ROLE PLAY: Operator to install jumpers to defeat DW Cooler trips (wait 5 min): Verify trigger 12 activated and report "the DW Cooler trip jumpers are installed".

Op-Test No	o: <u>ILT 01-1</u>	Scenario No.: <u>ILT-N-5</u> Event No.: <u>7, 8 &amp; 9</u> Page <u>4</u> of <u>4</u>
Event Description: A spurious group 1 isolation and a reactor scram occurs. A partial hydraulic lock of the scram discharge volume results in an ATWS. The SBLC system fails due pum relief valves failing open. The crew initiates alternate SBLC injection.		
Time	Position	Applicant's Actions or Behavior
		SIMULATOR OPERATOR: After the crew has reset the scram and at the discretion of the NRC chief examiner, RUN CAEP File ClrHydLk.cae which will remove the SDV hydraulic lock.
	SRO	<ul> <li>Based on report that all rods are inserted, exits DEOP 400-05, Failure to Scram, and enters DEOP100, RPV Control and directs:</li> <li>Securing Boron injection if any was started.</li> <li>Restoring RPV level to +8 to +48 inches.</li> <li>Develop a cooldown strategy (i.e.; reopen MSIVs and/or restart RWCU)</li> </ul>
	NSO	Reports that all rods inserted and performs the following as directed: Restores level to +8 to +48 inches.
		<ul> <li>Critical Tasks: (identified by √ in guide)</li> <li>With a reactor scram required and the reactor not shutdown, <i>TAKE</i> ACTION TO REDUCE POWER by injecting boron and/or inserting control rods, to prevent exceeding the primary containment design limits.</li> <li>With a reactor scram required, reactor not shutdown, and conditions for ADS blowdown are met, <i>INHIBIT ADS</i> to prevent an uncontrolled RPV depressurization, to prevent causing a significant power excursion.</li> <li>During an ATWS with conditions met to perform power/level control <i>TERMINATE AND PREVENT INJECTION</i>, with exception of boron and CRD, into the RPV until conditions are met to re-establish injection.</li> <li>When conditions are met to re-establish injection use available injection systems to <i>MAINTAIN</i> RPV water level above -164".</li> </ul>
		Scenario Completion Criteria: Control rods inserted. RPV level and pressure stabilized. OR, at the direction of the NRC chief examiner.

Īī