

5.4 SHIELDING EVALUATION

The MCNP-4A code was used for all of the shielding analyses [5.1.1]. MCNP is a continuous energy, three-dimensional, coupled neutron-photon-electron Monte Carlo transport code. Continuous energy cross section data are represented with sufficient energy points to permit linear-linear interpolation between points. The individual cross section libraries used for each nuclide are those recommended by the MCNP manual. All of these data are based on ENDF/B-V data. MCNP has been extensively benchmarked against experimental data by the large user community. References [5.4.2], [5.4.3], and [5.4.4] are three examples of the benchmarking that has been performed.

The energy distribution of the source term, as described earlier, is used explicitly in the MCNP model. A different MCNP calculation is performed for each of the three source terms (neutron, decay gamma, and ^{60}Co). The axial distribution of the fuel source term is described in Table 2.1.11 and Figures 2.1.3 and 2.1.4. The PWR and BWR axial burnup distributions were obtained from References [5.4.5] and [5.4.6], respectively. These axial distributions were obtained from operating plants and are representative of PWR and BWR fuel with burnups greater than 30,000 MWD/MTU. The ^{60}Co source in the hardware was assumed to be uniformly distributed over the appropriate regions.

It has been shown that the neutron source strength varies as the burnup level raised by the power of 4.2. Since this relationship is non-linear and since the burnup in the axial center of a fuel assembly is greater than the average burnup, the neutron source strength in the axial center of the assembly is greater than the relative burnup times the average neutron source strength. In order to account for this effect, the neutron source strength in each of the 10 axial nodes listed in Table 2.1.11 was determined by multiplying the average source strength by the relative burnup level raised to the power of 4.2. The peak relative burnups listed in Table 2.1.11 for the PWR and BWR fuels are 1.105 and 1.195 respectively. Using the power of 4.2 relationship results in a 37.6% ($1.105^{4.2}/1.105$) and 76.8% ($1.195^{4.2}/1.195$) increase in the neutron source strength in the peak nodes for the PWR and BWR fuel respectively. The total neutron source strength increases by 15.6% for the PWR fuel assemblies and 36.9% for the BWR fuel assemblies.

MCNP was used to calculate doses at the various desired locations. MCNP calculates neutron or photon flux and these values can be converted into dose by the use of dose response functions. This is done internally in MCNP and the dose response functions are listed in the input file in Appendix 5.C. The response functions used in these calculations are listed in Table 5.4.1 and were taken from ANSI/ANS 6.1.1, 1977 [5.4.1].

The HI-STORM shielding analysis was performed for conservative burnup and cooling time combinations which bound the uniform and regionalized loading specifications for zircaloy clad fuel specified in Appendix B to the CoC. Therefore, the HI-STORM shielding analysis presented in this chapter is conservatively bounding for the MPC-24, MPC-32, and MPC-68.

Tables 5.1.1 through 5.1.3 provide the maximum dose rates adjacent to the HI-STORM overpack during normal conditions for each of the MPCs. Tables 5.1.4 through 5.1.6 provide the maximum dose rates at one meter from the overpack. A detailed discussion of the normal, off-normal, and accident condition dose rates is provided in Sections 5.1.1 and 5.1.2.

Tables 5.1.7 and 5.1.8 provide dose rates for the 100-ton and 125-ton HI-TRAC transfer casks, respectively, with the MPC-24 loaded with design basis fuel in the normal condition, in which the MPC is dry and the HI-TRAC water jacket is filled with water. Table 5.4.2 shows the corresponding dose rates adjacent to and one meter away from the 100-ton HI-TRAC for the fully flooded MPC condition with an empty water-jacket (condition in which the HI-TRAC is removed from the spent fuel pool). Table 5.4.3 shows the dose rates adjacent to and one meter away from the 100-ton HI-TRAC for the fully flooded MPC condition with the water jacket filled with water (condition in which welding operations are performed). Dose locations 4 and 5, which are on the top and bottom of the HI-TRAC were not calculated at the one-meter distance for these configurations. For the conditions involving a fully flooded MPC, the internal water level was 10 inches below the MPC lid. These dose rates represent the various conditions of the HI-TRAC during operations. Comparing these results to Table 5.1.7 indicates that the dose rates in the upper and lower portions of the HI-TRAC are reduced by about 50% with the water in the MPC. The dose at the center of the HI-TRAC is reduced by approximately 50% when there is also water in the water jacket and is essentially unchanged when there is no water in the water jacket as compared to the normal condition results shown in Table 5.1.7.

The burnup and cooling time combination of 42,500 MWD/MTU and 5 years was selected for the 100-ton MPC-24 HI-TRAC analysis because this combination of burnup and cooling time results in the highest dose rates, and therefore, bounds all other requested combinations in the 100-ton HI-TRAC. For comparison, dose rates corresponding to a burnup of 52,500 MWD/MTU and 10 year cooling time for the MPC-24 are provided in Table 5.4.4. The dose rate at 1 meter from the pool lid was not calculated because a concrete floor was placed 6 inches below the pool lid to account for potential ground scattering. These results clearly indicate that as the burnup and cooling time increase, the reduction in the gamma dose rate due to the increased cooling time results in a net decrease in the total dose rate. This result is due to the fact that the dose rates surrounding the 100-ton HI-TRAC transfer cask are gamma dominated.

In contrast, the dose rates surrounding the HI-TRAC 125 and 125D transfer casks have significantly higher neutron component. Therefore, the dose rates at 57,500 MWD/MTU burnup and 12 year cooling are slightly higher than the dose rates at 42,500 MWD/MTU burnup and 5 year cooling. The dose rates for the 125-ton HI-TRACs with the MPC-24 at 57,500 MWD/MTU and 12 year cooling are listed in Table 5.1.8 of Section 5.1. For comparison, dose rates corresponding to a burnup of 42,500 MWD/MTU and 5 year cooling time for the MPC-24 are provided in Table 5.4.5.

Tables 5.4.9 and 5.4.10 provide dose rates adjacent to and one meter away from the 100-ton HI-TRAC with the MPC-68 at burnup and cooling time combinations of 40,000 MWD/MTU and

5 years and 50,000 MWD/MTU and 10 years, respectively. The dose rate at 1 meter from the pool lid was not calculated because a concrete floor was placed 6 inches below the pool lid to account for potential ground scattering. These results demonstrate that the dose rates on contact at the top and bottom of the 100-ton HI-TRAC are somewhat higher in the MPC-68 case than in the MPC-24 case. However, the MPC-24 produces higher dose rates than the MPC-68 at the center of the HI-TRAC, on-contact, and at locations 1 to 2 feet away from the HI-TRAC. Therefore, the MPC-24 is still used for the exposure calculations in Chapter 10 of the FSAR.

Tables 5.4.11 and 5.4.12 provide dose rates adjacent to and one meter away from the 100-ton HI-TRAC with the MPC-32 at burnup and cooling time combinations of 32,500 MWD/MTU and 5 years and 45,000 MWD/MTU and 10 years, respectively. The dose rate at 1 meter from the pool lid was not calculated because a concrete floor was placed 6 inches below the pool lid to account for potential ground scattering. These results demonstrate that the dose rates on contact at the top and bottom of the 100-ton HI-TRAC are somewhat higher in the MPC-32 case than in the MPC-24 case. However, the MPC-24 produces comparable or higher dose rates than the MPC-32 at the center of the HI-TRAC, on-contact, and at locations 1 to 2 feet away from the HI-TRAC. Therefore, the MPC-24 is still used for the exposure calculations in Chapter 10 of the FSAR.

As mentioned in Section 5.0, all MPCs offer a regionalized loading pattern as described in Appendix B to the CoC. This loading pattern authorizes fuel of higher decay heat than uniform loading (i.e. higher burnups and shorter cooling times) to be stored in the center region, region 1, of the MPC. The outer region, region 2, of the MPC in regionalized loading is authorized to store fuel of lower decay heat than uniform loading (i.e. lower burnups and longer cooling times). From a shielding perspective, the older fuel on the outside provides shielding for the inner fuel in the radial direction. Regionalized patterns were specifically analyzed in each MPC in the 100-ton HI-TRAC. Based on analysis using the same burnup and cooling times in region 1 and 2 the following percentages were calculated for dose location 2 on the 100-ton HI-TRAC.

- Approximately 21%, 27%, and 8% of the neutron dose at the edge of the water jacket comes from region 1 fuel assemblies in the MPC-32, MPC-68, and MPC-24 respectively. Region 1 contains 12 (38% of total), 32 (47% of total), and 4 (17% of total) assemblies in the MPC-32, MPC-68, and MPC-24 respectively.
- Approximately 1%, 2%, and 0.2% of the photon dose at the edge of the water jacket comes from region 1 fuel assemblies in the MPC-32, MPC-68, and MPC-24 respectively.

These results clearly indicate that the outer fuel assemblies shield almost all of the gamma source from the inner assemblies in the radial direction and a significant percentage of the neutron source. The conclusion from this analysis is that the total dose rate on the external radial surfaces of the cask can be greatly reduced by placing longer cooled and lower burnup fuels on the outside of the basket. In the axial direction, regionalized loading results in higher dose rates

in the center portion of the cask since the region 2 assemblies are not shielding the region 1 assemblies for axial dose locations.

All burnup and cooling time combinations for regionalized loading were analyzed and compared to the dose rates from uniform loading patterns. It was concluded that, in general, the radial dose rates from regionalized loading are bounded by the radial dose rates from uniform loading patterns. Therefore, dose rates for specific regionalized loading patterns are not presented in this chapter. In the axial direction, the reverse may be true since the inner fuel assemblies in a regionalized loading pattern have a higher burnup than the assemblies in the uniform loading patterns. However, as depicted in the graphical data in Section 5.1.1, the dose rate along the pool or transfer lids decrease substantially moving radially outward from the center of the lid. Therefore, this increase in the dose rate in the center of the lids due to regionalized loading does not significantly impact the occupational exposure. Section 5.4.9 provides additional discussion on regionalized loading dose rates compared to uniform loading dose rates.

Unless otherwise stated all tables containing dose rates for design basis fuel refer to design basis intact zircaloy clad fuel.

Since MCNP is a statistical code, there is an uncertainty associated with the calculated values. In MCNP the uncertainty is expressed as the relative error which is defined as the standard deviation of the mean divided by the mean. Therefore, the standard deviation is represented as a percentage of the mean. The relative error for the total dose rates presented in this chapter were typically less than 5% and the relative error for the individual dose components was typically less than 10%.

5.4.1 Streaming Through Radial Steel Fins and Pocket Trunnions and Azimuthal Variations

The HI-STORM 100 overpack and the HI-TRAC utilize radial steel fins for structural support and cooling. The attenuation of neutrons through steel is substantially less than the attenuation of neutrons through concrete and water. Therefore, it is possible to have neutron streaming through the fins that could result in a localized dose peak. The reverse is true for photons, which would result in a localized reduction in the photon dose. In addition to the fins, the pocket trunnions in the HI-TRAC 100 and 125 are essentially blocks of steel that are approximately 12 inches wide and 12 inches high. The effect of the pocket trunnion on neutron streaming and photon transmission will be more substantial than the effect of a single fin.

Analysis of the pocket trunnions in the HI-TRAC 100 and 125 and the steel fins in the HI-TRAC 100, 125, and 125D indicate that neutron streaming is noticeable at the surface of the transfer cask. The neutron dose rate on the surface of the pocket trunnion is approximately 5 times higher than the circumferential average dose rate at that location. The gamma dose rate is approximately 10 times lower than the circumferential average dose rate at that location. The streaming at the rib location is the largest in the HI-TRAC 125D because the ribs are thicker than in the HI-TRAC 100 or 125. The neutron dose rate on the surface of the rib in the 125D is

approximately 3 times higher than the circumferential average dose rate at that location. The gamma dose rate on the surface of the rib in the 125D is approximately 3 times lower than the circumferential average dose rate at that location. At one meter from the cask surface there is little difference between the dose rates calculated over the fins and the pocket trunnions compared to the other areas of the water jackets.

These conclusions indicate that localized neutron streaming is noticeable on the surface of the transfer casks. However, at one meter from the surface the streaming has dissipated. Since most HI-TRAC operations will involve personnel moving around the transfer cask at some distance from the cask only surface average dose rates are reported in this chapter.

Below each lifting trunnion, there is a localized area where the water jacket has been reduced in height by 4.125 inches to accommodate the lift yoke (see Figures 5.3.12 and 5.3.13). This area experiences a significantly higher than average dose rate on contact of the HI-TRAC. The peak dose in this location is 1.5 Rem/hr for the MPC-32, 1.4 Rem/hr for the MPC-68 and 1.3 Rem/hr for the MPC-24 in the 100-ton HI-TRAC and 649 mrem/hr for the MPC-24 in the HI-TRAC 125D. At a distance of 1 to 2 feet from the edge of the HI-TRAC the localized effect is greatly reduced. This dose rate is acceptable because during lifting operations the lift yoke will be in place, which, due to the additional lift yoke steel (~3 inches), will greatly reduce the dose rate. However, more importantly, people will be prohibited from being in the vicinity of the lifting trunnions during lifting operations as a standard rigging practice. In addition the lift yoke is remote in its attachment and detachment, further minimizing personnel exposure. Immediately following the detachment of the lift yoke, in preparation for closure operations, temporary shielding may be placed in this area. Any temporary shielding (e.g., lead bricks, water tanks, lead blankets, steel plates, etc.) is sufficient to attenuate the localized hot spot. The operating procedure in Chapter 8 discusses the placement of temporary shielding in this area. For the 100-ton HI-TRAC, the optional temporary shield ring will replace the water that was lost from the axial reduction in the water jacket thereby eliminating the localized hot spot. When the HI-TRAC is in the horizontal position, during transport operations, it will (at a minimum) be positioned a few feet off the ground by the transport vehicle and therefore this location below the lifting trunnions will be positioned above people which will minimize the effect on personnel exposure. In addition, good operating practice will dictate that personnel remain at least a few feet away from the transport vehicle. During vertical transport of a loaded HI-TRAC, the localized hot spot will be even further from the operating personnel. Based on these considerations, the conclusion is that this localized hot spot does not significantly impact the personnel exposure.

5.4.2 Damaged Fuel Post-Accident Shielding Evaluation

5.4.2.1 Dresden 1 and Humboldt Bay Damaged Fuel

As discussed in Section 5.2.5.2, the analysis presented below, even though it is for damaged fuel, demonstrates the acceptability of storing intact Humboldt Bay 6x6 and intact Dresden 1 6x6 fuel assemblies.

For the damaged fuel and fuel debris accident condition, it is conservatively assumed that the damaged fuel cladding ruptures and all the fuel pellets fall and collect at the bottom of the damaged fuel container. The inner dimension of the damaged fuel container, specified in the Design Drawings of Chapter 1, and the design basis damaged fuel and fuel debris assembly dimensions in Table 5.2.2 are used to calculate the axial height of the rubble in the damaged fuel container assuming 50% compaction. Neglecting the fuel pellet to cladding inner diameter gap, the volume of cladding and fuel pellets available for deposit is calculated assuming the fuel rods are solid. Using the volume in conjunction with the damaged fuel container, the axial height of rubble is calculated to be 80 inches.

Dividing the total fuel gamma source for a 6x6 fuel assembly in Table 5.2.7 by the 80 inch rubble height provides a gamma source per inch of $3.41\text{E}+12$ photon/s. Dividing the total neutron source for a 6x6 fuel assembly in Table 5.2.18 by 80 inches provides a neutron source per inch of $2.75\text{E}+05$ neutron/s. These values are both bounded by the BWR design basis fuel gamma source per inch and neutron source per inch values of $1.08\text{E}+13$ photon/s and $9.17\text{E}+05$ neutron/s, respectively. These BWR design basis values were calculated by dividing the total source strengths for 40,000 MWD/MTU and 5 year cooling in Tables 5.2.6 and 5.2.17 by the active fuel length of 144 inches. Therefore, damaged Dresden 1 and Humboldt Bay fuel assemblies are bounded by the design basis intact BWR fuel assembly for accident conditions. No explicit analysis of the damaged fuel dose rates from Dresden 1 or Humboldt Bay fuel assemblies are provided as they are bounded by the intact fuel analysis.

5.4.2.2 Generic PWR and BWR Damaged Fuel

The Holtec Generic PWR and BWR DFCs are designed to accommodate any PWR or BWR fuel assembly that can physically fit inside the DFC. Damaged fuel assemblies under normal conditions, for the most part, resemble intact fuel assemblies from a shielding perspective. Under accident conditions, it can not be guaranteed that the damaged fuel assembly will remain intact. As a result, the damaged fuel assembly may begin to resemble fuel debris in its possible configuration after an accident.

Since damaged fuel is identical to intact fuel from a shielding perspective no specific analysis is required for damaged fuel under normal conditions. However, a generic shielding evaluation was performed to demonstrate that fuel debris under normal or accident conditions, or damaged fuel in a post-accident configuration, will not result in a significant increase in the dose rates around

the 100-ton HI-TRAC. Only the 100-ton HI-TRAC was analyzed because it can be concluded that if the dose rate change is not significant for the 100-ton HI-TRAC then the change will not be significant for the 125-ton HI-TRACs or the HI-STORM overpacks.

Fuel debris or a damaged fuel assembly which has collapsed can have an average fuel density which is higher than the fuel density for an intact fuel assembly. If the damaged fuel assembly were to fully or partially collapse, the fuel density in one portion of the assembly would increase and the density in the other portion of the assembly would decrease. This scenario was analyzed with MCNP-4A in a conservative bounding fashion to determine the potential change in dose rate as a result of fuel debris or a damaged fuel assembly collapse. The analysis consisted of modeling the fuel assemblies in the damaged fuel locations in the MPC-24 (4 peripheral locations in the MPC-24E or MPC-24EF) and the MPC-68 (16 peripheral locations) with a fuel density that was twice the normal fuel density and correspondingly increasing the source rate for these locations by a factor of two. A flat axial power distribution was used which is approximately representative of the source distribution if the top half of an assembly collapsed into the bottom half of the assembly. Increasing the fuel density over the entire fuel length, rather than in the top half or bottom half of the fuel assembly, is conservative and provides the dose rate change in both the top and bottom portion of the cask.

Tables 5.4.13 and 5.4.14 provide the results for the MPC-24 and MPC-68, respectively. Only the radial dose rates are provided since the axial dose rates will not be significantly affected because the damaged fuel assemblies are located on the periphery of the baskets. A comparison of these results to the results in Tables 5.1.7 and 5.4.9 indicate that the dose rates in the top and bottom portion of the 100-ton HI-TRAC increase by less than 20% while the dose rate in the center of the HI-TRAC actually decreases a little bit. The increase in the bottom and top is due to the assumed flat power distribution. The dose rates shown in Tables 5.4.13 and 5.4.14 were averaged over the circumference of the cask. Since almost all of the peripheral cells in the MPC-68 are filled with DFCs, an azimuthal variation would not be expected for the MPC-68. However, since there are only 4 DFCs in the MPC-24E, an azimuthal variation in dose due to the damaged fuel/fuel debris might be expected. Therefore, the dose rates were evaluated in four smaller regions, one outside each DFC, that encompass about 44% of the circumference. There was no significant change in the dose rate as a result of the localized dose calculation. These results indicate that the potential effect on the dose rate is not very significant for the storage of damaged fuel and/or fuel debris. This conclusion is further reinforced by the fact that the majority of the significantly damaged fuel assemblies in the spent fuel inventories are older assemblies from the earlier days of nuclear plant operations. Therefore, these assemblies will have a considerably lower burnup and longer cooling times than the assemblies analyzed in this chapter.

5.4.3 Site Boundary Evaluation

NUREG-1536 [5.2.1] states that detailed calculations need not be presented since SAR Chapter 12 assigns ultimate compliance responsibilities to the site licensee. Therefore, this subsection describes, by example, the general methodology for performing site boundary dose calculations. The site-specific fuel characteristics, burnup, cooling time, and the site characteristics would be factored into the evaluation performed by the licensee.

As an example of the methodology, the dose from a single HI-STORM overpack loaded with an MPC-24 and various arrays of loaded HI-STORMs at distances equal to and greater than 100 meters were evaluated with MCNP. In the model, the casks were placed on an infinite slab of dirt to account for earth-shine effects. The atmosphere was represented by dry air at a uniform density corresponding to 20 degrees C. The height of air modeled was 700 meters. This is more than sufficient to properly account for skyshine effects. The models included either 500 or 1050 meters of air around the cask. Based on the behavior of the dose rate as a function of distance, 50 meters of air, beyond the detector locations, is sufficient to account for back-scattering. Therefore, the HI-STORM MCNP off-site dose models account for back scattering by including more than 50 meters of air beyond the detector locations for all cited dose rates. Since gamma back-scattering has an effect on the off-site dose, it is recommended that the site-specific evaluation under 10CFR72.212 include at least 50 to 100 meters of air, beyond the detector locations, in the calculational models.

The MCNP calculations of the off-site dose used a two-stage process. In the first stage a binary surface source file (MCNP terminology) containing particle track information was written for particles crossing the outer radial and top surfaces of the HI-STORM overpack. In the second stage of the calculation, this surface source file was used with the particle tracks originating on the outer edge of the overpack and the dose rate was calculated at the desired location (hundreds of meters away from the overpack). The results from this two-stage process are statistically the same as the results from a single calculation. However, the advantage of the two-stage process is that each stage can be optimized independently.

The annual dose, assuming 100% occupancy (8760 hours), at 200 meters from one cask is presented in Table 5.4.6 for the design basis burnup and cooling time analyzed. This table indicates that the dose due to neutrons is 7 % of the total dose. This is an important observation because it implies that simplistic analytical methods such as point kernel techniques may not properly account for the neutron transmissions and could lead to low estimates of the site boundary dose.

The annual dose, assuming 8760 hour occupancy, at distance from an array of casks was calculated in three steps.

1. The annual dose from the radiation leaving the side of the HI-STORM 100 overpack was calculated at the distance desired. Dose value = A.

2. The annual dose from the radiation leaving the top of the HI-STORM 100 overpack was calculated at the distance desired. Dose value = B.
3. The annual dose from the radiation leaving the side of a HI-STORM 100 overpack, when it is behind another cask, was calculated at the distance desired. The casks have an assumed 15-foot pitch. Dose value = C.

The doses calculated in the steps above are listed in Table 5.4.7 for the bounding burnup and cooling time of 52,500 MWD/MTU and 5-year cooling. Using these values, the annual dose (at the center of the long side) from an arbitrary 2 by Z array of HI-STORM 100 overpacks can easily be calculated. The following formula describes the method.

Z = number of casks along long side

$$\text{Dose} = ZA + 2ZB + ZC$$

As an example, the dose from a 2x3 array at 300 meters is presented.

1. The annual dose from the side of a single cask: Dose A = 5.20
2. The annual dose from the top of a single cask: Dose B = 6.57e-2
3. The annual dose from the side of a cask positioned behind another cask:
Dose C = 1.04

Using the formula shown above (Z=3), the total dose at 300 meters from a 2x3 array of HI-STORM overpacks is 19.11 mrem/year, assuming a 8760 hour occupancy.

An important point to notice here is that the dose from the side of the back row of casks is 16 % of the total dose. This is a significant contribution and one that would probably not be accounted for properly by simpler methods of analysis.

The results for various typical arrays of HI-STORM overpacks can be found in Section 5.1. While the off-site dose analyses were performed for typical arrays of casks containing design basis fuel, compliance with the requirements of 10CFR72.104(a) can only be demonstrated on a site-specific basis. Therefore, a site-specific evaluation of dose at the controlled area boundary must be performed for each ISFSI in accordance with 10CFR72.212. The site-specific evaluation will consider the site-specific characteristics (such as exposure duration and the number of casks deployed), dose from other portions of the facility and the specifics of the fuel being stored (burnup and cooling time).

5.4.4 Stainless Steel Clad Fuel Evaluation

Table 5.4.8 presents the dose rates at the center of the HI-STORM 100 overpack, adjacent and at one meter distance, from the stainless steel clad fuel. These dose rates, when compared to Tables 5.1.1 through 5.1.6, are similar to the dose rates from the design basis zircaloy clad fuel, indicating that these fuel assemblies are acceptable for storage.

As described in Section 5.2.3, it would be incorrect to compare the total source strength from the stainless steel clad fuel assemblies to the source strength from the design basis zircaloy clad fuel assemblies since these assemblies do not have the same active fuel length and since there is a significant gamma source from Cobalt-60 activation in the stainless steel. Therefore it is necessary to calculate the dose rates from the stainless steel clad fuel and compare them to the dose rates from the zircaloy clad fuel. In calculating the dose rates, the source term for the stainless steel fuel was calculated with an artificial active fuel length of 144 inches to permit a simple comparison of dose rates from stainless steel clad fuel and zircaloy clad fuel at the center of the HI-STORM 100 overpack. Since the true active fuel length is shorter than 144 inches and since the end fitting masses of the stainless steel clad fuel are assumed to be identical to the end fitting masses of the zircaloy clad fuel, the dose rates at the other locations on the overpack are bounded by the dose rates from the design basis zircaloy clad fuel, and therefore, no additional dose rates are presented.

5.4.5 Mixed Oxide Fuel Evaluation

The source terms calculated for the Dresden 1 GE 6x6 MOX fuel assemblies can be compared to the source terms for the BWR design basis zircaloy clad fuel assembly (GE 7x7) which demonstrates that the MOX fuel source terms are bounded by the design basis source terms and no additional shielding analysis is needed.

Since the active fuel length of the MOX fuel assemblies is shorter than the active fuel length of the design basis fuel, the source terms must be compared on a per inch basis. Dividing the total fuel gamma source for the MOX fuel in Table 5.2.22 by the 110 inch active fuel height provides a gamma source per inch of $2.36e+12$ photons/s. Dividing the total neutron source for the MOX fuel assemblies in Table 5.2.23 by 110 inches provides a neutron source strength per inch of $3.06e+5$ neutrons/s. These values are both bounded by the BWR design basis fuel gamma source per inch and neutron source per inch values of $1.08e+13$ photons/s and $9.17e+5$ neutrons/s. These BWR design basis values were calculated by dividing the total source strengths for 40,000 MWD/MTU and 5 year cooling in Tables 5.2.6 and 5.2.17 by the active fuel length of 144 inches. This comparison shows that the MOX fuel source terms are bound by the design basis source terms. Therefore, no explicit analysis of dose rates is provided for MOX fuel.

Since the MOX fuel assemblies are Dresden Unit 1 6x6 assemblies, they can also be considered as damaged fuel. Using the same methodology as described in Section 5.4.2.1, the source term for the MOX fuel is calculated on a per inch basis assuming a post accident rubble height of 80

inches. The resulting gamma and neutron source strengths are $3.25e+12$ photons/s and $4.21e+5$ neutrons/s. These values are also bounded by the design basis fuel gamma source per inch and neutron source per inch. Therefore, no explicit analysis of dose rates is provided for MOX fuel in a post accident configuration.

5.4.6 Non-Fuel Hardware

As discussed in Section 5.2.4, non-fuel hardware in the form of BPRAs, TPDs, CRAs, and APSRs are permitted for storage, integral with a PWR fuel assembly, in the HI-STORM 100 System. Since each device occupies the same location within an assembly, only one device will be present in a given assembly. BPRAs and TPDs are authorized for unrestricted storage in an MPC while the CRAs and APSRs are restricted to the center four locations in the MPC-24, MPC-24E, MPC-24EF and MPC-32. The calculation of the source term and a description of the bounding fuel devices was provided in Section 5.2.4. The dose rate due to BPRAs and TPDs being stored in a fuel assembly was explicitly calculated. Table 5.4.15 provides the dose rates at various locations on the surface and one meter from the 100-ton HI-TRAC due to the BPRAs and TPDs for the MPC-24 and MPC-32. These results were added to the totals in the other table to provide the total dose rate with BPRAs. Table 5.4.15 indicates that the dose rates from BPRAs bound the dose rates from TPDs.

As discussed in Section 5.2.4, two different configurations were analyzed for CRAs and three different configurations were analyzed for APSRs. The dose rate due to CRAs and APSRs being stored in the inner four fuel locations was explicitly calculated for dose locations around the 100-ton HI-TRAC. Tables 5.4.16 and 5.4.17 provide the results for the different configurations of CRAs and APSRs, respectively, in the MPC-24 and MPC-32. These results indicate the dose rate on the radial surfaces of the overpack due to the storage of these devices is minimal and the dose rate out the top of the overpack is essentially 0. The latter is due to the fact that CRAs and APSRs do not achieve significant activation in the upper portion of the devices due to the manner in which they are utilized during normal reactor operations. In contrast, the dose rate out the bottom of the overpack is substantial due to these devices. However, as noted in Tables 5.4.16 and 5.4.17, the dose rate at the edge of the transfer lid is almost negligible due to APSRs and CRAs. Therefore, even though the dose rates calculated (using a very conservative source term evaluation) are daunting, they do not pose a risk from an operations perspective because they are localized in nature. Section 5.1.1 provides additional discussion on the acceptability of the relatively high localized doses on the bottom of the HI-TRACs.

5.4.7 Dresden Unit 1 Antimony-Beryllium Neutron Sources

Dresden Unit 1 has antimony-beryllium neutron sources which are placed in the water rod location of their fuel assemblies. These sources are steel rods which contain a cylindrical antimony-beryllium source which is 77.25 inches in length. The steel rod is approximately 95 inches in length. Information obtained from Dresden Unit 1 characterizes these sources in the following manner: "About one-quarter pound of beryllium will be employed as a special neutron

source material. The beryllium produces neutrons upon gamma irradiation. The gamma rays for the source at initial start-up will be provided by neutron-activated antimony (about 865 curies). The source strength is approximately $1E+8$ neutrons/second."

As stated above, beryllium produces neutrons through gamma irradiation and in this particular case antimony is used as the gamma source. The threshold gamma energy for producing neutrons from beryllium is 1.666 MeV. The outgoing neutron energy increases as the incident gamma energy increases. Sb-124, which decays by Beta decay with a half life of 60.2 days, produces a gamma of energy 1.69 MeV which is just energetic enough to produce a neutron from beryllium. Approximately 54% of the Beta decays for Sb-124 produce gammas with energies greater than or equal to 1.69 MeV. Therefore, the neutron production rate in the neutron source can be specified as $5.8E-6$ neutrons per gamma ($1E+8/865/3.7e+10/0.54$) with energy greater than 1.666 MeV or $1.16E+5$ neutrons/curie ($1E+8/865$) of Sb-124.

With the short half life of 60.2 days all of the initial Sb-124 is decayed and any Sb-124 that was produced while the neutron source was in the reactor is also decayed since these neutron sources are assumed to have the same minimum cooling time as the Dresden 1 fuel assemblies (array classes 6x6A, 6x6B, 6x6C, and 8x8A) of 18 years. Therefore, there are only two possible gamma sources which can produce neutrons from this antimony-beryllium source. The first is the gammas from the decay of fission products in the fuel assemblies in the MPC. The second gamma source is from Sb-124 which is being produced in the MPC from neutron activation from neutrons from the decay of fission products.

MCNP calculations were performed to determine the gamma source as a result of decay gammas from fuel assemblies and Sb-124 activation. The calculations explicitly modeled the 6x6 fuel assembly described in Table 5.2.2. A single fuel rod was removed and replaced by a guide tube. In order to determine the amount of Sb-124 that is being activated from neutrons in the MPC it was necessary to estimate the amount of antimony in the neutron source. The O.D. of the source was assumed to be the I.D. of the steel rod encasing the source (0.345 in.). The length of the source is 77.25 inches. The beryllium is assumed to be annular in shape encompassing the antimony. Using the assumed O.D. of the beryllium and the mass and length, the I.D. of the beryllium was calculated to be 0.24 inches. The antimony is assumed to be a solid cylinder with an O.D. equal to the I.D. of the beryllium. These assumptions are conservative since the antimony and beryllium are probably encased in another material which would reduce the mass of antimony. A larger mass of antimony is conservative since the calculated activity of Sb-124 is directly proportional to the initial mass of antimony.

The number of gammas from fuel assemblies with energies greater than 1.666 MeV entering the 77.25 inch long neutron source was calculated to be $1.04E+8$ gammas/sec which would produce a neutron source of 603.2 neutrons/sec ($1.04E+8 * 5.8E-6$). The steady state amount of Sb-124 activated in the antimony was calculated to be 39.9 curies. This activity level would produce a neutron source of $4.63E+6$ neutrons/sec ($39.9 * 1.16E+5$) or $6.0E+4$ neutrons/sec/inch ($4.63E+6/77.25$). These calculations conservatively neglect the reduction in antimony and

beryllium which would have occurred while the neutron sources were in the core and being irradiated at full reactor power.

Since this is a localized source (77.25 inches in length) it is appropriate to compare the neutron source per inch from the design basis Dresden Unit 1 fuel assembly, 6x6, containing an Sb-Be neutron source to the design basis fuel neutron source per inch. This comparison, presented in Table 5.4.18, demonstrates that a Dresden Unit 1 fuel assembly containing an Sb-Be neutron source is bounded by the design basis fuel.

As stated above, the Sb-Be source is encased in a steel rod. Therefore, the gamma source from the activation of the steel was considered assuming a burnup of 120,000 MWD/MTU which is the maximum burnup assuming the Sb-Be source was in the reactor for the entire 18 year life of Dresden Unit 1. The cooling time assumed was 18 years which is the minimum cooling time for Dresden Unit 1 fuel. The source from the steel was bounded by the design basis fuel assembly. In conclusion, storage of a Dresden Unit 1 Sb-Be neutron source in a Dresden Unit 1 fuel assembly is acceptable and bounded by the current analysis.

5.4.8 Thoria Rod Canister

Based on a comparison of the gamma spectra from Tables 5.2.37 and 5.2.7 for the thoria rod canister and design basis 6x6 fuel assembly, respectively, it is difficult to determine if the thoria rods will be bounded by the 6x6 fuel assemblies. However, it is obvious that the neutron spectra from the 6x6, Table 5.2.18, bounds the thoria rod neutron spectra, Table 5.2.38, with a significant margin. In order to demonstrate that the gamma spectrum from the single thoria rod canister is bounded by the gamma spectrum from the design basis 6x6 fuel assembly, the gamma dose rate on the outer radial surface of the 100-ton HI-TRAC and the HI-STORM overpack was estimated conservatively assuming an MPC full of thoria rod canisters. This gamma dose rate was compared to an estimate of the dose rate from an MPC full of design basis 6x6 fuel assemblies. The gamma dose rate from the 6x6 fuel was higher for the 100-ton HI-TRAC and only 15% lower for the HI-STORM overpack than the dose rate from an MPC full of thoria rod canisters. This in conjunction with the significant margin in neutron spectrum and the fact that there is only one thoria rod canister clearly demonstrates that the thoria rod canister is acceptable for storage in the MPC-68 or the MPC-68F.

5.4.9 Regionalized Loading Dose Rate Evaluation

Dose rates were calculated for regionalized loading patterns for the MPC-24, MPC-32, and MPC-68 using MCNP-4A. All burnup and cooling time combinations in Appendix B to the CoC were analyzed for both uniform and regionalized loading. The dose rates for all dose locations reported in this chapter were compared for the uniform loading patterns and the regionalized loading patterns.

It was determined that for the MPC-32, all radial surface and 1 meter dose rates for regionalized loading were bounded by the uniform loading dose rates reported in this chapter. The maximum calculated surface dose rates in the axial locations for regionalized loading were less than 15% higher than the uniform dose rates reported in this chapter for the surface of the overpack. At one-meter from the overpack, dose location 4 (in the center) was the only dose location which produced a slightly higher (5%) dose rate for regionalized loading compared to uniform loading.

For the MPC-24 it was determined that the maximum calculated dose rates in the axial direction for regionalized loading were less than 21% higher than the maximum calculated dose rates for uniform loading reported in this chapter. At one meter distance, the uniform loading dose rates reported in this chapter bound the regionalized loading dose rates. In the radial direction, the uniform loading dose rates reported in this chapter bound the regionalized loading dose rates for both surface and one-meter locations.

For the MPC-68 it was determined that all radial surface and 1 meter dose rates for regionalized loading were bounded by the uniform loading dose rates reported in this chapter. The maximum calculated surface dose rates in the axial locations for regionalized loading were less than 21% higher than the uniform dose rates reported in this chapter for the surface of the overpack. At one-meter from the overpack, dose locations 4 (in the center) and 5 (transfer lid center) were the only dose locations which produced a slightly higher (5% and 1.5% respectively) dose rate for regionalized loading compared to uniform loading.

Based on these results it can be stated that regionalized loading patterns will reduce the dose rate in the radial direction by shielding the hotter fuel on the inside of the cask with colder fuel on the outside of the cask. However, in the axial direction the localized dose rates in the center of the cask may increase as a result of the regionalized loading pattern. This is a localized effect, which has dissipated at the edge of the cask, and therefore will not result in a significant increase to the occupational exposure rates. In addition, it should be mentioned that the localized increase on the bottom center of the overpack is an area where workers will normally not be present and the increase in the top center of the overpack is an area where workers minimize their stay.

Table 5.4.1

FLUX-TO-DOSE CONVERSION FACTORS
(FROM [5.4.1])

Gamma Energy (MeV)	(rem/hr)/ (photon/cm ² -s)
0.01	3.96E-06
0.03	5.82E-07
0.05	2.90E-07
0.07	2.58E-07
0.1	2.83E-07
0.15	3.79E-07
0.2	5.01E-07
0.25	6.31E-07
0.3	7.59E-07
0.35	8.78E-07
0.4	9.85E-07
0.45	1.08E-06
0.5	1.17E-06
0.55	1.27E-06
0.6	1.36E-06
0.65	1.44E-06
0.7	1.52E-06
0.8	1.68E-06
1.0	1.98E-06
1.4	2.51E-06
1.8	2.99E-06
2.2	3.42E-06

Table 5.4.1 (continued)

FLUX-TO-DOSE CONVERSION FACTORS
(FROM [5.4.1])

Gamma Energy (MeV)	(rem/hr)/ (photon/cm ² -s)
2.6	3.82E-06
2.8	4.01E-06
3.25	4.41E-06
3.75	4.83E-06
4.25	5.23E-06
4.75	5.60E-06
5.0	5.80E-06
5.25	6.01E-06
5.75	6.37E-06
6.25	6.74E-06
6.75	7.11E-06
7.5	7.66E-06
9.0	8.77E-06
11.0	1.03E-05
13.0	1.18E-05
15.0	1.33E-05

Table 5.4.1 (continued)

FLUX-TO-DOSE CONVERSION FACTORS
(FROM [5.4.1])

Neutron Energy (MeV)	Quality Factor	(rem/hr) [†] /(n/cm ² -s)
2.5E-8	2.0	3.67E-6
1.0E-7	2.0	3.67E-6
1.0E-6	2.0	4.46E-6
1.0E-5	2.0	4.54E-6
1.0E-4	2.0	4.18E-6
1.0E-3	2.0	3.76E-6
1.0E-2	2.5	3.56E-6
0.1	7.5	2.17E-5
0.5	11.0	9.26E-5
1.0	11.0	1.32E-4
2.5	9.0	1.25E-4
5.0	8.0	1.56E-4
7.0	7.0	1.47E-4
10.0	6.5	1.47E-4
14.0	7.5	2.08E-4
20.0	8.0	2.27E-4

† Includes the Quality Factor.

Table 5.4.2

DOSE RATES FOR THE 100-TON HI-TRAC FOR THE FULLY FLOODED MPC
 CONDITION WITH AN EMPTY NEUTRON SHIELD
 MPC-24 DESIGN BASIS ZIRCALOY CLAD FUEL AT
 42,500 MWD/MTU AND 5-YEAR COOLING

Dose Point [†] Location	Fuel Gammas ^{††} (mrem/hr)	⁶⁰ Co Gammas (mrem/hr)	Neutrons (mrem/hr)	Totals (mrem/hr)	Totals with BPRAs (mrem/hr)
ADJACENT TO THE 100-TON HI-TRAC					
1	11.98	208.28	17.67	237.93	240.36
2	832.91	0.49	307.58	1140.98	1329.90
3	2.87	316.93	4.29	324.09	446.27
4	11.60	240.82	0.71	253.13	348.08
5 (pool lid)	33.90	1355.73	2.42	1392.06 ^{†††}	1401.52
ONE METER FROM THE 100-TON HI-TRAC					
1	109.69	47.02	43.72	200.43	224.97
2	366.56	6.37	100.08	473.00	556.78
3	43.99	77.09	18.08	139.17	180.31

Note: MPC internal water level is 10 inches below the MPC lid.

[†] Refer to Figures 5.1.2 and 5.1.4.

^{††} Gammas generated by neutron capture are included with fuel gammas.

^{†††} Cited dose rates correspond to the cask center. Figures 5.1.6, 5.1.7, and 5.1.11 illustrate the substantial reduction in dose rates moving radially outward from the axial center of the HI-TRAC.

Table 5.4.3

DOSE RATES FOR THE 100-TON HI-TRAC FOR THE FULLY FLOODED MPC
CONDITION WITH A FULL NEUTRON SHIELD
MPC-24 DESIGN BASIS ZIRCALOY CLAD FUEL AT
42,500 MWD/MTU AND 5-YEAR COOLING

Dose Point [†] Location	Fuel Gammas ^{††} (mrem/hr)	⁶⁰ Co Gammas (mrem/hr)	Neutrons (mrem/hr)	Totals (mrem/hr)	Totals with BPRAs (mrem/hr)
ADJACENT TO THE 100-TON HI-TRAC					
1	9.94	208.39	2.30	220.63	222.58
2	484.93	0.33	20.09	505.35	612.64
3	1.72	316.17	0.40	318.29	439.86
4	11.58	240.81	0.73	253.13	348.07
5 (pool lid)	33.79	1355.87	2.05	1391.71 ^{†††}	1401.15
ONE METER FROM THE 100-TON HI-TRAC					
1	62.48	32.38	2.68	97.55	111.28
2	211.20	2.57	7.56	221.34	268.28
3	24.97	53.19	0.91	79.07	105.88

Note: MPC internal water level is 10 inches below the MPC lid.

[†] Refer to Figures 5.1.2 and 5.1.4.

^{††} Gammas generated by neutron capture are included with fuel gammas.

^{†††} Cited dose rates correspond to the cask center. Figures 5.1.6, 5.1.7, and 5.1.11 illustrate the substantial reduction in dose rates moving radially outward from the axial center of the HI-TRAC.

Table 5.4.4

DOSE RATES FROM THE 100-TON HI-TRAC FOR NORMAL CONDITIONS
MPC-24 DESIGN BASIS ZIRCALOY CLAD FUEL AT
52,500 MWD/MTU AND 10-YEAR COOLING

Dose Point Location	Fuel Gammas (mrem/hr)	(n, γ) Gammas (mrem/hr)	⁶⁰ Co Gammas (mrem/hr)	Neutrons (mrem/hr)	Totals (mrem/hr)	Totals with BPRAs (mrem/hr)
ADJACENT TO THE 100-TON HI-TRAC						
1	15.02	19.24	364.33	272.44	671.03	678.96
2	429.66	78.51	0.43	146.51	655.11	884.67
3	3.94	3.77	200.88	227.95	436.55	574.46
3 (temp)	1.80	6.71	93.11	3.66	105.29	168.56
4	9.98	1.49	161.68	280.60	453.74	569.34
4 (outer)	2.72	0.95	40.25	189.42	233.33	262.42
5 (pool lid)	74.81	27.47	1835.64	1829.84	3767.75	3827.89
5 (transfer)	192.71	1.51	2735.38	1047.89	3977.49	4067.40
5(t-outer)	44.12	0.51	264.76	413.98	723.38	743.32
ONE METER FROM THE 100-TON HI-TRAC						
1	56.59	10.34	54.16	44.29	165.38	195.54
2	189.01	23.94	4.28	54.19	271.42	373.07
3	23.56	5.76	44.99	21.26	95.57	141.47
3 (temp)	23.42	6.19	38.32	8.04	75.96	117.34
4	3.39	0.26	49.91	69.91	123.47	159.28
5 (transfer)	79.07	0.28	1117.41	292.41	1489.17	1525.50
5(t-outer)	9.82	0.89	100.72	83.95	195.39	199.01

Notes:

- Refer to Figures 5.1.2 and 5.1.4 for dose locations.
- Dose location 3(temp) represents dose location 3 with temporary shielding installed.
- Dose location 4(outer) is the radial segment at dose location 4 which is 18-30 inches from the center of the overpack.
- Dose location 5(t-outer) is the radial segment at dose location 5 (transfer lid) which is 30-42 and 54-66 inches from the center of the lid for the adjacent and one meter locations, respectively. The inner radius of the HI-TRAC is 34.375 in. and the outer radius of the water jacket is 44.375 in.
- Dose rate based on no water within the MPC. For the majority of the duration that the HI-TRAC pool lid is installed, the MPC cavity will be flooded with water. The water within the MPC greatly reduces the dose rate.

Table 5.4.5

**DOSE RATES FROM THE 125-TON HI-TRAC FOR NORMAL CONDITIONS
MPC-24 DESIGN BASIS ZIRCALOY CLAD FUEL AT
42,500 MWD/MTU AND 5-YEAR COOLING**

Dose Point Location	Fuel Gammas (mrem/hr)	(n, γ) Gammas (mrem/hr)	⁶⁰ Co Gammas (mrem/hr)	Neutrons (mrem/hr)	Totals (mrem/hr)	Totals with BPRAs (mrem/hr)
ADJACENT TO THE 125-TON HI-TRAC						
1	3.96	12.95	74.95	87.10	178.97	179.63
2	70.38	38.36	0.01	60.33	169.07	183.80
3	0.88	1.37	46.37	138.96	187.59	205.89
4	24.90	1.76	253.74	160.84	441.23	548.40
4 (outer)	3.01	1.26	31.51	3.37	39.15	52.33
5 (pool)	35.11	0.65	390.89	556.82	983.46	990.62
5 (transfer)	38.67	1.00	447.94	92.22	579.83	584.92
ONE METER FROM THE 125-TON HI-TRAC						
1	9.32	5.17	9.61	14.33	38.43	40.32
2	31.44	12.43	0.39	20.57	64.83	71.40
3	3.55	2.92	9.37	12.79	28.62	33.19
4	6.93	0.42	61.09	16.56	85.01	110.69
5 (transfer)	15.19	0.19	216.56	15.97	247.92	250.64

Notes:

- Refer to Figures 5.1.2 and 5.1.4 for dose locations.
- Dose location 4(outer) is the radial segment at dose location 4 which is 18-24 inches from the center of the overpack.
- Dose rate based on no water within the MPC. For the majority of the duration that the HI-TRAC pool lid is installed, the MPC cavity will be flooded with water. The water within the MPC greatly reduces the dose rate.

Table 5.4.6

ANNUAL DOSE AT 200 METERS FROM A SINGLE
HI-STORM OVERPACK WITH AN MPC-24 WITH DESIGN BASIS
ZIRCALOY CLAD FUEL[†]

Dose Component	52,500 MWD/MTU 5-Year Cooling (mrem/yr)
Fuel gammas ^{††}	16.52
⁶⁰ Co Gammas	2.17
Neutrons	1.50
Total	20.19

[†] 8760 hour annual occupancy is assumed.

^{††} Gammas generated by neutron capture are included with fuel gammas.

Table 5.4.7

DOSE VALUES USED IN CALCULATING ANNUAL DOSE FROM
 VARIOUS ISFSI CONFIGURATIONS
 52,500 MWD/MTU AND 5-YEAR COOLING ZIRCALOY CLAD FUEL[†]

Distance	A Side of Overpack (mrem/yr)	B Top of Overpack (mrem/yr)	C Side of Shielded Overpack (mrem/yr)
100 meters	129.0	1.59	25.80
150 meters	45.6	0.61	9.12
200 meters	19.9	0.27	3.98
250 meters	9.72	0.13	1.94
300 meters	5.20	6.57e-2	1.04
350 meters	3.05	3.35e-2	0.61
400 meters	1.75	1.77e-2	0.35

[†] 8760 hour annual occupancy is assumed.

Table 5.4.8

DOSE RATES AT THE CENTERLINE OF THE OVERPACK FOR
DESIGN BASIS STAINLESS STEEL CLAD FUEL
WITHOUT BPRAs

Dose Point [†] Location	Fuel Gammas ^{††} (mrem/hr)	⁶⁰ Co Gammas (mrem/hr)	Neutrons (mrem/hr)	Totals (mrem/hr)
MPC-24 (40,000 MWD/MTU AND 8-YEAR COOLING)				
2 (Adjacent)	36.97	0.02	1.11	38.10
2 (One Meter)	18.76	0.17	0.50	19.43
MPC-32 (40,000 MWD/MTU AND 9-YEAR COOLING)				
2 (Adjacent)	37.58	0.00	1.49	39.08
2 (One Meter)	18.74	0.25	0.58	19.57
MPC-68 (22,500 MWD/MTU AND 10-YEAR COOLING)				
2 (Adjacent)	17.79	0.01	0.10	17.90
2 (One Meter)	8.98	0.13	0.04	9.15

[†] Refer to Figure 5.1.1.

^{††} Gammas generated by neutron capture are included with fuel gammas.

Table 5.4.9

DOSE RATES FROM THE 100-TON HI-TRAC FOR NORMAL CONDITIONS
MPC-68 DESIGN BASIS ZIRCALOY CLAD FUEL AT
40,000 MWD/MTU AND 5-YEAR COOLING

Dose Point Location	Fuel Gammas (mrem/hr)	(n, γ) Gammas (mrem/hr)	⁶⁰ Co Gammas (mrem/hr)	Neutrons (mrem/hr)	Totals (mrem/hr)
ADJACENT TO THE 100-TON HI-TRAC					
1	38.21	13.11	884.16	175.11	1110.58
2	893.89	64.66	0.56	117.12	1076.23
3	2.14	1.36	535.10	74.19	612.80
3 (temp)	1.32	2.19	254.09	1.37	258.97
4	4.91	0.58	210.60	96.74	312.82
4 (outer)	1.39	0.39	55.78	57.95	115.51
5 (pool lid)	114.16	16.05	3959.24	1049.66	5139.11
5 (transfer lid)	183.52	0.75	5965.40	661.25	6810.92
5 (t-outer)	64.15	0.31	525.96	246.99	837.41
ONE METER FROM THE 100-TON HI-TRAC					
1	121.31	7.95	82.57	30.94	242.78
2	383.23	18.30	6.09	40.37	447.99
3	29.82	3.14	121.08	8.57	162.61
3 (temp)	29.78	3.28	98.08	4.11	135.26
4	1.98	0.11	70.28	20.16	92.53
5 (transfer lid)	90.70	0.32	2646.66	177.18	2914.86
5 (t-outer)	11.23	0.57	223.53	50.10	285.42

Notes:

- Refer to Figures 5.1.2 and 5.1.4 for dose locations.
- Dose location 3(temp) represents dose location 3 with temporary shielding installed.
- Dose location 4(outer) is the radial segment at dose location 4 which is 18-30 inches from the center of the overpack.
- Dose location 5(t-outer) is the radial segment at dose location 5 (transfer lid) which is 30-42 and 54-66 inches from the center of the lid for the adjacent and one meter locations, respectively. The inner radius of the HI-TRAC is 34.375 in. and the outer radius of the water jacket is 44.375 in.
- Dose rate based on no water within the MPC. For the majority of the duration that the HI-TRAC pool lid is installed, the MPC cavity will be flooded with water. The water within the MPC greatly reduces the dose rate.

Table 5.4.10

DOSE RATES FROM THE 100-TON HI-TRAC FOR NORMAL CONDITIONS
MPC-68 DESIGN BASIS ZIRCALOY CLAD FUEL AT
50,000 MWD/MTU AND 10-YEAR COOLING

Dose Point Location	Fuel Gammas (mrem/hr)	(n, γ) Gammas (mrem/hr)	⁶⁰ Co Gammas (mrem/hr)	Neutrons (mrem/hr)	Totals (mrem/hr)
ADJACENT TO THE 100-TON HI-TRAC					
1	15.11	19.37	501.33	258.76	794.57
2	367.98	95.55	0.32	173.08	636.93
3	0.75	2.02	303.41	109.64	415.81
3 (temp)	0.48	3.24	144.07	2.03	149.82
4	1.68	0.85	119.41	142.95	264.89
4 (outer)	0.49	0.57	31.63	85.63	118.32
5 (pool lid)	44.84	23.72	2244.93	1551.16	3864.65
5 (transfer lid)	83.23	1.11	3382.44	977.18	4443.96
5 (t-outer)	26.72	0.47	298.23	364.99	690.41
ONE METER FROM THE 100-TON HI-TRAC					
1	50.03	11.75	46.82	45.73	154.33
2	157.59	27.05	3.45	59.65	247.74
3	12.25	4.64	68.65	12.66	98.21
3 (temp)	12.23	4.85	55.61	6.08	78.78
4	0.76	0.16	39.85	29.80	70.57
5 (transfer lid)	38.01	0.47	1500.68	261.84	1801.00
5 (t-outer)	4.71	0.84	126.74	74.03	206.32

Notes:

- Refer to Figures 5.1.2 and 5.1.4 for dose locations.
- Dose location 3(temp) represents dose location 3 with temporary shielding installed.
- Dose location 4(outer) is the radial segment at dose location 4 which is 18-30 inches from the center of the overpack.
- Dose location 5(t-outer) is the radial segment at dose location 5 (transfer lid) which is 30-42 and 54-66 inches from the center of the lid for the adjacent and one meter locations, respectively. The inner radius of the HI-TRAC is 34.375 in. and the outer radius of the water jacket is 44.375 in.
- Dose rate based on no water within the MPC. For the majority of the duration that the HI-TRAC pool lid is installed, the MPC cavity will be flooded with water. The water within the MPC greatly reduces the dose rate.

Table 5.4.11

DOSE RATES FROM THE 100-TON HI-TRAC FOR NORMAL CONDITIONS
MPC-32 DESIGN BASIS ZIRCALOY CLAD FUEL
32,500 MWD/MTU AND 5-YEAR COOLING

Dose Point Location	Fuel Gammas (mrem/hr)	(n, γ) Gammas (mrem/hr)	⁶⁰ Co Gammas (mrem/hr)	Neutrons (mrem/hr)	Totals (mrem/hr)	Totals with BPRAs (mrem/hr)
ADJACENT TO THE 100-TON HI-TRAC						
1	34.19	6.26	713.29	91.01	844.75	854.30
2	908.57	27.74	1.02	52.05	989.38	1260.07
3	11.14	1.22	450.32	71.69	534.37	743.66
4	28.45	0.80	337.51	89.81	456.58	622.75
4 (outer)	7.59	0.32	84.67	61.11	153.68	195.46
5 (pool)	217.00	8.76	3935.75	592.50	4754.00	4830.44
5 (transfer)	403.36	0.34	5939.74	332.20	6675.64	6767.42
5(t-outer)	73.70	0.18	500.60	132.39	706.87	725.52
ONE METER FROM THE 100-TON HI-TRAC						
1	119.57	3.67	106.31	15.04	244.59	279.86
2	399.54	8.71	8.10	18.99	435.35	555.53
3	51.57	2.07	92.54	7.15	153.34	215.38
4	8.54	0.12	100.64	22.18	131.48	180.84
5 (transfer)	166.15	0.11	2361.13	92.74	2620.13	2660.63
5(t-outer)	17.09	0.35	209.36	27.00	253.81	257.96

Notes:

- Refer to Figures 5.1.2 and 5.1.4 for dose locations.
- Dose location 4(outer) is the radial segment at dose location 4 which is 18-30 inches from the center of the overpack.
- Dose location 5(t-outer) is the radial segment at dose location 5 (transfer lid) which is 30-42 and 54-66 inches from the center of the lid for the adjacent and one meter locations, respectively. The inner radius of the HI-TRAC is 34.375 in. and the outer radius of the water jacket is 44.375 in.
- Dose rate based on no water within the MPC. For the majority of the duration that the HI-TRAC pool lid is installed, the MPC cavity will be flooded with water. The water within the MPC greatly reduces the dose rate.

Table 5.4.12

DOSE RATES FROM THE 100-TON HI-TRAC FOR NORMAL CONDITIONS
MPC-32 DESIGN BASIS ZIRCALOY CLAD FUEL
45,000 MWD/MTU AND 10-YEAR COOLING

Dose Point Location	Fuel Gammas (mrem/hr)	(n, γ) Gammas (mrem/hr)	⁶⁰ Co Gammas (mrem/hr)	Neutrons (mrem/hr)	Totals (mrem/hr)	Totals with BPRAs (mrem/hr)
ADJACENT TO THE 100-TON HI-TRAC						
1	14.91	13.25	442.90	192.77	663.83	673.38
2	422.32	58.76	0.64	110.24	591.95	862.64
3	4.50	2.59	279.61	151.86	438.55	647.84
4	12.19	1.70	209.57	190.21	413.67	579.84
4 (outer)	3.10	0.67	52.57	129.44	185.79	227.56
5 (pool)	102.06	18.55	2443.80	1254.97	3819.38	3895.82
5 (transfer)	190.81	0.72	3688.12	703.71	4583.37	4675.15
5(t-outer)	34.38	0.39	310.83	280.41	626.01	644.67
ONE METER FROM THE 100-TON HI-TRAC						
1	54.75	7.78	66.01	31.84	160.38	195.65
2	185.04	18.46	5.03	40.22	248.74	368.92
3	23.55	4.38	57.46	15.15	100.54	162.59
4	3.39	0.24	62.49	46.97	113.10	162.46
5 (transfer)	77.12	0.23	1466.08	196.43	1739.86	1780.36
5(t-outer)	7.85	0.74	130.00	57.20	195.78	199.93

Notes:

- Refer to Figures 5.1.2 and 5.1.4 for dose locations.
- Dose location 4(outer) is the radial segment at dose location 4 which is 18-30 inches from the center of the overpack.
- Dose location 5(t-outer) is the radial segment at dose location 5 (transfer lid) which is 30-42 and 54-66 inches from the center of the lid for the adjacent and one meter locations, respectively. The inner radius of the HI-TRAC is 34.375 in. and the outer radius of the water jacket is 44.375 in.
- Dose rate based on no water within the MPC. For the majority of the duration that the HI-TRAC pool lid is installed, the MPC cavity will be flooded with water. The water within the MPC greatly reduces the dose rate.

Table 5.4.13

DOSE RATES FROM THE 100-TON HI-TRAC FOR ACCIDENT CONDITIONS
 WITH FOUR DAMAGED FUEL CONTAINERS
 MPC-24 DESIGN BASIS ZIRCALOY CLAD FUEL
 42,500 MWD/MTU AND 5-YEAR COOLING
 WITHOUT BPRAs

Dose Point [†] Location	Fuel Gammas (mrem/hr)	(n,γ) Gammas (mrem/hr)	⁶⁰ Co Gammas (mrem/hr)	Neutrons (mrem/hr)	Totals (mrem/hr)
ADJACENT TO THE 100-TON HI-TRAC					
1	48.07	15.17	627.06	230.25	920.55
2	996.02	54.95	0.75	100.04	1151.76
3	13.63	3.41	345.75	221.02	583.80
ONE METER FROM THE 100-TON HI-TRAC					
1	140.31	7.79	93.21	34.99	276.30
2	442.10	17.01	7.37	37.68	504.15
3	63.42	4.45	77.43	20.56	165.86

[†] Refer to Figures 5.1.2 and 5.1.4.

Table 5.4.14

DOSE RATES FROM THE 100-TON HI-TRAC FOR ACCIDENT CONDITIONS
 WITH SIXTEEN DAMAGED FUEL CONTAINERS
 MPC-68 DESIGN BASIS ZIRCALOY CLAD FUEL
 40,000 MWD/MTU AND 5-YEAR COOLING

Dose Point [†] Location	Fuel Gammas (mrem/hr)	(n,γ) Gammas (mrem/hr)	⁶⁰ Co Gammas (mrem/hr)	Neutrons (mrem/hr)	Totals (mrem/hr)
ADJACENT TO THE 100-TON HI-TRAC					
1	90.03	18.80	884.16	324.33	1317.31
2	845.02	65.12	0.56	110.16	1020.86
3	2.60	2.76	535.10	164.10	704.57
ONE METER FROM THE 100-TON HI-TRAC					
1	141.32	9.84	82.57	46.71	280.44
2	369.64	19.91	6.09	44.10	439.74
3	40.76	4.37	121.08	17.02	183.23

[†] Refer to Figures 5.1.2 and 5.1.4.

Table 5.4.15

DOSE RATES DUE TO BPRAs AND TPDs FROM THE 100-TON HI-TRAC
FOR NORMAL CONDITIONS

Dose Point Location	MPC-24		MPC-32	
	BPRAs (mrem/hr)	TPDs (mrem/hr)	BPRAs (mrem/hr)	TPDs (mrem/hr)
ADJACENT TO THE 100-TON HI-TRAC				
1	7.93	0.00	9.55	0.01
2	229.56	0.03	270.69	0.04
3	137.91	125.75	209.28	188.04
3 (temp)	63.27	56.21	86.91	76.97
4	115.60	106.71	166.17	156.15
4 (outer)	29.09	27.12	41.78	39.32
5 (pool lid)	60.14	0.00	76.44	0.00
5 (transfer lid)	89.91	0.00	91.78	0.00
5 (t-outer)	19.94	0.00	18.65	0.00
ONE METER FROM THE 100-TON HI-TRAC				
1	30.16	0.18	35.26	0.23
2	101.65	1.20	120.18	1.62
3	45.90	38.93	62.05	54.93
3 (temp)	41.38	35.01	54.88	48.77
4	35.81	33.37	49.36	47.19
5 (transfer lid)	36.33	0.00	40.50	0.00
5 (t-outer)	3.62	0.00	4.15	0.00

Notes:

- Refer to Figures 5.1.2 and 5.1.4 for dose locations.
- Dose location 3(temp) represents dose location 3 with temporary shielding installed.
- Dose location 4(outer) is the radial segment at dose location 4 which is 18-30 inches from the center of the overpack.
- Dose location 5(t-outer) is the radial segment at dose location 5 (transfer lid) which is 30-42 and 54-66 inches from the center of the lid for the adjacent and one meter locations, respectively. The inner radius of the HI-TRAC is 34.375 in. and the outer radius of the water jacket is 44.375 in.
- Dose rate based on no water within the MPC. For the majority of the duration that the HI-TRAC pool lid is installed, the MPC cavity will be flooded with water. The water within the MPC greatly reduces the dose rate.

Table 5.4.16

DOSE RATES DUE TO CRAs FROM THE 100-TON HI-TRAC
FOR NORMAL CONDITIONS

Dose Point Location	MPC-24		MPC-32	
	Config. 1 (mrem/hr)	Config. 2 (mrem/hr)	Config. 1 (mrem/hr)	Config. 2 (mrem/hr)
ADJACENT TO THE 100-TON HI-TRAC				
1	5.39	1.02	3.28	0.68
2	0.09	0.00	0.01	0.00
3	0.00	0.00	0.00	0.00
4	0.00	0.00	0.00	0.00
5 (pool lid)	919.59	170.85	1141.10	213.24
5 (transfer lid)	1519.98	287.72	2012.93	380.57
5 (t-outer)	1.54	0.25	1.01	0.19
ONE METER FROM THE 100-TON HI-TRAC				
1	1.20	0.20	0.69	0.14
2	0.26	0.03	0.05	0.01
3	0.01	0.00	0.00	0.00
4	0.00	0.00	0.00	0.00
5 (transfer lid)	223.62	41.60	257.95	49.19
5 (t-outer)	8.26	1.54	8.87	1.70

Notes:

- Refer to Figures 5.1.2 and 5.1.4 for dose locations.
- Dose location 5 (t-outer) is the radial segment at dose location 5 (transfer lid) which is 30-42 and 54-66 inches from the center of the lid for the adjacent and one meter locations, respectively. The inner radius of the HI-TRAC is 34.375 in. and the outer radius of the water jacket is 44.375 in.
- Dose rate based on no water within the MPC. For the majority of the duration that the HI-TRAC pool lid is installed, the MPC cavity will be flooded with water. The water within the MPC greatly reduces the dose rate.

Table 5.4.17

DOSE RATES DUE TO APSRs FROM THE 100-TON HI-TRAC
FOR NORMAL CONDITIONS

Dose Point Location	MPC-24			MPC-32		
	Config. 1 (mrem/hr)	Config. 2 (mrem/hr)	Config. 3 (mrem/hr)	Config. 1 (mrem/hr)	Config. 2 (mrem/hr)	Config. 3 (mrem/hr)
ADJACENT TO THE 100-TON HI-TRAC						
1	12.42	2.35	12.25	7.57	1.56	7.51
2	0.21	0.01	9.12	0.03	0.00	0.19
3	0.00	0.00	0.00	0.00	0.00	0.00
4	0.00	0.00	0.00	0.00	0.00	0.00
5 (pool lid)	1996.57	371.98	1941.51	2414.84	453.88	2687.17
5 (transfer)	3021.08	572.85	2994.54	3980.02	750.17	3860.83
5 (t-outer)	3.41	0.54	3.57	2.23	0.42	1.94
ONE METER FROM THE 100-TON HI-TRAC						
1	2.73	0.46	3.49	1.57	0.32	1.58
2	0.61	0.07	3.31	0.12	0.02	0.18
3	0.02	0.00	0.04	0.01	0.00	0.01
4	0.00	0.00	0.00	0.00	0.00	0.00
5 (transfer)	458.06	84.81	444.44	521.02	99.10	510.78
5 (t-outer)	17.11	3.19	17.36	18.34	3.48	18.20

Notes:

- Refer to Figures 5.1.2 and 5.1.4 for dose locations.
- Dose location 5 (t-outer) is the radial segment at dose location 5 (transfer lid) which is 30-42 and 54-66 inches from the center of the lid for the adjacent and one meter locations, respectively. The inner radius of the HI-TRAC is 34.375 in. and the outer radius of the water jacket is 44.375 in.
- Dose rate based on no water within the MPC. For the majority of the duration that the HI-TRAC pool lid is installed, the MPC cavity will be flooded with water. The water within the MPC greatly reduces the dose rate.

Table 5.4.18

COMPARISON OF NEUTRON SOURCE PER INCH PER SECOND FOR
DESIGN BASIS 7X7 FUEL AND DESIGN BASIS DRESDEN UNIT 1 FUEL

Assembly	Active fuel length (inch)	Neutrons per sec per inch	Neutrons per sec per inch with Sb-Be source	Reference for neutrons per sec per inch
7x7 design basis	144	9.17E+5	N/A	Table 5.2.17 - 40 GWD/MTU and 5 year cooling
6x6 design basis	110	2.0E+5	2.6E+5	Table 5.2.18
6x6 design basis MOX	110	3.06E+5	3.66E+5	Table 5.2.23

5.5 REGULATORY COMPLIANCE

Chapters 1 and 2 and this chapter of this FSAR describe in detail the shielding structures, systems, and components (SSCs) important to safety.

This chapter has evaluated these shielding SSCs important to safety and has assessed the impact on health and safety resulting from operation of an independent spent fuel storage installation (ISFSI) utilizing the HI-STORM 100 System.

It has been shown that the design of the shielding system of the HI-STORM 100 System is in compliance with 10CFR72 and that the applicable design and acceptance criteria including 10CFR20 have been satisfied. Thus, this shielding evaluation provides reasonable assurance that the HI-STORM 100 System will allow safe storage of spent fuel.

5.6 REFERENCES

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- [5.2.3] A. Luksic, "Spent Fuel Assembly Hardware: Characterization and 10CFR 61 Classification for Waste Disposal," PNL-6906-vol. 1, Pacific Northwest Laboratory, June 1989.
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- [5.4.5] S. E. Turner, "Uncertainty Analysis - Axial Burnup Distribution Effects," presented in "Proceedings of a Workshop on the Use of Burnup Credit in Spent Fuel Transport Casks," SAND-89-0018, Sandia National Laboratory, Oct. 1989.

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APPENDIX 5.A

SAMPLE INPUT FILE FOR SAS2H

(Total number of pages in this appendix : 3)

```

=SAS2H      PARM='halt05,skipshipdata'
bw 15x15 PWR assembly
' fuel temp 923
44groupndf5      LATTICECELL
UO2 1 0.95 923 92234 0.03204 92235 3.6 92236 0.01656
    92238 96.3514 END
'
' Zirc 4 composition
ARBM-ZIRC4 6.55 4 1 0 0 50000 1.7 26000 0.24 24000 0.13 40000 97.93
    2 1.0 595 END
'
' water with 652.5 ppm boron
H2O      3 DEN=0.7135 1 579 END
ARBM-BORMOD 0.7135 1 1 0 0 5000 100 3 652.5E-6 579 END
'
co-59 3 0 1-20 579 end
kr-83 1 0 1-20 923 end
kr-84 1 0 1-20 923 end
kr-85 1 0 1-20 923 end
kr-86 1 0 1-20 923 end
sr-90 1 0 1-20 923 end
y-89 1 0 1-20 923 end
zr-94 1 0 1-20 923 end
zr-95 1 0 1-20 923 end
mo-94 1 0 1-20 923 end
mo-95 1 0 1-20 923 end
nb-94 1 0 1-20 923 end
nb-95 1 0 1-20 923 end
tc-99 1 0 1-20 923 end
ru-106 1 0 1-20 923 end
rh-103 1 0 1-20 923 end
rh-105 1 0 1-20 923 end
sb-124 1 0 1-20 923 end
sn-126 1 0 1-20 923 end
xe-131 1 0 1-20 923 end
xe-132 1 0 1-20 923 end
xe-134 1 0 1-20 923 end
'
xe-135 1 0 1-09 923 end
'
xe-136 1 0 1-20 923 end
cs-133 1 0 1-20 923 end
cs-134 1 0 1-20 923 end
cs-135 1 0 1-20 923 end
cs-137 1 0 1-20 923 end
ba-136 1 0 1-20 923 end
la-139 1 0 1-20 923 end
ce-144 1 0 1-20 923 end
pr-143 1 0 1-20 923 end
nd-143 1 0 1-20 923 end
nd-144 1 0 1-20 923 end
nd-145 1 0 1-20 923 end
nd-146 1 0 1-20 923 end
nd-147 1 0 1-20 923 end
nd-148 1 0 1-20 923 end
nd-150 1 0 1-20 923 end
pm-147 1 0 1-20 923 end
pm-148 1 0 1-20 923 end
pm-149 1 0 1-20 923 end

```



```

sm-147  1 0 1-20 923 end
sm-148  1 0 1-20 923 end
sm-149  1 0 1-20 923 end
sm-150  1 0 1-20 923 end
sm-151  1 0 1-20 923 end
sm-152  1 0 1-20 923 end
eu-151  1 0 1-20 923 end
eu-153  1 0 1-20 923 end
eu-154  1 0 1-20 923 end
eu-155  1 0 1-20 923 end
gd-154  1 0 1-20 923 end
gd-155  1 0 1-20 923 end
gd-157  1 0 1-20 923 end
gd-158  1 0 1-20 923 end
gd-160  1 0 1-20 923 end

```

```

END COMP

```

```

-----

```

```

' FUEL-PIN-CELL GEOMETRY:

```

```

SQUAREPITCH  1.44272 0.950468 1 3 1.08712 2 0.97028 0 END

```

```

-----

```

```

' MTU in this model is 0.495485 based on fuel dimensions provided
'
' 1 power cycle will be used and a library will be generated every
' 2500 MWD/MTU power level is 40 MW/MTU
' therefore 62.5 days per 2500 MWD/MTU
' Below
' BURN=62.5*NLIB/CYC
' POWER=MTU*40

```

```

' Number of libraries is 20 which is 50,000 MWD/MTU burnup (20*2500)

```

```

' ASSEMBLY AND CYCLE PARAMETERS:

```

```

NPIN/ASSM=208 FUELNGTH=365.76 NCYCLES=1 NLIB/CYC=20
PRINTLEVEL=1
LIGHTTEL=5 INPLEVEL=1 NUMHOLES=17
NUMINStr= 0 ORTUBE= 0.6731 SRTUBE=0.63246 END
POWER=19.81938 BURN=1250.0 END

```

```

O 66.54421
FE 0.24240868
ZR 98.78151 CR 0.1311304 SN 1.714782

```

```

END

```

```

=SAS2H PARM='restarts, halt10, skipshipdata'
bw 15x15 PWR assembly

```

```

END

```

```

=SAS2H PARM='restarts, halt15, skipshipdata'
bw 15x15 PWR assembly

```

```

END

```

```

=SAS2H PARM='restarts, halt20, skipshipdata'
bw 15x15 PWR assembly

```

```

END

```

APPENDIX 5.B

SAMPLE INPUT FILE FOR ORIGEN-S

(Total number of pages in this appendix : 7)

```

#ORIGENS
0$$ A4 33 A8 26 A11 71 E
1$$ 1 T
bw 15x15 FUEL -- FT33F001 -
'
' SUBCASE 1 LIBRARY POSITION 1
'
' lib pos grms photon group
3$$ 33 A3 1 0 A16 2 E T
35$$ 0 T
56$$ 5 5 A6 3 A10 0 A13 9 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
FUEL 3.6
BW 15x15 0.495485 MTU
58** 19.81938 19.81938 19.81938 19.81938 19.81938
60** 1.0000 3.0000 15.0000 30.0000 62.5
66$$ A1 2 A5 2 A9 2 E
73$$ 922350 922340 922360 922380 80000 500000
260000 240000 400000
74** 17837.45 158.7533 82.05225 477406.4 66544.21 1714.782
242.0868 131.1304 98781.51
75$$ 2 2 2 2 4 4 4 4 4 T
'
' SUBCASE 2 LIBRARY POSITION 2
'
3$$ 33 A3 2 0 A16 2 A33 0 E T
35$$ 0 T
56$$ 3 3 A6 3 A10 5 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
fuel
BW 15X15
58** 19.81938 19.81938 19.81938
60** 18.5 37.0 62.5
66$$ A1 2 A5 2 A9 2 E T
'
' SUBCASE 3 LIBRARY POSITION 3
'
3$$ 33 A3 3 0 A16 2 A33 0 E T
35$$ 0 T
56$$ 3 3 A6 3 A10 3 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
fuel
BW 15X15
58** 19.81938 19.81938 19.81938
60** 18.5 37.0 62.5
66$$ A1 2 A5 2 A9 2 E T
'
' SUBCASE 4 LIBRARY POSITION 4
'
3$$ 33 A3 4 0 A16 2 A33 0 E T
35$$ 0 T
56$$ 3 3 A6 3 A10 3 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
fuel
BW 15X15
58** 19.81938 19.81938 19.81938
60** 18.5 37.0 62.5
66$$ A1 2 A5 2 A9 2 E T
'

```

' SUBCASE 5 LIBRARY POSITION 5
,
3\$\$ 33 A3 5 0 A16 2 A33 0 E T
35\$\$ 0 T
56\$\$ 3 3 A6 3 A10 3 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
fuel
BW 15X15
58** 19.81938 19.81938 19.81938
60** 18.5 37.0 62.5
66\$\$ A1 2 A5 2 A9 2 E T
,

' SUBCASE 6 LIBRARY POSITION 6
,
3\$\$ 33 A3 6 0 A16 2 A33 0 E T
35\$\$ 0 T
56\$\$ 3 3 A6 3 A10 3 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
fuel
BW 15X15
58** 19.81938 19.81938 19.81938
60** 18.5 37.0 62.5
66\$\$ A1 2 A5 2 A9 2 E T
,

' SUBCASE 7 LIBRARY POSITION 7
,
3\$\$ 33 A3 7 0 A16 2 A33 0 E T
35\$\$ 0 T
56\$\$ 3 3 A6 3 A10 3 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
fuel
BW 15X15
58** 19.81938 19.81938 19.81938
60** 18.5 37.0 62.5
66\$\$ A1 2 A5 2 A9 2 E T
,

' SUBCASE 8 LIBRARY POSITION 8
,
3\$\$ 33 A3 8 0 A16 2 A33 0 E T
35\$\$ 0 T
56\$\$ 3 3 A6 3 A10 3 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
fuel
BW 15X15
58** 19.81938 19.81938 19.81938
60** 18.5 37.0 62.5
66\$\$ A1 2 A5 2 A9 2 E T
,

' SUBCASE 9 LIBRARY POSITION 9
,
3\$\$ 33 A3 9 0 A16 2 A33 0 E T
35\$\$ 0 T
56\$\$ 3 3 A6 3 A10 3 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
fuel
BW 15X15
58** 19.81938 19.81938 19.81938
60** 18.5 37.0 62.5
66\$\$ A1 2 A5 2 A9 2 E T

'
' SUBCASE 10 LIBRARY POSITION 10
'
3\$\$ 33 A3 10 0 A16 2 A33 0 E T
35\$\$ 0 T
56\$\$ 3 3 A6 3 A10 3 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
fuel
BW 15X15
58** 19.81938 19.81938 19.81938
60** 18.5 37.0 62.5
66\$\$ A1 2 A5 2 A9 2 E T

' SUBCASE 11 LIBRARY POSITION 11
'
3\$\$ 33 A3 11 0 A16 2 A33 0 E T
35\$\$ 0 T
56\$\$ 3 3 A6 3 A10 3 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
fuel
BW 15X15
58** 19.81938 19.81938 19.81938
60** 18.5 37.0 62.5
66\$\$ A1 2 A5 2 A9 2 E T

' SUBCASE 12 LIBRARY POSITION 12
'
3\$\$ 33 A3 12 0 A16 2 A33 0 E T
35\$\$ 0 T
56\$\$ 3 3 A6 3 A10 3 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
fuel
BW 15X15
58** 19.81938 19.81938 19.81938
60** 18.5 37.0 62.5
66\$\$ A1 2 A5 2 A9 2 E T

' SUBCASE 13 LIBRARY POSITION 13
'
3\$\$ 33 A3 13 0 A16 2 A33 0 E T
35\$\$ 0 T
56\$\$ 3 3 A6 3 A10 3 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
fuel
BW 15X15
58** 19.81938 19.81938 19.81938
60** 18.5 37.0 62.5
66\$\$ A1 2 A5 2 A9 2 E T

' SUBCASE 14 LIBRARY POSITION 14
'
3\$\$ 33 A3 14 0 A16 2 A33 0 E T
35\$\$ 0 T
56\$\$ 3 3 A6 3 A10 3 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
fuel
BW 15X15
58** 19.81938 19.81938 19.81938
60** 18.5 37.0 62.5

66\$\$ A1 2 A5 2 A9 2 E T
'
' SUBCASE 15 LIBRARY POSITION 15
'
3\$\$ 33 A3 15 0 A16 2 A33 0 E T
35\$\$ 0 T
56\$\$ 3 3 A6 3 A10 3 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
fuel
BW 15X15
58** 19.81938 19.81938 19.81938
60** 18.5 37.0 62.5
66\$\$ A1 2 A5 2 A9 2 E T

' SUBCASE 16 LIBRARY POSITION 16
'
3\$\$ 33 A3 16 0 A16 2 A33 0 E T
35\$\$ 0 T
56\$\$ 3 3 A6 3 A10 3 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
fuel
BW 15X15
58** 19.81938 19.81938 19.81938
60** 18.5 37.0 62.5
66\$\$ A1 2 A5 2 A9 2 E T

' SUBCASE 17 LIBRARY POSITION 17
'
3\$\$ 33 A3 17 0 A16 2 A33 0 E T
35\$\$ 0 T
56\$\$ 3 3 A6 3 A10 3 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
fuel
BW 15X15
58** 19.81938 19.81938 19.81938
60** 18.5 37.0 62.5
66\$\$ A1 2 A5 2 A9 2 E T

' SUBCASE 18 LIBRARY POSITION 18
'
3\$\$ 33 A3 18 A4 7 0 A16 2 A33 18 E T
35\$\$ 0 T
56\$\$ 3 3 A6 1 A10 3 A15 3 A19 1 E
57** 0.0 A3 1.E-5 0.05556 E T
fuel
BW 15X15
58** 19.81938 19.81938 19.81938
60** 18.5 37.0 62.5
66\$\$ A1 2 A5 2 A9 2 E T

' SUBCASE - decay
'
54\$\$ A8 1 E
56\$\$ 0 9 A6 1 A10 3 A14 3 A15 1 A19 1 E
57** 0.0 0 1.E-5 E T
fuel enrichment above
60** 0.5 0.75 1.0 4.0 8.0 12.0 24.0 48.0 96.0
61** F0.1
65\$\$

'GRAM-ATOMS	GRAMS	CURIES	WATTS-ALL	WATTS-GAMMA	
3Z	0 1 0	0 0 0	1 0 0	3Z	6Z
3Z	0 1 0	0 0 0	1 0 0	3Z	6Z
3Z	0 1 0	0 0 0	1 0 0	3Z	6Z T

' SUBCASE - decay

54\$\$ A8 1 E
 56\$\$ 0 9 A6 1 A10 9 A14 4 A15 1 A19 1 E
 57** 4.0 0 1.E-5 E T
 fuel enrichment above
 60** 10.0 20.0 30.0 60.0 90.0 120.0 180.0 240.0 365.0
 61** F0.1

'GRAM-ATOMS	GRAMS	CURIES	WATTS-ALL	WATTS-GAMMA	
3Z	0 1 0	0 0 0	1 0 0	3Z	6Z
3Z	0 1 0	0 0 0	1 0 0	3Z	6Z
3Z	0 1 0	0 0 0	1 0 0	3Z	6Z T

' SUBCASE - decay

54\$\$ A8 0 E
 56\$\$ 0 9 A6 1 A10 9 A14 5 A15 1 A19 1 E
 57** 1.0 0 1.E-5 E T
 fuel enrichment above
 60** 1.5 3.0 4.0 5.0 6.0 7.0 8.0 9.0 10.0
 61** F1.0e-5

'GRAM-ATOMS	GRAMS	CURIES	WATTS-ALL	WATTS-GAMMA	
3Z	0 1 0	1 0 0	1 0 0	3Z	6Z
3Z	0 1 0	1 0 0	1 0 0	3Z	6Z
3Z	0 1 0	1 0 0	1 0 0	3Z	6Z

81\$\$ 2 0 26 1 E
 82\$\$ 0 2 2 2 2 2 2 2
 83** 1.1E+7 8.0E+6 6.0E+6 4.0E+6 3.0E+6 2.5E+6 2.0E+6 1.5E+6
 1.0E+6 7.0E+5 4.5E+5 3.0E+5 1.5E+5 1.0E+5 7.0E+4 4.5E+4
 3.0E+4 2.0E+4 1.0E+4
 84** 20.0E+6 6.43E+6 3.0E+6 1.85E+6 1.40E+6 9.00E+5 4.00E+5 1.0E+5 T

' SUBCASE - decay

54\$\$ A8 0 E
 56\$\$ 0 10 A6 1 A10 9 A14 5 A15 1 A19 1 E
 57** 10.0 0 1.E-5 E T
 fuel enrichment above
 60** 11.0 12.0 13.0 14.0 15.0 16.0 17.0 18.0 19.0 20.0
 61** F1.0e-5

'GRAM-ATOMS	GRAMS	CURIES	WATTS-ALL	WATTS-GAMMA	
3Z	0 1 0	1 0 0	1 0 0	3Z	6Z
3Z	0 1 0	1 0 0	1 0 0	3Z	6Z
3Z	0 1 0	1 0 0	1 0 0	3Z	6Z

APPENDIX 5.C

SAMPLE INPUT FILE FOR MCNP

(Total number of pages in this appendix : 61)

message: outp=hs24c1l0 srctp=hs24c1l1s runtpe=hs24c1l1r
mctal=hs24c1l1m wssa=hs24c1l1w rssa=pt001w

hs24c11

c
c origin is 6 inches below mpc
c
c only cells that contain material are split axially
c importance splitting is not done in cells with 0 material
c
c axial segmentation is at the following boundaries
c 615, 620, 420, 430, 445, 455, 675, 651, 652 ,653
c 654, 655, 656, 657, 680
c
c universe 1

301	0			(-40:41:-42:43)	-400	u=1	
302	0	37 -38		-12	400 -410	u=1	
303	0	37 -38	15		400 -410	u=1	
304	0	35 -36			-20 400 -410	u=1	
305	0	35 -36		23	400 -410	u=1	
306	0	37 -38		-12	435 -460	u=1	
307	0	37 -38	15		435 -460	u=1	
308	0	35 -36			-20 435 -460	u=1	
309	0	35 -36		23	435 -460	u=1	
310	0	37 -38		-10	410 -435	u=1	
311	0	37 -38	17		410 -435	u=1	
312	0	35 -36			-18 410 -435	u=1	
313	0	35 -36		25	410 -435	u=1	
314	5	-7.92	10 -11 26	-27 410 -420	u=-1	\$ left	
315	6	-2.644	11 -12 26	-27 410 -420	u=-1	\$ left	
316	6	-2.644	15 -16 26	-27 410 -420	u=-1	\$ right	
317	5	-7.92	16 -17 26	-27 410 -420	u=-1	\$ right	
318	5	-7.92	28 -29 18 -19	410 -420	u=-1	\$ bot	
319	6	-2.644	28 -29 19 -20	410 -420	u=-1	\$ bot	
320	6	-2.644	28 -29 23 -24	410 -420	u=-1	\$ top	
321	5	-7.92	28 -29 24 -25	410 -420	u=-1	\$ top	
322	5	-7.92	10 -11 26	-27 420 -430	u=-1	\$ left	
323	6	-2.644	11 -12 26	-27 420 -430	u=-1	\$ left	
324	6	-2.644	15 -16 26	-27 420 -430	u=-1	\$ right	
325	5	-7.92	16 -17 26	-27 420 -430	u=-1	\$ right	
326	5	-7.92	28 -29 18 -19	420 -430	u=-1	\$ bot	
327	6	-2.644	28 -29 19 -20	420 -430	u=-1	\$ bot	
328	6	-2.644	28 -29 23 -24	420 -430	u=-1	\$ top	
329	5	-7.92	28 -29 24 -25	420 -430	u=-1	\$ top	
330	5	-7.92	10 -11 26	-27 430 -435	u=-1	\$ left	
331	6	-2.644	11 -12 26	-27 430 -435	u=-1	\$ left	
332	6	-2.644	15 -16 26	-27 430 -435	u=-1	\$ right	
333	5	-7.92	16 -17 26	-27 430 -435	u=-1	\$ right	
334	5	-7.92	28 -29 18 -19	430 -435	u=-1	\$ bot	
335	6	-2.644	28 -29 19 -20	430 -435	u=-1	\$ bot	
336	6	-2.644	28 -29 23 -24	430 -435	u=-1	\$ top	
337	5	-7.92	28 -29 24 -25	430 -435	u=-1	\$ top	
338	0		10 -12 27 -38	410 -435	u=-1	\$ left	
339	0		10 -12 37 -26	410 -435	u=-1	\$ left	
340	0		15 -17 27 -38	410 -435	u=-1	\$ right	
341	0		15 -17 37 -26	410 -435	u=-1	\$ right	
342	0		35 -28 18 -20	410 -435	u=-1	\$ bot	
343	0		29 -36 18 -20	410 -435	u=-1	\$ bot	
344	0		35 -28 23 -25	410 -435	u=-1	\$ top	
345	0		29 -36 23 -25	410 -435	u=-1	\$ top	
346	5	-7.92	12 -15 20 -23	(-13:-21:22:14)	400 -420	u=-1	
347	5	-7.92	12 -15 20 -23	(-13:-21:22:14)	420 -430	u=-1	
348	5	-7.92	12 -15 20 -23	(-13:-21:22:14)	430 -445	u=-1	
349	5	-7.92	12 -15 20 -23	(-13:-21:22:14)	445 -460	u=-1	

```

350 0          13 -14 21 -22 (-40:41:-42:43) 400 -455 u=-1
351 9 -1.17e-3          460          u=1
c fuel element
352 0          40 -41 42 -43          -415 u=1
353 5 -1.0783 40 -41 42 -43 415 -420 u=-1 $ lower nozzle
354 0          40 -41 42 -43 420 -425 u=-1 $ space
355 2 -3.8699 40 -41 42 -43 425 -430 u=-1 $ active fuel
356 5 -0.1591 40 -41 42 -43 430 -440 u=-1 $ space
357 5 -0.1591 40 -41 42 -43 440 -445 u=-1 $ plenum spacer
358 5 -1.5410 40 -41 42 -43 445 -455 u=-1 $ top nozzle
359 0          13 -14 21 -22 455 -460 u=-1
c
360 5 -7.92    38 -23 -12 400 -420 u=1
361 5 -7.92    20 -37 -12 400 -420 u=1
362 5 -7.92    12 -35 23 400 -420 u=1
363 5 -7.92    12 -35 -20 400 -420 u=1
364 5 -7.92    36 -15 23 400 -420 u=1
365 5 -7.92    36 -15 -20 400 -420 u=1
366 5 -7.92    38 -23 15 400 -420 u=1
367 5 -7.92    20 -37 15 400 -420 u=1
368 5 -7.92    38 -23 -12 420 -430 u=1
369 5 -7.92    20 -37 -12 420 -430 u=1
370 5 -7.92    12 -35 23 420 -430 u=1
371 5 -7.92    12 -35 -20 420 -430 u=1
372 5 -7.92    36 -15 23 420 -430 u=1
373 5 -7.92    36 -15 -20 420 -430 u=1
374 5 -7.92    38 -23 15 420 -430 u=1
375 5 -7.92    20 -37 15 420 -430 u=1
376 5 -7.92    38 -23 -12 430 -445 u=1
377 5 -7.92    20 -37 -12 430 -445 u=1
378 5 -7.92    12 -35 23 430 -445 u=1
379 5 -7.92    12 -35 -20 430 -445 u=1
380 5 -7.92    36 -15 23 430 -445 u=1
381 5 -7.92    36 -15 -20 430 -445 u=1
382 5 -7.92    38 -23 15 430 -445 u=1
383 5 -7.92    20 -37 15 430 -445 u=1
384 5 -7.92    38 -23 -12 445 -460 u=1
385 5 -7.92    20 -37 -12 445 -460 u=1
386 5 -7.92    12 -35 23 445 -460 u=1
387 5 -7.92    12 -35 -20 445 -460 u=1
388 5 -7.92    36 -15 23 445 -460 u=1
389 5 -7.92    36 -15 -20 445 -460 u=1
390 5 -7.92    38 -23 15 445 -460 u=1
391 5 -7.92    20 -37 15 445 -460 u=1
392 0          23 -12          400 -460 u=1
393 0          23 15          400 -460 u=1
394 0          15 -20          400 -460 u=1
395 0          -12 -20          400 -460 u=1
c
c universe 2
c
401 0          (-40:41:-42:43) -400 u=2
402 0 37 -38          -12          400 -410 u=2
403 0 37 -38 15          400 -410 u=2
404 0 35 -36          -20 400 -410 u=2
405 0 35 -36          23          400 -410 u=2
406 0 37 -38          -12          435 -460 u=2
407 0 37 -38 15          435 -460 u=2
408 0 35 -36          -20 435 -460 u=2
409 0 35 -36          23          435 -460 u=2
410 0 37 -38          -10          410 -435 u=2
411 0 37 -38 17          410 -435 u=2
412 0 35 -36          -18 410 -435 u=2
413 0 35 -36          25          410 -435 u=2
414 5 -7.92    10 -11 26 -27 410 -420 u=-2 $ left

```

415	6	-2.644	11	-12	26	-27	410	-420	u=-2	\$ left
416	6	-2.644	15	-16	30	-31	410	-420	u=-2	\$ right
417	5	-7.92	16	-17	30	-31	410	-420	u=-2	\$ right
418	5	-7.92	28	-29	18	-19	410	-420	u=-2	\$ bot
419	6	-2.644	28	-29	19	-20	410	-420	u=-2	\$ bot
420	6	-2.644	32	-33	23	-24	410	-420	u=-2	\$ top
421	5	-7.92	32	-33	24	-25	410	-420	u=-2	\$ top
422	5	-7.92	10	-11	26	-27	420	-430	u=-2	\$ left
423	6	-2.644	11	-12	26	-27	420	-430	u=-2	\$ left
424	6	-2.644	15	-16	30	-31	420	-430	u=-2	\$ right
425	5	-7.92	16	-17	30	-31	420	-430	u=-2	\$ right
426	5	-7.92	28	-29	18	-19	420	-430	u=-2	\$ bot
427	6	-2.644	28	-29	19	-20	420	-430	u=-2	\$ bot
428	6	-2.644	32	-33	23	-24	420	-430	u=-2	\$ top
429	5	-7.92	32	-33	24	-25	420	-430	u=-2	\$ top
430	5	-7.92	10	-11	26	-27	430	-435	u=-2	\$ left
431	6	-2.644	11	-12	26	-27	430	-435	u=-2	\$ left
432	6	-2.644	15	-16	30	-31	430	-435	u=-2	\$ right
433	5	-7.92	16	-17	30	-31	430	-435	u=-2	\$ right
434	5	-7.92	28	-29	18	-19	430	-435	u=-2	\$ bot
435	6	-2.644	28	-29	19	-20	430	-435	u=-2	\$ bot
436	6	-2.644	32	-33	23	-24	430	-435	u=-2	\$ top
437	5	-7.92	32	-33	24	-25	430	-435	u=-2	\$ top
438	0		10	-12	27	-38	410	-435	u=-2	\$ left
439	0		10	-12	37	-26	410	-435	u=-2	\$ left
440	0		15	-17	31	-38	410	-435	u=-2	\$ right
441	0		15	-17	37	-30	410	-435	u=-2	\$ right
442	0		35	-28	18	-20	410	-435	u=-2	\$ bot
443	0		29	-36	18	-20	410	-435	u=-2	\$ bot
444	0		35	-32	23	-25	410	-435	u=-2	\$ top
445	0		33	-36	23	-25	410	-435	u=-2	\$ top
446	5	-7.92	12	-15	20	-23	(-13:-21:22:14)	400	-420	u=-2
447	5	-7.92	12	-15	20	-23	(-13:-21:22:14)	420	-430	u=-2
448	5	-7.92	12	-15	20	-23	(-13:-21:22:14)	430	-445	u=-2
449	5	-7.92	12	-15	20	-23	(-13:-21:22:14)	445	-460	u=-2
450	0		13	-14	21	-22	(-40:41:-42:43)	400	-455	u=-2
451	9	-1.17e-3					460		u=2	
c										fuel element
452	0		40	-41	42	-43		-415	u=2	
453	5	-1.0783	40	-41	42	-43	415	-420	u=-2	\$ lower nozzle
454	0		40	-41	42	-43	420	-425	u=-2	\$ space
455	2	-3.8699	40	-41	42	-43	425	-430	u=-2	\$ active fuel
456	5	-0.1591	40	-41	42	-43	430	-440	u=-2	\$ space
457	5	-0.1591	40	-41	42	-43	440	-445	u=-2	\$ plenum spacer
458	5	-1.5410	40	-41	42	-43	445	-455	u=-2	\$ top nozzle
459	0		13	-14	21	-22	455	-460	u=-2	
c										
460	5	-7.92	38	-23	-12		400	-420	u=2	
461	5	-7.92	20	-37	-12		400	-420	u=2	
462	0		12	-35	23		400	-420	u=2	
463	5	-7.92	12	-35	-20		400	-420	u=2	
464	0		36	-15	23		400	-420	u=2	
465	5	-7.92	36	-15	-20		400	-420	u=2	
466	0		38	-23	15		400	-420	u=2	
467	0		20	-37	15		400	-420	u=2	
468	5	-7.92	38	-23	-12		420	-430	u=2	
469	5	-7.92	20	-37	-12		420	-430	u=2	
470	0		12	-35	23		420	-430	u=2	
471	5	-7.92	12	-35	-20		420	-430	u=2	
472	0		36	-15	23		420	-430	u=2	
473	5	-7.92	36	-15	-20		420	-430	u=2	
474	0		38	-23	15		420	-430	u=2	
475	0		20	-37	15		420	-430	u=2	
476	5	-7.92	38	-23	-12		430	-445	u=2	
477	5	-7.92	20	-37	-12		430	-445	u=2	

478	0		12	-35	23	430	-445	u=2	
479	5	-7.92	12	-35	-20	430	-445	u=2	
480	0		36	-15	23	430	-445	u=2	
481	5	-7.92	36	-15	-20	430	-445	u=2	
482	0		38	-23	15	430	-445	u=2	
483	0		20	-37	15	430	-445	u=2	
484	5	-7.92	38	-23	-12	445	-460	u=2	
485	5	-7.92	20	-37	-12	445	-460	u=2	
486	0		12	-35	23	445	-460	u=2	
487	5	-7.92	12	-35	-20	445	-460	u=2	
488	0		36	-15	23	445	-460	u=2	
489	5	-7.92	36	-15	-20	445	-460	u=2	
490	0		38	-23	15	445	-460	u=2	
491	0		20	-37	15	445	-460	u=2	
492	0		23	-12		400	-460	u=2	
493	0		23	15		400	-460	u=2	
494	0		15	-20		400	-460	u=2	
495	0		-12	-20		400	-460	u=2	
c									
c	universe 3								
c									
501	0					(-40:41:-42:43)	-400	u=3	
502	0	37	-38		-12		400	-410	u=3
503	0	37	-38	15			400	-410	u=3
504	0	35	-36			-20	400	-410	u=3
505	0	35	-36		23		400	-410	u=3
506	0	37	-38		-12		435	-460	u=3
507	0	37	-38	15			435	-460	u=3
508	0	35	-36			-20	435	-460	u=3
509	0	35	-36		23		435	-460	u=3
510	0	37	-38		-10		410	-435	u=3
511	0	37	-38	17			410	-435	u=3
512	0	35	-36			-18	410	-435	u=3
513	0	35	-36		25		410	-435	u=3
514	5	-7.92		10	-11	30	-31	410	-420 u=-3 \$ left
515	6	-2.644		11	-12	30	-31	410	-420 u=-3 \$ left
516	6	-2.644		15	-16	26	-27	410	-420 u=-3 \$ right
517	5	-7.92		16	-17	26	-27	410	-420 u=-3 \$ right
518	5	-7.92		28	-29	18	-19	410	-420 u=-3 \$ bot
519	6	-2.644		28	-29	19	-20	410	-420 u=-3 \$ bot
520	6	-2.644		32	-33	23	-24	410	-420 u=-3 \$ top
521	5	-7.92		32	-33	24	-25	410	-420 u=-3 \$ top
522	5	-7.92		10	-11	30	-31	420	-430 u=-3 \$ left
523	6	-2.644		11	-12	30	-31	420	-430 u=-3 \$ left
524	6	-2.644		15	-16	26	-27	420	-430 u=-3 \$ right
525	5	-7.92		16	-17	26	-27	420	-430 u=-3 \$ right
526	5	-7.92		28	-29	18	-19	420	-430 u=-3 \$ bot
527	6	-2.644		28	-29	19	-20	420	-430 u=-3 \$ bot
528	6	-2.644		32	-33	23	-24	420	-430 u=-3 \$ top
529	5	-7.92		32	-33	24	-25	420	-430 u=-3 \$ top
530	5	-7.92		10	-11	30	-31	430	-435 u=-3 \$ left
531	6	-2.644		11	-12	30	-31	430	-435 u=-3 \$ left
532	6	-2.644		15	-16	26	-27	430	-435 u=-3 \$ right
533	5	-7.92		16	-17	26	-27	430	-435 u=-3 \$ right
534	5	-7.92		28	-29	18	-19	430	-435 u=-3 \$ bot
535	6	-2.644		28	-29	19	-20	430	-435 u=-3 \$ bot
536	6	-2.644		32	-33	23	-24	430	-435 u=-3 \$ top
537	5	-7.92		32	-33	24	-25	430	-435 u=-3 \$ top
538	0			10	-12	31	-38	410	-435 u=-3 \$ left
539	0			10	-12	37	-30	410	-435 u=-3 \$ left
540	0			15	-17	27	-38	410	-435 u=-3 \$ right
541	0			15	-17	37	-26	410	-435 u=-3 \$ right
542	0			35	-28	18	-20	410	-435 u=-3 \$ bot
543	0			29	-36	18	-20	410	-435 u=-3 \$ bot
544	0			35	-32	23	-25	410	-435 u=-3 \$ top

```

545 0          33 -36 23 -25 410 -435 u=-3 $ top
546 5 -7.92   12 -15 20 -23 (-13:-21:22:14) 400 -420 u=-3
547 5 -7.92   12 -15 20 -23 (-13:-21:22:14) 420 -430 u=-3
548 5 -7.92   12 -15 20 -23 (-13:-21:22:14) 430 -445 u=-3
549 5 -7.92   12 -15 20 -23 (-13:-21:22:14) 445 -460 u=-3
550 0          13 -14 21 -22 (-40:41:-42:43) 400 -455 u=-3
551 9 -1.17e-3          460          u=3
c fuel element
552 0          40 -41 42 -43          -415 u=3
553 5 -1.0783  40 -41 42 -43 415 -420 u=-3 $ lower nozzle
554 0          40 -41 42 -43 420 -425 u=-3 $ space
555 2 -3.8699  40 -41 42 -43 425 -430 u=-3 $ active fuel
556 5 -0.1591  40 -41 42 -43 430 -440 u=-3 $ space
557 5 -0.1591  40 -41 42 -43 440 -445 u=-3 $ plenum spacer
558 5 -1.5410  40 -41 42 -43 445 -455 u=-3 $ top nozzle
559 0          13 -14 21 -22 455 -460 u=-3
c
560 0          38 -23 -12 400 -420 u=3
561 0          20 -37 -12 400 -420 u=3
562 0          12 -35 23 400 -420 u=3
563 5 -7.92   12 -35 -20 400 -420 u=3
564 0          36 -15 23 400 -420 u=3
565 5 -7.92   36 -15 -20 400 -420 u=3
566 5 -7.92   38 -23 15 400 -420 u=3
567 5 -7.92   20 -37 15 400 -420 u=3
568 5 -7.92   38 -23 -12 420 -430 u=3
569 5 -7.92   20 -37 -12 420 -430 u=3
570 0          12 -35 23 420 -430 u=3
571 0          12 -35 -20 420 -430 u=3
572 5 -7.92   36 -15 23 420 -430 u=3
573 0          36 -15 -20 420 -430 u=3
574 5 -7.92   38 -23 15 420 -430 u=3
575 5 -7.92   20 -37 15 420 -430 u=3
576 5 -7.92   38 -23 -12 430 -445 u=3
577 5 -7.92   20 -37 -12 430 -445 u=3
578 0          12 -35 23 430 -445 u=3
579 0          12 -35 -20 430 -445 u=3
580 5 -7.92   36 -15 23 430 -445 u=3
581 0          36 -15 -20 430 -445 u=3
582 5 -7.92   38 -23 15 430 -445 u=3
583 5 -7.92   20 -37 15 430 -445 u=3
584 5 -7.92   38 -23 -12 445 -460 u=3
585 5 -7.92   20 -37 -12 445 -460 u=3
586 0          12 -35 23 445 -460 u=3
587 0          12 -35 -20 445 -460 u=3
588 5 -7.92   36 -15 23 445 -460 u=3
589 0          36 -15 -20 445 -460 u=3
590 5 -7.92   38 -23 15 445 -460 u=3
591 5 -7.92   20 -37 15 445 -460 u=3
592 0          23 -12          400 -460 u=3
593 0          23 15          400 -460 u=3
594 0          15 -20          400 -460 u=3
595 0          -12 -20          400 -460 u=3
c
c universe 4
c
601 0          (-40:41:-42:43) -400 u=4
602 0 37 -38 -12          400 -410 u=4
603 0 37 -38 15          400 -410 u=4
604 0 35 -36          -20 400 -410 u=4
605 0 35 -36          23 400 -410 u=4
606 0 37 -38 -12          435 -460 u=4
607 0 37 -38 15          435 -460 u=4
608 0 35 -36          -20 435 -460 u=4
609 0 35 -36          23 435 -460 u=4

```

610	0	37 -38	-10	410	-435	u=4	
611	0	37 -38	17	410	-435	u=4	
612	0	35 -36		-18	410	-435	u=4
613	0	35 -36	25	410	-435	u=4	
614	5	-7.92	10 -11 30 -31	410	-420	u=-4	\$ left
615	6	-2.644	11 -12 30 -31	410	-420	u=-4	\$ left
616	6	-2.644	15 -16 26 -27	410	-420	u=-4	\$ right
617	5	-7.92	16 -17 26 -27	410	-420	u=-4	\$ right
618	5	-7.92	32 -33 18 -19	410	-420	u=-4	\$ bot
619	6	-2.644	32 -33 19 -20	410	-420	u=-4	\$ bot
620	6	-2.644	28 -29 23 -24	410	-420	u=-4	\$ top
621	5	-7.92	28 -29 24 -25	410	-420	u=-4	\$ top
622	5	-7.92	10 -11 30 -31	420	-430	u=-4	\$ left
623	6	-2.644	11 -12 30 -31	420	-430	u=-4	\$ left
624	6	-2.644	15 -16 26 -27	420	-430	u=-4	\$ right
625	5	-7.92	16 -17 26 -27	420	-430	u=-4	\$ right
626	5	-7.92	32 -33 18 -19	420	-430	u=-4	\$ bot
627	6	-2.644	32 -33 19 -20	420	-430	u=-4	\$ bot
628	6	-2.644	28 -29 23 -24	420	-430	u=-4	\$ top
629	5	-7.92	28 -29 24 -25	420	-430	u=-4	\$ top
630	5	-7.92	10 -11 30 -31	430	-435	u=-4	\$ left
631	6	-2.644	11 -12 30 -31	430	-435	u=-4	\$ left
632	6	-2.644	15 -16 26 -27	430	-435	u=-4	\$ right
633	5	-7.92	16 -17 26 -27	430	-435	u=-4	\$ right
634	5	-7.92	32 -33 18 -19	430	-435	u=-4	\$ bot
635	6	-2.644	32 -33 19 -20	430	-435	u=-4	\$ bot
636	6	-2.644	28 -29 23 -24	430	-435	u=-4	\$ top
637	5	-7.92	28 -29 24 -25	430	-435	u=-4	\$ top
638	0		10 -12 31 -38	410	-435	u=-4	\$ left
639	0		10 -12 37 -30	410	-435	u=-4	\$ left
640	0		15 -17 27 -38	410	-435	u=-4	\$ right
641	0		15 -17 37 -26	410	-435	u=-4	\$ right
642	0		35 -32 18 -20	410	-435	u=-4	\$ bot
643	0		33 -36 18 -20	410	-435	u=-4	\$ bot
644	0		35 -28 23 -25	410	-435	u=-4	\$ top
645	0		29 -36 23 -25	410	-435	u=-4	\$ top
646	5	-7.92	12 -15 20 -23	(-13:-21:22:14)	400	-420	u=-4
647	5	-7.92	12 -15 20 -23	(-13:-21:22:14)	420	-430	u=-4
648	5	-7.92	12 -15 20 -23	(-13:-21:22:14)	430	-445	u=-4
649	5	-7.92	12 -15 20 -23	(-13:-21:22:14)	445	-460	u=-4
650	0		13 -14 21 -22	(-40:41:-42:43)	400	-455	u=-4
651	9	-1.17e-3			460	u=4	
c		fuel element					
652	0		40 -41 42 -43	-415	u=4		
653	5	-1.0783	40 -41 42 -43	415	-420	u=-4	\$ lower nozzle
654	0		40 -41 42 -43	420	-425	u=-4	\$ space
655	2	-3.8699	40 -41 42 -43	425	-430	u=-4	\$ active fuel
656	5	-0.1591	40 -41 42 -43	430	-440	u=-4	\$ space
657	5	-0.1591	40 -41 42 -43	440	-445	u=-4	\$ plenum spacer
658	5	-1.5410	40 -41 42 -43	445	-455	u=-4	\$ top nozzle
659	0		13 -14 21 -22	455	-460	u=-4	
c							
660	0		38 -23 -12	400	-420	u=4	
661	0		20 -37 -12	400	-420	u=4	
662	5	-7.92	12 -35 23	400	-420	u=4	
663	0		12 -35 -20	400	-420	u=4	
664	5	-7.92	36 -15 23	400	-420	u=4	
665	0		36 -15 -20	400	-420	u=4	
666	5	-7.92	38 -23 15	400	-420	u=4	
667	5	-7.92	20 -37 15	400	-420	u=4	
668	0		38 -23 -12	420	-430	u=4	
669	0		20 -37 -12	420	-430	u=4	
670	5	-7.92	12 -35 23	420	-430	u=4	
671	0		12 -35 -20	420	-430	u=4	
672	5	-7.92	36 -15 23	420	-430	u=4	

673	0		36 -15 -20	420 -430	u=4
674	5 -7.92		38 -23 15	420 -430	u=4
675	5 -7.92		20 -37 15	420 -430	u=4
676	0		38 -23 -12	430 -445	u=4
677	0		20 -37 -12	430 -445	u=4
678	5 -7.92		12 -35 23	430 -445	u=4
679	0		12 -35 -20	430 -445	u=4
680	5 -7.92		36 -15 23	430 -445	u=4
681	0		36 -15 -20	430 -445	u=4
682	5 -7.92		38 -23 15	430 -445	u=4
683	5 -7.92		20 -37 15	430 -445	u=4
684	0		38 -23 -12	445 -460	u=4
685	0		20 -37 -12	445 -460	u=4
686	5 -7.92		12 -35 23	445 -460	u=4
687	0		12 -35 -20	445 -460	u=4
688	5 -7.92		36 -15 23	445 -460	u=4
689	0		36 -15 -20	445 -460	u=4
690	5 -7.92		38 -23 15	445 -460	u=4
691	5 -7.92		20 -37 15	445 -460	u=4
692	0		23 -12	400 -460	u=4
693	0		23 15	400 -460	u=4
694	0		15 -20	400 -460	u=4
695	0		-12 -20	400 -460	u=4
c					
c	universe 5				
c					
701	0		(-40:41:-42:43)	-400	u=5
702	0	37 -38	-12	400 -410	u=5
703	0	37 -38	15	400 -410	u=5
704	0	35 -36		-20 400 -410	u=5
705	0	35 -36		23 400 -410	u=5
706	0	37 -38	-12	435 -460	u=5
707	0	37 -38	15	435 -460	u=5
708	0	35 -36		-20 435 -460	u=5
709	0	35 -36		23 435 -460	u=5
710	0	37 -38	-10	410 -435	u=5
711	0	37 -38	17	410 -435	u=5
712	0	35 -36		-18 410 -435	u=5
713	0	35 -36		25 410 -435	u=5
714	5 -7.92		10 -11 26 -27	410 -420	u=-5 \$ left
715	6 -2.644		11 -12 26 -27	410 -420	u=-5 \$ left
716	6 -2.644		15 -16 30 -31	410 -420	u=-5 \$ right
717	5 -7.92		16 -17 30 -31	410 -420	u=-5 \$ right
718	5 -7.92		32 -33 18 -19	410 -420	u=-5 \$ bot
719	6 -2.644		32 -33 19 -20	410 -420	u=-5 \$ bot
720	6 -2.644		28 -29 23 -24	410 -420	u=-5 \$ top
721	5 -7.92		28 -29 24 -25	410 -420	u=-5 \$ top
722	5 -7.92		10 -11 26 -27	420 -430	u=-5 \$ left
723	6 -2.644		11 -12 26 -27	420 -430	u=-5 \$ left
724	6 -2.644		15 -16 30 -31	420 -430	u=-5 \$ right
725	5 -7.92		16 -17 30 -31	420 -430	u=-5 \$ right
726	5 -7.92		32 -33 18 -19	420 -430	u=-5 \$ bot
727	6 -2.644		32 -33 19 -20	420 -430	u=-5 \$ bot
728	6 -2.644		28 -29 23 -24	420 -430	u=-5 \$ top
729	5 -7.92		28 -29 24 -25	420 -430	u=-5 \$ top
730	5 -7.92		10 -11 26 -27	430 -435	u=-5 \$ left
731	6 -2.644		11 -12 26 -27	430 -435	u=-5 \$ left
732	6 -2.644		15 -16 30 -31	430 -435	u=-5 \$ right
733	5 -7.92		16 -17 30 -31	430 -435	u=-5 \$ right
734	5 -7.92		32 -33 18 -19	430 -435	u=-5 \$ bot
735	6 -2.644		32 -33 19 -20	430 -435	u=-5 \$ bot
736	6 -2.644		28 -29 23 -24	430 -435	u=-5 \$ top
737	5 -7.92		28 -29 24 -25	430 -435	u=-5 \$ top
738	0		10 -12 27 -38	410 -435	u=-5 \$ left
739	0		10 -12 37 -26	410 -435	u=-5 \$ left


```

740 0 15 -17 31 -38 410 -435 u=-5 $ right
741 0 15 -17 37 -30 410 -435 u=-5 $ right
742 0 35 -32 18 -20 410 -435 u=-5 $ bot
743 0 33 -36 18 -20 410 -435 u=-5 $ bot
744 0 35 -28 23 -25 410 -435 u=-5 $ top
745 0 29 -36 23 -25 410 -435 u=-5 $ top
746 5 -7.92 12 -15 20 -23 (-13:-21:22:14) 400 -420 u=-5
747 5 -7.92 12 -15 20 -23 (-13:-21:22:14) 420 -430 u=-5
748 5 -7.92 12 -15 20 -23 (-13:-21:22:14) 430 -445 u=-5
749 5 -7.92 12 -15 20 -23 (-13:-21:22:14) 445 -460 u=-5
750 0 13 -14 21 -22 (-40:41:-42:43) 400 -455 u=-5
751 9 -1.17e-3 460 u=5
c fuel element
752 0 40 -41 42 -43 -415 u=5
753 5 -1.0783 40 -41 42 -43 415 -420 u=-5 $ lower nozzle
754 0 40 -41 42 -43 420 -425 u=-5 $ space
755 2 -3.8699 40 -41 42 -43 425 -430 u=-5 $ active fuel
756 5 -0.1591 40 -41 42 -43 430 -440 u=-5 $ space
757 5 -0.1591 40 -41 42 -43 440 -445 u=-5 $ plenum spacer
758 5 -1.5410 40 -41 42 -43 445 -455 u=-5 $ top nozzle
759 0 13 -14 21 -22 455 -460 u=-5
c
760 5 -7.92 38 -23 -12 400 -420 u=5
761 5 -7.92 20 -37 -12 400 -420 u=5
762 5 -7.92 12 -35 23 400 -420 u=5
763 0 12 -35 -20 400 -420 u=5
764 5 -7.92 36 -15 23 400 -420 u=5
765 0 36 -15 -20 400 -420 u=5
766 0 38 -23 15 400 -420 u=5
767 0 20 -37 15 400 -420 u=5
768 5 -7.92 38 -23 -12 420 -430 u=5
769 5 -7.92 20 -37 -12 420 -430 u=5
770 5 -7.92 12 -35 23 420 -430 u=5
771 0 12 -35 -20 420 -430 u=5
772 5 -7.92 36 -15 23 420 -430 u=5
773 0 36 -15 -20 420 -430 u=5
774 0 38 -23 15 420 -430 u=5
775 0 20 -37 15 420 -430 u=5
776 5 -7.92 38 -23 -12 430 -445 u=5
777 5 -7.92 20 -37 -12 430 -445 u=5
778 5 -7.92 12 -35 23 430 -445 u=5
779 0 12 -35 -20 430 -445 u=5
780 5 -7.92 36 -15 23 430 -445 u=5
781 0 36 -15 -20 430 -445 u=5
782 0 38 -23 15 430 -445 u=5
783 0 20 -37 15 430 -445 u=5
784 5 -7.92 38 -23 -12 445 -460 u=5
785 5 -7.92 20 -37 -12 445 -460 u=5
786 5 -7.92 12 -35 23 445 -460 u=5
787 0 12 -35 -20 445 -460 u=5
788 5 -7.92 36 -15 23 445 -460 u=5
789 0 36 -15 -20 445 -460 u=5
790 0 38 -23 15 445 -460 u=5
791 0 20 -37 15 445 -460 u=5
792 0 23 -12 400 -460 u=5
793 0 23 15 400 -460 u=5
794 0 15 -20 400 -460 u=5
795 0 -12 -20 400 -460 u=5
c
c egg crate
c
c storage locations
c
c 201 0 -301 -112 101 620 -675
202 0 -301 112 -113 101 620 -675

```

```

203 0 -301 113 -114 101      620 -675
c   204  0 -301 114      101      620 -675
c
205 0 -301      -111 102      620 -675
206 0 -301 111 -112 102 -101 620 -675
101 0 -301 112 -113 102 -101 620 -675
    fill=3 (-13.68679 68.43395 0.0)
102 0 -301 113 -114 102 -101 620 -675
    fill=2 ( 13.68679 68.43395 0.0)
207 0 -301 114 -115 102 -101 620 -675
208 0 -301 115      102      620 -675
c
c   209  0 -301      -110 103      620 -675
210 0 -301 110 -111 103 -102 620 -675
103 0      111 -112 103 -102 620 -675
    fill=3 (-41.06037 41.06037 0.0)
104 0      112 -113 103 -102 620 -675
    fill=1 (-13.68679 41.06037 0.0)
105 0      113 -114 103 -102 620 -675
    fill=1 ( 13.68679 41.06037 0.0)
106 0      114 -115 103 -102 620 -675
    fill=2 ( 41.06037 41.06037 0.0)
211 0 -301 115 -116 103 -102 620 -675
c   212  0 -301 116      103      620 -675
c
c   213  0 -301      -110 104 -103 620 -675
107 0 -301 110 -111 104 -103 620 -675
    fill=3 (-68.43395 13.68679 0.0)
108 0      111 -112 104 -103 620 -675
    fill=1 (-41.06037 13.68679 0.0)
109 0      112 -113 104 -103 620 -675
    fill=1 (-13.68679 13.68679 0.0)
110 0      113 -114 104 -103 620 -675
    fill=1 ( 13.68679 13.68679 0.0)
111 0      114 -115 104 -103 620 -675
    fill=1 ( 41.06037 13.68679 0.0)
112 0 -301 115 -116 104 -103 620 -675
    fill=2 ( 68.43395 13.68679 0.0)
214 0 -301 116      104 -103 620 -675
c
c   215  0 -301      -110 105 -104 620 -675
113 0 -301 110 -111 105 -104 620 -675
    fill=4 (-68.43395 -13.68679 0.0)
114 0      111 -112 105 -104 620 -675
    fill=1 (-41.06037 -13.68679 0.0)
115 0      112 -113 105 -104 620 -675
    fill=1 (-13.68679 -13.68679 0.0)
116 0      113 -114 105 -104 620 -675
    fill=1 ( 13.68679 -13.68679 0.0)
117 0      114 -115 105 -104 620 -675
    fill=1 ( 41.06037 -13.68679 0.0)
118 0 -301 115 -116 105 -104 620 -675
    fill=5 ( 68.43395 -13.68679 0.0)
216 0 -301 116      105 -104 620 -675
c
c   217  0 -301      -110      -105 620 -675
218 0 -301 110 -111 106 -105 620 -675
119 0      111 -112 106 -105 620 -675
    fill=4 (-41.06037 -41.06037 0.0)
120 0      112 -113 106 -105 620 -675
    fill=1 (-13.68679 -41.06037 0.0)
121 0      113 -114 106 -105 620 -675
    fill=1 ( 13.68679 -41.06037 0.0)
122 0      114 -115 106 -105 620 -675
    fill=5 ( 41.06037 -41.06037 0.0)

```

```

219 0 -301 115 -116 106 -105 620 -675
c 220 0 -301 116 -105 620 -675
c
221 0 -301 -111 -106 620 -675
222 0 -301 111 -112 107 -106 620 -675
123 0 -301 112 -113 107 -106 620 -675
fill=4 (-13.68679 -68.43395 0.0)
124 0 -301 113 -114 107 -106 620 -675
fill=5 ( 13.68679 -68.43395 0.0)
223 0 -301 114 -115 107 -106 620 -675
224 0 -301 115 -106 620 -675
c
c 225 0 -301 -112 -107 620 -675
226 0 -301 112 -113 -107 620 -675
227 0 -301 113 -114 -107 620 -675
c 228 0 -301 114 -107 620 -675
c
1001 5 -7.92 301 -302 610 -615 $ MPC shell
1003 5 -7.92 301 -302 615 -616 $ MPC shell
1005 5 -7.92 301 -302 616 -620 $ MPC shell
1007 5 -7.92 301 -302 620 -420 $ MPC shell
1009 5 -7.92 301 -302 420 -430 $ MPC shell
1011 5 -7.92 301 -302 430 -445 $ MPC shell
1013 5 -7.92 301 -302 445 -460 $ MPC shell
1014 5 -7.92 301 -302 460 -675 $ MPC shell
1015 5 -7.92 301 -302 675 -651 $ MPC shell
1017 5 -7.92 301 -302 651 -652 $ MPC shell
1019 5 -7.92 301 -302 652 -653 $ MPC shell
1021 5 -7.92 301 -302 653 -654 $ MPC shell
1023 5 -7.92 301 -302 654 -655 $ MPC shell
1025 5 -7.92 301 -302 655 -656 $ MPC shell
1027 5 -7.92 301 -302 656 -657 $ MPC shell
1028 5 -7.92 301 -302 657 -658 $ MPC shell
1029 5 -7.92 301 -302 658 -659 $ MPC shell
1031 5 -7.92 301 -302 659 -680 $ MPC shell
c
1051 5 -7.92 -301 610 -615 $ MPC baseplate
1052 5 -7.92 -301 615 -616 $ MPC baseplate
1053 5 -7.92 -301 616 -620 $ MPC baseplate
1060 5 -7.92 -301 675 -651 $ MPC lid
1061 5 -7.92 -301 651 -652 $ MPC lid
1062 5 -7.92 -301 652 -653 $ MPC lid
1063 5 -7.92 -301 653 -654 $ MPC lid
1064 5 -7.92 -301 654 -655 $ MPC lid
1065 5 -7.92 -301 655 -656 $ MPC lid
1066 5 -7.92 -301 656 -657 $ MPC lid
1067 5 -7.92 -301 657 -658 $ MPC lid
1068 5 -7.92 -301 658 -659 $ MPC lid
1069 5 -7.92 -301 659 -680 $ MPC lid
c
c overpack universes
c
c pedestal
c
2001 8 -7.82 -302 801 -610
2002 8 -7.82 -302 802 -801
2003 8 -7.82 -302 803 -802
2004 8 -7.82 -302 804 -803
2005 8 -7.82 -302 805 -804
c
2006 7 -2.35 -306 806 -805
2007 7 -2.35 -306 807 -806
2008 7 -2.35 -306 808 -807
2009 7 -2.35 -306 809 -808
2010 7 -2.35 -306 810 -809

```

2011	7	-2.35		-306	811	-810
2012	7	-2.35		-306	812	-811
2013	7	-2.35		-306	813	-812
2014	7	-2.35		-306	814	-813
c						
2016	8	-7.82	306	-302	806	-805
2017	8	-7.82	306	-302	807	-806
2028	8	-7.82	306	-302	808	-807
2019	8	-7.82	306	-302	809	-808
2020	8	-7.82	306	-302	810	-809
2021	8	-7.82	306	-302	811	-810
2022	8	-7.82	306	-302	812	-811
2023	8	-7.82	306	-302	813	-812
2024	8	-7.82	306	-302	814	-813
c						
overpack baseplate						
2031	8	-7.82		-302	815	-814
2032	8	-7.82		-302	816	-815
2033	7	-2.35		-302	817	-816
c						
gap between overpack and lid						
3001	9	-1.17e-3		-302	680	-901
c						
c						
lid						
c						
3002	8	-7.82		-307	901	-902
3003	8	-7.82		-307	902	-903
c						
3004	7	-2.35		-305	903	-904
3005	7	-2.35		-305	904	-905
3006	7	-2.35		-305	905	-906
3007	7	-2.35		-305	906	-907
3008	7	-2.35		-305	907	-908
3009	7	-2.35		-305	908	-909
c						
3010	8	-7.82	305	-307	903	-904
3011	8	-7.82	305	-307	904	-905
3012	8	-7.82	305	-307	905	-906
3013	8	-7.82	305	-307	906	-907
3014	8	-7.82	305	-307	907	-908
3015	8	-7.82	305	-307	908	-909
c						
3021	8	-7.82		-307	909	-910
3022	8	-7.82		-307	910	-911
3023	8	-7.82		-307	911	-912
3024	8	-7.82		-307	912	-913
c						
3030	0			-303	913	-914
3031	0			-303	914	-915
3032	0			-303	915	-916
3033	0			-303	916	-917
3034	0			-303	917	-918
c						
3035	8	-7.82	303	-304	913	-914
3036	8	-7.82	303	-304	914	-915
3037	8	-7.82	303	-304	915	-916
3038	8	-7.82	303	-304	916	-917
3039	0		303	-304	917	-918
c						
3040	7	-2.35	304	-307	913	-914
3041	7	-2.35	304	-307	914	-915
3042	7	-2.35	304	-307	915	-916
3043	7	-2.35	304	-307	916	-917
3044	0		304	-307	917	-918
c						
c						
c						
steel,concrete and air in gap between mpc and overpack						

```

c
4000 7 -2.35 302 -700 817 -816
4001 8 -7.82 302 -700 816 -815
4002 8 -7.82 302 -700 815 -814
4003 9 -1.17e-3 302 -700 814 -813
4004 9 -1.17e-3 302 -700 813 -812
4005 9 -1.17e-3 302 -700 812 -811
4006 9 -1.17e-3 302 -700 811 -810
4007 9 -1.17e-3 302 -700 810 -809
4008 9 -1.17e-3 302 -700 809 -808
4009 9 -1.17e-3 302 -700 808 -807
4010 9 -1.17e-3 302 -700 807 -806
4011 9 -1.17e-3 302 -700 806 -805
4012 9 -1.17e-3 302 -700 805 -804
4013 9 -1.17e-3 302 -700 804 -803
4014 9 -1.17e-3 302 -700 803 -802
4015 9 -1.17e-3 302 -700 802 -801
4016 9 -1.17e-3 302 -700 801 -610
4017 9 -1.17e-3 302 -700 610 -615
4018 9 -1.17e-3 302 -700 615 -620
4019 9 -1.17e-3 302 -700 620 -420
4020 9 -1.17e-3 302 -700 420 -430
4021 9 -1.17e-3 302 -700 430 -445
4022 9 -1.17e-3 302 -700 445 -460
4023 9 -1.17e-3 302 -700 460 -675
4024 9 -1.17e-3 302 -700 675 -651
4025 9 -1.17e-3 302 -700 651 -652
4026 9 -1.17e-3 302 -700 652 -653
4027 9 -1.17e-3 302 -700 653 -654
4028 9 -1.17e-3 302 -700 654 -655
4029 9 -1.17e-3 302 -700 655 -656
4030 9 -1.17e-3 302 -700 656 -657
4031 9 -1.17e-3 302 -700 657 -658
4032 9 -1.17e-3 302 -700 658 -659
4033 9 -1.17e-3 302 -700 659 -680
4034 9 -1.17e-3 302 -700 680 -901
4035 9 -1.17e-3 307 -700 901 -902
4036 9 -1.17e-3 307 -700 902 -903
4037 9 -1.17e-3 307 -700 903 -904
4038 9 -1.17e-3 307 -700 904 -905
4039 9 -1.17e-3 307 -700 905 -906
4040 9 -1.17e-3 307 -700 906 -907
4041 9 -1.17e-3 307 -700 907 -908
4042 9 -1.17e-3 307 -700 908 -909
4043 8 -7.82 307 -700 909 -910
4044 8 -7.82 307 -700 910 -911
4045 8 -7.82 307 -700 911 -912
4046 8 -7.82 307 -700 912 -913
4047 7 -2.35 307 -700 913 -914
4048 7 -2.35 307 -700 914 -915
4049 7 -2.35 307 -700 915 -916
4050 7 -2.35 307 -700 916 -917
4051 0 307 -700 917 -918

```

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c
importance splitting regions in overpack
c

```

```

5000 7 -2.35 700 -701 817 -816
5001 0 700 -701 816 -815 fill=10
5002 0 700 -701 815 -814 fill=10
5003 0 700 -701 814 -813 fill=11
5004 0 700 -701 813 -812 fill=11
5005 0 700 -701 812 -811 fill=11
5006 0 700 -701 811 -810 fill=11
5007 0 700 -701 810 -809 fill=11
5008 0 700 -701 809 -808 fill=11

```

5009	0	700	-701	808	-807	fill=11
5010	8	-7.82	700	-701	807	-806
5011	8	-7.82	700	-701	806	-805
5012	8	-7.82	700	-701	805	-804
5013	8	-7.82	700	-701	804	-803
5014	8	-7.82	700	-701	803	-802
5015	8	-7.82	700	-701	802	-801
5016	8	-7.82	700	-701	801	-610
5017	8	-7.82	700	-701	610	-615
5018	8	-7.82	700	-701	615	-620
5019	8	-7.82	700	-701	620	-420
5020	8	-7.82	700	-701	420	-430
5021	8	-7.82	700	-701	430	-445
5022	8	-7.82	700	-701	445	-460
5023	8	-7.82	700	-701	460	-675
5024	8	-7.82	700	-701	675	-651
5025	8	-7.82	700	-701	651	-652
5026	8	-7.82	700	-701	652	-653
5027	8	-7.82	700	-701	653	-654
5028	8	-7.82	700	-701	654	-655
5029	8	-7.82	700	-701	655	-656
5030	8	-7.82	700	-701	656	-657
5031	8	-7.82	700	-701	657	-658
5032	8	-7.82	700	-701	658	-659
5033	8	-7.82	700	-701	659	-680
5034	8	-7.82	700	-701	680	-901
5035	8	-7.82	700	-701	901	-902
5036	8	-7.82	700	-701	902	-903
5037	8	-7.82	700	-701	903	-904
5038	0	700	-701	904	-905	fill=13
5039	0	700	-701	905	-906	fill=13
5040	0	700	-701	906	-907	fill=13
5041	0	700	-701	907	-908	fill=13
5042	0	700	-701	908	-909	fill=13
5043	8	-7.82	700	-701	909	-910
5044	8	-7.82	700	-701	910	-911
5045	8	-7.82	700	-701	911	-912
5046	8	-7.82	700	-701	912	-913
5047	7	-2.35	700	-701	913	-914
5048	7	-2.35	700	-701	914	-915
5049	7	-2.35	700	-701	915	-916
5050	7	-2.35	700	-701	916	-917
5051	0	700	-701	917	-918	
c						
5100	7	-2.35	701	-702	817	-816
5101	0	701	-702	816	-815	fill=10
5102	0	701	-702	815	-814	fill=10
5103	0	701	-702	814	-813	fill=11
5104	0	701	-702	813	-812	fill=11
5105	0	701	-702	812	-811	fill=11
5106	0	701	-702	811	-810	fill=11
5107	0	701	-702	810	-809	fill=11
5108	0	701	-702	809	-808	fill=11
5109	0	701	-702	808	-807	fill=11
5110	8	-7.82	701	-311	807	-806
5111	8	-7.82	701	-311	806	-805
5112	8	-7.82	701	-311	805	-804
5113	8	-7.82	701	-311	804	-803
5114	8	-7.82	701	-311	803	-802
5115	8	-7.82	701	-311	802	-801
5116	8	-7.82	701	-311	801	-610
5117	8	-7.82	701	-311	610	-615
5118	8	-7.82	701	-311	615	-620
5119	8	-7.82	701	-311	620	-420
5120	8	-7.82	701	-311	420	-430

5121	8	-7.82	701	-311	430	-445	
5122	8	-7.82	701	-311	445	-460	
5123	8	-7.82	701	-311	460	-675	
5124	8	-7.82	701	-311	675	-651	
5125	8	-7.82	701	-311	651	-652	
5126	8	-7.82	701	-311	652	-653	
5127	8	-7.82	701	-311	653	-654	
5128	8	-7.82	701	-311	654	-655	
5129	8	-7.82	701	-311	655	-656	
5130	8	-7.82	701	-311	656	-657	
5131	8	-7.82	701	-311	657	-658	
5132	8	-7.82	701	-311	658	-659	
5133	8	-7.82	701	-311	659	-680	
5134	8	-7.82	701	-311	680	-901	
5135	8	-7.82	701	-311	901	-902	
5136	8	-7.82	701	-311	902	-903	
5137	8	-7.82	701	-311	903	-904	
5138	0		701	-702	904	-905	fill=13
5139	0		701	-702	905	-906	fill=13
5140	0		701	-702	906	-907	fill=13
5141	0		701	-702	907	-908	fill=13
5142	0		701	-702	908	-909	fill=13
5143	8	-7.82	701	-702	909	-910	
5144	8	-7.82	701	-702	910	-911	
5145	8	-7.82	701	-702	911	-912	
5146	8	-7.82	701	-702	912	-913	
5147	7	-2.35	701	-702	913	-914	
5148	7	-2.35	701	-702	914	-915	
5149	7	-2.35	701	-702	915	-916	
5150	7	-2.35	701	-702	916	-917	
5151	0		701	-702	917	-918	
c							
5200	7	-2.35	702	-703	817	-816	
5201	0		702	-703	816	-815	fill=10
5202	0		702	-703	815	-814	fill=10
5203	0		702	-703	814	-813	fill=11
5204	0		702	-703	813	-812	fill=11
5205	0		702	-703	812	-811	fill=11
5206	0		702	-703	811	-810	fill=11
5207	0		702	-703	810	-809	fill=11
5208	0		702	-703	809	-808	fill=11
5209	0		702	-703	808	-807	fill=11
5210	0		311	-703	807	-806	fill=112
5211	0		311	-703	806	-805	fill=112
5212	0		311	-703	805	-804	fill=112
5213	0		311	-703	804	-803	fill=112
5214	0		311	-703	803	-802	fill=112
5215	0		311	-703	802	-801	fill=112
5216	0		311	-703	801	-610	fill=112
5217	0		311	-703	610	-615	fill=112
5218	0		311	-703	615	-620	fill=112
5219	0		311	-703	620	-420	fill=112
5220	0		311	-703	420	-430	fill=112
5221	0		311	-703	430	-445	fill=112
5222	0		311	-703	445	-460	fill=112
5223	0		311	-703	460	-675	fill=112
5224	0		311	-703	675	-651	fill=112
5225	0		311	-703	651	-652	fill=112
5226	0		311	-703	652	-653	fill=112
5227	0		311	-703	653	-654	fill=112
5228	0		311	-703	654	-655	fill=112
5229	0		311	-703	655	-656	fill=112
5230	0		311	-703	656	-657	fill=112
5231	0		311	-703	657	-658	fill=112
5232	0		311	-703	658	-659	fill=112

5233	0	311	-703	659	-680	fill=112
5234	0	311	-703	680	-901	fill=112
5235	0	311	-703	901	-902	fill=112
5236	0	311	-703	902	-903	fill=112
5237	0	311	-703	903	-904	fill=112
5238	0	702	-703	904	-905	fill=13
5239	0	702	-703	905	-906	fill=13
5240	0	702	-703	906	-907	fill=13
5241	0	702	-703	907	-908	fill=13
5242	0	702	-703	908	-909	fill=13
5243	8	-7.82	702	-703	909	-910
5244	8	-7.82	702	-703	910	-911
5245	8	-7.82	702	-703	911	-912
5246	8	-7.82	702	-703	912	-913
5247	7	-2.35	702	-703	913	-914
5248	7	-2.35	702	-703	914	-915
5249	7	-2.35	702	-703	915	-916
5250	7	-2.35	702	-703	916	-917
5251	0	702	-703	917	-918	
c						
5300	7	-2.35	703	-705	817	-816
5301	0	703	-705	816	-815	fill=10
5302	0	703	-705	815	-814	fill=10
5303	0	703	-705	814	-813	fill=11
5304	0	703	-705	813	-812	fill=11
5305	0	703	-705	812	-811	fill=11
5306	0	703	-705	811	-810	fill=11
5307	0	703	-705	810	-809	fill=11
5308	0	703	-705	809	-808	fill=11
5309	0	703	-705	808	-807	fill=11
5310	0	703	-705	807	-806	fill=112
5311	0	703	-705	806	-805	fill=112
5312	0	703	-705	805	-804	fill=112
5313	0	703	-705	804	-803	fill=112
5314	0	703	-705	803	-802	fill=112
5315	0	703	-705	802	-801	fill=112
5316	0	703	-705	801	-610	fill=112
5317	0	703	-705	610	-615	fill=112
5318	0	703	-705	615	-620	fill=112
5319	0	703	-705	620	-420	fill=112
5320	0	703	-705	420	-430	fill=112
5321	0	703	-705	430	-445	fill=112
5322	0	703	-705	445	-460	fill=112
5323	0	703	-705	460	-675	fill=112
5324	0	703	-705	675	-651	fill=112
5325	0	703	-705	651	-652	fill=112
5326	0	703	-705	652	-653	fill=112
5327	0	703	-705	653	-654	fill=112
5328	0	703	-705	654	-655	fill=112
5329	0	703	-705	655	-656	fill=112
5330	0	703	-705	656	-657	fill=112
5331	0	703	-705	657	-658	fill=112
5332	0	703	-705	658	-659	fill=112
5333	0	703	-705	659	-680	fill=112
5334	0	703	-705	680	-901	fill=112
5335	0	703	-705	901	-902	fill=112
5336	0	703	-705	902	-903	fill=112
5337	0	703	-705	903	-904	fill=112
5338	0	703	-705	904	-905	fill=13
5339	0	703	-705	905	-906	fill=13
5340	0	703	-705	906	-907	fill=13
5341	0	703	-705	907	-908	fill=13
5342	0	703	-705	908	-909	fill=13
5343	8	-7.82	703	-705	909	-910
5344	8	-7.82	703	-705	910	-911

5345	8	-7.82	703	-705	911	-912
5346	8	-7.82	703	-705	912	-913
5347	0		703	-705	913	-914 fill=14
5348	0		703	-705	914	-915 fill=14
5349	0		703	-705	915	-916 fill=14
5350	0		703	-705	916	-917 fill=14
5351	0		703	-705	917	-918
c						
c	5401	0	704	-705	816	-815 fill=10
c	5402	0	704	-705	815	-814 fill=10
c	5403	0	704	-705	814	-813 fill=11
c	5404	0	704	-705	813	-812 fill=11
c	5405	0	704	-705	812	-811 fill=11
c	5406	0	704	-705	811	-810 fill=11
c	5407	0	704	-705	810	-809 fill=11
c	5408	0	704	-705	809	-808 fill=11
c	5409	0	704	-705	808	-807 fill=11
c	5410	0	704	-705	807	-806 fill=112
c	5411	0	704	-705	806	-805 fill=112
c	5412	0	704	-705	805	-804 fill=112
c	5413	0	704	-705	804	-803 fill=112
c	5414	0	704	-705	803	-802 fill=112
c	5415	0	704	-705	802	-801 fill=112
c	5416	0	704	-705	801	-610 fill=112
c	5417	0	704	-705	610	-615 fill=112
c	5418	0	704	-705	615	-620 fill=112
c	5419	0	704	-705	620	-420 fill=112
c	5420	0	704	-705	420	-430 fill=112
c	5421	0	704	-705	430	-445 fill=112
c	5422	0	704	-705	445	-460 fill=112
c	5423	0	704	-705	460	-675 fill=112
c	5424	0	704	-705	675	-651 fill=112
c	5425	0	704	-705	651	-652 fill=112
c	5426	0	704	-705	652	-653 fill=112
c	5427	0	704	-705	653	-654 fill=112
c	5428	0	704	-705	654	-655 fill=112
c	5429	0	704	-705	655	-656 fill=112
c	5430	0	704	-705	656	-657 fill=112
c	5431	0	704	-705	657	-658 fill=112
c	5432	0	704	-705	658	-659 fill=112
c	5433	0	704	-705	659	-680 fill=112
c	5434	0	704	-705	680	-901 fill=112
c	5435	0	704	-705	901	-902 fill=112
c	5436	0	704	-705	902	-903 fill=112
c	5437	0	704	-705	903	-904 fill=112
c	5438	0	704	-705	904	-905 fill=113
c	5439	0	704	-705	905	-906 fill=13
c	5440	0	704	-705	906	-907 fill=13
c	5441	0	704	-705	907	-908 fill=13
c	5442	0	704	-705	908	-909 fill=13
c	5443	8	-7.82	704	-705	909 -910
c	5444	8	-7.82	704	-705	910 -911
c	5445	8	-7.82	704	-705	911 -912
c	5446	8	-7.82	704	-705	912 -913
c	5447	0	704	-705	913	-914 fill=14
c	5448	0	704	-705	914	-915 fill=14
c	5449	0	704	-705	915	-916 fill=14
c	5450	0	704	-705	916	-917 fill=14
c	5451	0	704	-705	917	-918
c						
5500	7	-2.35	705	-707	817	-816
5501	0		705	-707	816	-815 fill=10
5502	0		705	-707	815	-814 fill=10
5503	0		705	-707	814	-813 fill=11
5504	0		705	-707	813	-812 fill=11

5505	0	705	-707	812	-811	fill=11
5506	0	705	-707	811	-810	fill=11
5507	0	705	-707	810	-809	fill=11
5508	0	705	-707	809	-808	fill=11
5509	0	705	-707	808	-807	fill=11
5510	0	705	-707	807	-806	fill=112
5511	0	705	-707	806	-805	fill=112
5512	0	705	-707	805	-804	fill=112
5513	0	705	-707	804	-803	fill=112
5514	0	705	-707	803	-802	fill=112
5515	0	705	-707	802	-801	fill=112
5516	0	705	-707	801	-610	fill=112
5517	0	705	-707	610	-615	fill=112
5518	0	705	-707	615	-620	fill=112
5519	0	705	-707	620	-420	fill=112
5520	0	705	-707	420	-430	fill=112
5521	0	705	-707	430	-445	fill=112
5522	0	705	-707	445	-460	fill=112
5523	0	705	-707	460	-675	fill=112
5524	0	705	-707	675	-651	fill=112
5525	0	705	-707	651	-652	fill=112
5526	0	705	-707	652	-653	fill=112
5527	0	705	-707	653	-654	fill=112
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c	6246	8	-7.82	712	-713	912	-913
c	6247	0	712	-713	913	-914	
c	6248	0	712	-713	914	-915	
c	6249	0	712	-713	915	-916	
c	6250	0	712	-713	916	-917	
c	6251	0	712	-713	917	-918	
c							
6300	7	-2.35	713	-714	817	-816	
6301	0		713	-714	816	-815	fill=10
6302	0		713	-714	815	-814	fill=10
6303	0		713	-714	814	-813	fill=11
6304	0		713	-714	813	-812	fill=11
6305	0		713	-714	812	-811	fill=11
6306	0		713	-714	811	-810	fill=11
6307	0		713	-714	810	-809	fill=11
6308	0		713	-714	809	-808	fill=11
6309	0		713	-714	808	-807	fill=11
6310	0		713	-714	807	-806	fill=112
6311	0		713	-714	806	-805	fill=112
6312	0		713	-714	805	-804	fill=112
6313	0		713	-714	804	-803	fill=112
6314	0		713	-714	803	-802	fill=112
6315	0		713	-714	802	-801	fill=112
6316	0		713	-714	801	-610	fill=112
6317	0		713	-714	610	-615	fill=112
6318	0		713	-714	615	-620	fill=112
6319	0		713	-714	620	-420	fill=112
6320	0		713	-714	420	-430	fill=112
6321	0		713	-714	430	-445	fill=112
6322	0		713	-714	445	-460	fill=112
6323	0		713	-714	460	-675	fill=112
6324	0		713	-714	675	-651	fill=112
6325	0		713	-714	651	-652	fill=112
6326	0		713	-714	652	-653	fill=112
6327	0		713	-714	653	-654	fill=112
6328	0		713	-714	654	-655	fill=112
6329	0		713	-714	655	-656	fill=112
6330	0		713	-714	656	-657	fill=112
6331	0		713	-714	657	-658	fill=112
6332	0		713	-714	658	-659	fill=112
6333	0		713	-714	659	-680	fill=112
6334	0		713	-714	680	-901	fill=112
6335	0		713	-714	901	-902	fill=112
6336	0		713	-714	902	-903	fill=112
6337	0		713	-714	903	-904	fill=112
6338	0		713	-714	904	-905	fill=13
6339	0		713	-714	905	-906	fill=13

6340	0	713	-714	906	-907	fill=13
6341	0	713	-714	907	-908	fill=13
6342	0	713	-714	908	-909	fill=13
6343	8	-7.82	713	-714	909	-910
6344	8	-7.82	713	-714	910	-911
6345	8	-7.82	713	-714	911	-912
6346	8	-7.82	713	-714	912	-913
c	6347	0	713	-714	913	-914
c	6348	0	713	-714	914	-915
c	6349	0	713	-714	915	-916
c	6350	0	713	-714	916	-917
c	6351	0	713	-714	917	-918
c						
6400	7	-2.35	714	-715	817	-816
6401	0	714	-715	816	-815	fill=10
6402	0	714	-715	815	-814	fill=10
6403	0	714	-715	814	-813	fill=11
6404	0	714	-715	813	-812	fill=11
6405	0	714	-715	812	-811	fill=11
6406	0	714	-715	811	-810	fill=11
6407	0	714	-715	810	-809	fill=11
6408	0	714	-715	809	-808	fill=11
6409	0	714	-715	808	-807	fill=11
6410	0	714	-715	807	-806	fill=112
6411	0	714	-715	806	-805	fill=112
6412	0	714	-715	805	-804	fill=112
6413	0	714	-715	804	-803	fill=112
6414	0	714	-715	803	-802	fill=112
6415	0	714	-715	802	-801	fill=112
6416	0	714	-715	801	-610	fill=112
6417	0	714	-715	610	-615	fill=112
6418	0	714	-715	615	-620	fill=112
6419	0	714	-715	620	-420	fill=112
6420	0	714	-715	420	-430	fill=112
6421	0	714	-715	430	-445	fill=112
6422	0	714	-715	445	-460	fill=112
6423	0	714	-715	460	-675	fill=112
6424	0	714	-715	675	-651	fill=112
6425	0	714	-715	651	-652	fill=112
6426	0	714	-715	652	-653	fill=112
6427	0	714	-715	653	-654	fill=112
6428	0	714	-715	654	-655	fill=112
6429	0	714	-715	655	-656	fill=112
6430	0	714	-715	656	-657	fill=112
6431	0	714	-715	657	-658	fill=112
6432	0	714	-715	658	-659	fill=112
6433	0	714	-715	659	-680	fill=112
6434	0	714	-715	680	-901	fill=112
6435	0	714	-715	901	-902	fill=112
6436	0	714	-715	902	-903	fill=112
6437	0	714	-715	903	-904	fill=112
6438	0	714	-715	904	-905	fill=13
6439	0	714	-715	905	-906	fill=13
6440	0	714	-715	906	-907	fill=13
6441	0	714	-715	907	-908	fill=13
6442	0	714	-715	908	-909	fill=13
6443	0	714	-715	909	-910	fill=15
6444	0	714	-715	910	-911	fill=15
6445	0	714	-715	911	-912	fill=15
6446	0	714	-715	912	-913	fill=15
c	6447	0	714	-715	913	-914
c	6448	0	714	-715	914	-915
c	6449	0	714	-715	915	-916
c	6450	0	714	-715	916	-917
c	6451	0	714	-715	917	-918

```

c
6500 7 -2.35 715 -716 817 -816
6501 0 715 -716 816 -815 fill=10
6502 0 715 -716 815 -814 fill=10
6503 0 715 -716 814 -813 fill=11
6504 0 715 -716 813 -812 fill=11
6505 0 715 -716 812 -811 fill=11
6506 0 715 -716 811 -810 fill=11
6507 0 715 -716 810 -809 fill=11
6508 0 715 -716 809 -808 fill=11
6509 0 715 -716 808 -807 fill=11
6510 0 715 -716 807 -806 fill=12
6511 0 715 -716 806 -805 fill=12
6512 0 715 -716 805 -804 fill=12
6513 0 715 -716 804 -803 fill=12
6514 0 715 -716 803 -802 fill=12
6515 0 715 -716 802 -801 fill=12
6516 0 715 -716 801 -610 fill=12
6517 0 715 -716 610 -615 fill=12
6518 0 715 -716 615 -620 fill=12
6519 0 715 -716 620 -420 fill=12
6520 0 715 -716 420 -430 fill=12
6521 0 715 -716 430 -445 fill=12
6522 0 715 -716 445 -460 fill=12
6523 0 715 -716 460 -675 fill=12
6524 0 715 -716 675 -651 fill=12
6525 0 715 -716 651 -652 fill=12
6526 0 715 -716 652 -653 fill=12
6527 0 715 -716 653 -654 fill=12
6528 0 715 -716 654 -655 fill=12
6529 0 715 -716 655 -656 fill=12
6530 0 715 -716 656 -657 fill=12
6531 0 715 -716 657 -658 fill=12
6532 0 715 -716 658 -659 fill=12
6533 0 715 -716 659 -680 fill=12
6534 0 715 -716 680 -901 fill=12
6535 0 715 -716 901 -902 fill=12
6536 0 715 -716 902 -903 fill=12
6537 0 715 -716 903 -904 fill=12
6538 0 715 -716 904 -905 fill=13
6539 0 715 -716 905 -906 fill=13
6540 0 715 -716 906 -907 fill=13
6541 0 715 -716 907 -908 fill=13
6542 0 715 -716 908 -909 fill=13
6543 0 715 -716 909 -910
6544 0 715 -716 910 -911
6545 0 715 -716 911 -912
6546 0 715 -716 912 -913
c 6547 0 715 -716 913 -914
c 6548 0 715 -716 914 -915
c 6549 0 715 -716 915 -916
c 6550 0 715 -716 916 -917
c 6551 0 715 -716 917 -918
c
6601 0 716 -717 816 -815
6602 0 716 -717 815 -814
6603 0 716 -717 814 -813
6604 0 716 -717 813 -812
6605 0 716 -717 812 -811
6606 0 716 -717 811 -810
6607 0 716 -717 810 -809
6608 0 716 -717 809 -808
6609 0 716 -717 808 -807
6610 0 716 -717 807 -806
6611 0 716 -717 806 -805

```

6612 0 716 -717 805 -804
 6613 0 716 -717 804 -803
 6614 0 716 -717 803 -802
 6615 0 716 -717 802 -801
 6616 0 716 -717 801 -610
 6617 0 716 -717 610 -615
 6618 0 716 -717 615 -620
 6619 0 716 -717 620 -420
 6620 0 716 -717 420 -430
 6621 0 716 -717 430 -445
 6622 0 716 -717 445 -460
 6623 0 716 -717 460 -675
 6624 0 716 -717 675 -651
 6625 0 716 -717 651 -652
 6626 0 716 -717 652 -653
 6627 0 716 -717 653 -654
 6628 0 716 -717 654 -655
 6629 0 716 -717 655 -656
 6630 0 716 -717 656 -657
 6631 0 716 -717 657 -658
 6632 0 716 -717 658 -659
 6633 0 716 -717 659 -680
 6634 0 716 -717 680 -901
 6635 0 716 -717 901 -902
 6636 0 716 -717 902 -903
 6637 0 716 -717 903 -904
 6638 0 716 -717 904 -905
 6639 0 716 -717 905 -906
 6640 0 716 -717 906 -907
 6641 0 716 -717 907 -908
 6642 0 716 -717 908 -909
 6643 0 716 -717 909 -910
 6644 0 716 -717 910 -911
 6645 0 716 -717 911 -912
 6646 0 716 -717 912 -913
 6647 0 716 -717 913 -914
 6648 0 716 -717 914 -915
 6649 0 716 -717 915 -916
 6650 0 716 -717 916 -917
 6651 0 716 -717 917 -918

c
 overpack universes
 c overpack baseplate
 10001 8 -7.82 371 -372 373 -374 u=10
 10002 8 -7.82 374 -362 u=10
 10003 8 -7.82 374 363 u=10
 10004 9 -1.17e-3 374 362 -363 u=10
 10005 8 -7.82 -371 367 -374 u=10
 10006 8 -7.82 -371 373 -366 u=10
 10007 9 -1.17e-3 -371 366 -367 u=10
 10008 8 -7.82 -373 -362 u=10
 10009 8 -7.82 -373 363 u=10
 10010 9 -1.17e-3 -373 362 -363 u=10
 10011 8 -7.82 372 373 -366 u=10
 10012 8 -7.82 372 367 -374 u=10
 10013 9 -1.17e-3 372 366 -367 u=10
 c
 walls and top of bottom duct
 10101 8 -7.82 361 -362 -932 u=11
 10102 8 -7.82 363 -364 -932 u=11
 10103 8 -7.82 362 -363 931 -932 u=11
 c
 10104 8 -7.82 365 -366 -932 u=11
 10105 8 -7.82 367 -368 -932 u=11
 10106 8 -7.82 366 -367 931 -932 u=11

```

c      inner and outer shell between bottom ducts
10107  8 -7.82      368 364 -932 315   u=11
10108  8 -7.82      368 -361 -932 315   u=11
10109  8 -7.82     -365 364 -932 315   u=11
10110  8 -7.82     -365 -361 -932 315   u=11
c
10111  8 -7.82      368 364 -932 -310  u=11
10112  8 -7.82      368 -361 -932 -310  u=11
10113  8 -7.82     -365 364 -932 -310  u=11
10114  8 -7.82     -365 -361 -932 -310  u=11
c      concrete and radial plates between bottom ducts
10121  8 -7.82      310 -315 391 -392  -932 u=11
10122  8 -7.82      310 -315 393 -394  -932 u=11
c
10131  7 -2.35      310 -315 394 -365  -932 u=11
10132  7 -2.35      310 -315 368 -391  -932 u=11
10133  7 -2.35      310 -315 392  364  -932 u=11
10134  7 -2.35      310 -315 -361 394  -932 u=11
10135  7 -2.35      310 -315 368 -393  -932 u=11
10136  7 -2.35      310 -315 392 -365  -932 u=11
10137  7 -2.35      310 -315 -391 -361  -932 u=11
10138  7 -2.35      310 -315 364 -393  -932 u=11
c      air and grid spacers in bottom ducts
10141  9 -1.17e-3  362 -363 -931 263 -264 u=11
10142  9 -1.17e-3  366 -367 -931 261 -262 u=11
c
10143  9 -1.17e-3  362 -201 -931 (-263:264) u=11
10144  9 -1.17e-3  206 -363 -931 (-263:264) u=11
10145  9 -1.17e-3  201 -202 -221 (-263:264) u=11
10146  5 -7.92      202 -203 -221 (-273:274) u=11
11146  9 -1.17e-3  202 -203 -221 273 -274 (-263:264) u=11
10147  9 -1.17e-3  203 -204 -221 (-263:264) u=11
10148  5 -7.92      204 -205 -221 (-273:274) u=11
11148  9 -1.17e-3  204 -205 -221 273 -274 (-263:264) u=11
10149  9 -1.17e-3  205 -206 -221 (-263:264) u=11
10150  5 -7.92      201 -202 221 -222 (-263:264) u=11
10151  5 -7.92      202 -203 221 -222 (-263:264) u=11
10152  5 -7.92      203 -204 221 -222 (-263:264) u=11
10153  5 -7.92      204 -205 221 -222 (-263:264) u=11
10154  5 -7.92      205 -206 221 -222 (-263:264) u=11
10155  9 -1.17e-3  201 -202 222 -223 (-263:264) u=11
10156  5 -7.92      202 -203 222 -223 (-263:264) u=11
10157  9 -1.17e-3  203 -204 222 -223 (-263:264) u=11
10158  5 -7.92      204 -205 222 -223 (-263:264) u=11
10159  9 -1.17e-3  205 -206 222 -223 (-263:264) u=11
10160  5 -7.92      201 -202 223 -224 (-263:264) u=11
10161  5 -7.92      202 -203 223 -224 (-263:264) u=11
10162  5 -7.92      203 -204 223 -224 (-263:264) u=11
10163  5 -7.92      204 -205 223 -224 (-263:264) u=11
10164  5 -7.92      205 -206 223 -224 (-263:264) u=11
10165  9 -1.17e-3  201 -202 224 -225 (-263:264) u=11
10166  5 -7.92      202 -203 224 -225 (-263:264) u=11
10167  9 -1.17e-3  203 -204 224 -225 (-263:264) u=11
10168  5 -7.92      204 -205 224 -225 (-263:264) u=11
10169  9 -1.17e-3  205 -206 224 -225 (-263:264) u=11
10170  9 -1.17e-3  201 -206 225 -931 (-263:264) u=11
c
10243  9 -1.17e-3  366 -211 -931 (-261:262) u=11
10244  9 -1.17e-3  216 -367 -931 (-261:262) u=11
10245  9 -1.17e-3  211 -212 -221 (-261:262) u=11
10246  5 -7.92      212 -213 -221 (-261:262) u=11
10247  9 -1.17e-3  213 -214 -221 (-261:262) u=11
10248  5 -7.92      214 -215 -221 (-261:262) u=11
10249  9 -1.17e-3  215 -216 -221 (-261:262) u=11
10250  5 -7.92      211 -212 221 -222 (-261:262) u=11

```

10251 5 -7.92 212 -213 221 -222 (-261:262) u=11
 10252 5 -7.92 213 -214 221 -222 (-261:262) u=11
 10253 5 -7.92 214 -215 221 -222 (-261:262) u=11
 10254 5 -7.92 215 -216 221 -222 (-261:262) u=11
 10255 9 -1.17e-3 211 -212 222 -223 (-261:262) u=11
 10256 5 -7.92 212 -213 222 -223 (-261:262) u=11
 10257 9 -1.17e-3 213 -214 222 -223 (-261:262) u=11
 10258 5 -7.92 214 -215 222 -223 (-261:262) u=11
 10259 9 -1.17e-3 215 -216 222 -223 (-261:262) u=11
 10260 5 -7.92 211 -212 223 -224 (-261:262) u=11
 10261 5 -7.92 212 -213 223 -224 (-261:262) u=11
 10262 5 -7.92 213 -214 223 -224 (-261:262) u=11
 10263 5 -7.92 214 -215 223 -224 (-261:262) u=11
 10264 5 -7.92 215 -216 223 -224 (-261:262) u=11
 10265 9 -1.17e-3 211 -212 224 -225 (-261:262) u=11
 10266 5 -7.92 212 -213 224 -225 (-261:262) u=11
 10267 9 -1.17e-3 213 -214 224 -225 (-261:262) u=11
 10268 5 -7.92 214 -215 224 -225 (-261:262) u=11
 10269 9 -1.17e-3 215 -216 224 -225 (-261:262) u=11
 10270 9 -1.17e-3 211 -216 225 -931 (-261:262) u=11

c
c inner, outer shells and concrete between top and bot ducts

10301 8 -7.82 932 -311 u=11
 10302 8 -7.82 932 315 u=11
 10303 8 -7.82 932 311 -315 391 -392 u=11
 10304 8 -7.82 932 311 -315 393 -394 u=11
 10305 7 -2.35 932 311 -315 394 -391 u=11
 10306 7 -2.35 932 311 -315 -392 394 u=11
 10307 7 -2.35 932 311 -315 392 -393 u=11
 10308 7 -2.35 932 311 -315 -391 -393 u=11

11302 8 -7.82 315 u=12
 11303 8 -7.82 -315 391 -392 u=12
 11304 8 -7.82 -315 393 -394 u=12
 11305 7 -2.35 -315 394 -391 u=12
 11306 7 -2.35 -315 392 394 u=12
 11307 7 -2.35 -315 392 -393 u=12
 11308 7 -2.35 -315 -391 -393 u=12

13303 8 -7.82 391 -392 u=112
 13304 8 -7.82 393 -394 u=112
 13305 7 -2.35 394 -391 u=112
 13306 7 -2.35 392 394 u=112
 13307 7 -2.35 392 -393 u=112
 13308 7 -2.35 -391 -393 u=112

12301 8 -7.82 -933 -311 u=13
 12302 8 -7.82 -933 315 u=13
 12303 8 -7.82 -933 311 -315 391 -392 u=13
 12304 8 -7.82 -933 311 -315 393 -394 u=13
 12305 7 -2.35 -933 311 -315 394 -391 u=13
 12306 7 -2.35 -933 311 -315 392 394 u=13
 12307 7 -2.35 -933 311 -315 392 -393 u=13
 12308 7 -2.35 -933 311 -315 -391 -393 u=13

c top duct bottom plates

10309 8 -7.82 933 -934 351 -354 u=13
 10310 8 -7.82 933 -934 355 -358 u=13

c top duct walls

10311 8 -7.82 934 351 -352 u=13
 10312 8 -7.82 934 353 -354 u=13
 10313 8 -7.82 934 355 -356 u=13
 10314 8 -7.82 934 357 -358 u=13

c inner and outer shell between top ducts

10407 8 -7.82 358 354 933 315 u=13

```

10408 8 -7.82      358 -351  933 315  u=13
10409 8 -7.82     -355 354  933 315  u=13
10410 8 -7.82     -355 -351  933 315  u=13
c
10411 8 -7.82      358 354  933 -310 u=13
10412 8 -7.82      358 -351  933 -310 u=13
10413 8 -7.82     -355 354  933 -310 u=13
10414 8 -7.82     -355 -351  933 -310 u=13
c  concrete and radial plates next to top ducts
10421 8 -7.82      310 -315 391 -392  933 -935 u=13
10422 8 -7.82      310 -315 393 -394  933 -935 u=13
c
10431 7 -2.35      310 -315 394 -355  933 -935 u=13
10432 7 -2.35      310 -315 358 -391  933 -935 u=13
10433 7 -2.35      310 -315 392  354  933 -935 u=13
10434 7 -2.35      310 -315 -351 394  933 -935 u=13
10435 7 -2.35      310 -315 358 -393  933 -935 u=13
10436 7 -2.35      310 -315 392 -355  933 -935 u=13
10437 7 -2.35      310 -315 -391 -351  933 -935 u=13
10438 7 -2.35      310 -315 354 -393  933 -935 u=13
c
c  air and grid spacers in top ducts
10441 9 -1.17e-3  352 -353  934 263 -264 u=13
10442 9 -1.17e-3  356 -357  934 261 -262 u=13
c
10443 9 -1.17e-3  352 -231  934 (-263:264) u=13
10444 9 -1.17e-3  236 -353  934 (-263:264) u=13
c
10445 9 -1.17e-3  231 -232  934 -251 (-263:264) u=13
10446 5 -7.92      232 -233  934 -251 (-263:264) u=13
10447 9 -1.17e-3  233 -234  934 -251 (-263:264) u=13
10448 5 -7.92      234 -235  934 -251 (-263:264) u=13
10449 9 -1.17e-3  235 -236  934 -251 (-263:264) u=13
10450 5 -7.92      231 -232  251 -252 (-263:264) u=13
10451 5 -7.92      232 -233  251 -252 (-263:264) u=13
10452 5 -7.92      233 -234  251 -252 (-263:264) u=13
10453 5 -7.92      234 -235  251 -252 (-263:264) u=13
10454 5 -7.92      235 -236  251 -252 (-263:264) u=13
10455 9 -1.17e-3  231 -232  252 -253 (-263:264) u=13
10456 5 -7.92      232 -233  252 -253 (-263:264) u=13
10457 9 -1.17e-3  233 -234  252 -253 (-263:264) u=13
10458 5 -7.92      234 -235  252 -253 (-263:264) u=13
10459 9 -1.17e-3  235 -236  252 -253 (-263:264) u=13
10470 9 -1.17e-3  231 -236  253      (-263:264) u=13
c
10543 9 -1.17e-3  356 -241  934 (-261:262) u=13
10544 9 -1.17e-3  246 -357  934 (-261:262) u=13
c
10545 9 -1.17e-3  241 -242  934 -251 (-261:262) u=13
10546 5 -7.92      242 -243  934 -251 (-261:262) u=13
10547 9 -1.17e-3  243 -244  934 -251 (-261:262) u=13
10548 5 -7.92      244 -245  934 -251 (-261:262) u=13
10549 9 -1.17e-3  245 -246  934 -251 (-261:262) u=13
10550 5 -7.92      241 -242  251 -252 (-261:262) u=13
10551 5 -7.92      242 -243  251 -252 (-261:262) u=13
10552 5 -7.92      243 -244  251 -252 (-261:262) u=13
10553 5 -7.92      244 -245  251 -252 (-261:262) u=13
10554 5 -7.92      245 -246  251 -252 (-261:262) u=13
10555 9 -1.17e-3  241 -242  252 -253 (-261:262) u=13
10556 5 -7.92      242 -243  252 -253 (-261:262) u=13
10557 9 -1.17e-3  243 -244  252 -253 (-261:262) u=13
10558 5 -7.92      244 -245  252 -253 (-261:262) u=13
10559 9 -1.17e-3  245 -246  252 -253 (-261:262) u=13
10570 9 -1.17e-3  241 -246  253      (-261:262) u=13
c  top plate

```

10641 8 -7.82 358 354 935 310 -315 u=13
10642 8 -7.82 358 -351 935 310 -315 u=13
10643 8 -7.82 -355 354 935 310 -315 u=13
10644 8 -7.82 -355 -351 935 310 -315 u=13

c
10701 8 -7.82 -314 u=15
10702 0 314 u=15

c
10711 7 -2.35 -312 u=14
10712 8 -7.82 312 -313 u=14
10713 0 313 u=14

c
99999 0 -817:918:717:(716 -816)

c
c BLANK LINE

c BLANK LINE

c MPC surfaces \ / \ / \ / \ /

c
10 px -12.169775
11 px -12.017375
12 px -11.826875
13 px -11.1125
14 px 11.1125
15 px 11.826875
16 px 12.017375
17 px 12.169775
18 py -12.169775
19 py -12.017375
20 py -11.826875
21 py -11.1125
22 py 11.1125
23 py 11.826875
24 py 12.017375
25 py 12.169775

c
26 py -9.525
27 py 9.525
28 px -9.525
29 px 9.525
30 py -6.35
31 py 6.35
32 px -6.35
33 px 6.35

c
35 px -11.46969
36 px 11.46969
37 py -11.46969
38 py 11.46969

c
40 px -10.8204
41 px 10.8204
42 py -10.8204
43 py 10.8204

c
101 py 82.12074
102 py 54.74716
103 py 27.37358
104 py 0.0
105 py -27.37358
106 py -54.74716
107 py -82.12074

c
116 px 82.12074

115	px	54.74716		
114	px	27.37358		
113	px	0.0		
112	px	-27.37358		
111	px	-54.74716		
110	px	-82.12074		
c				
301	cz	85.56625		
302	cz	86.83625		
c				
c	620	pz	21.59	\$ MPC baseplate - 2.5 inches
400	pz	23.876		\$ start of egg crate
410	pz	28.8925		\$ start of boral
415	pz	32.004		\$ begin fuel element
420	pz	50.7365		\$ end of lower nozzle
425	pz	53.2765		\$ end of space/ start of active fuel
430	pz	419.0365		\$ end of active fuel
435	pz	425.1325		\$ boral ends
440	pz	428.72025		\$ space above fuel
445	pz	439.83275		\$ plenum spacer ends
455	pz	452.6915		\$ top of top nozzle
460	pz	467.614		\$ top of basket
c				
610	pz	15.24		\$ overpack baseplate
615	pz	17.78		
616	pz	20.32		
620	pz	21.59		\$ MPC baseplate - 2.5 inches
675	pz	474.98		\$ bottom of MPC in lid - 178.5 inches from 620
651	pz	476.25		\$ 0.25 inch first segment
652	pz	478.79		
653	pz	481.33		
654	pz	483.87		
655	pz	486.41		
656	pz	488.95		
657	pz	491.49		
658	pz	494.03		
659	pz	496.57		
680	pz	499.11		\$ top of MPC outer lid
c				
c				MPC surfaces/\ / \ / \ / \ /
c				
c				overpack surfaces
c				
303	cz	80.01		\$ ID of item 27
304	cz	81.28		\$ OD of item 27
305	cz	85.09		\$ ID of item 7
306	cz	86.20125		\$ ID of item 5
307	cz	87.63		\$ OD of item 7
c				
310	cz	96.52		\$ outer rad of item 3 overpack inner shell
311	cz	98.425		\$ outer rad of item 28
312	cz	107.95		\$ ID of item 26
313	cz	109.22		\$ OD of item 26
314	cz	160.02		\$ OD of item 10
315	cz	166.37		\$ ID of item 2
c				top duct planes
351	px	-33.02		\$ start of item 12
352	px	-31.75		\$ end of item 12
353	px	31.75		\$ start of item 12
354	px	33.02		\$ end of item 12
c				
355	py	-33.02		\$ start of item 12
356	py	-31.75		\$ end of item 12
357	py	31.75		\$ start of item 12
358	py	33.02		\$ end of item 12


```

c      bottom duct planes
361 px  -20.955 $ start of item 13
362 px  -19.05  $ end of item 13
363 px   19.05  $ start of item 13
364 px   20.955 $ end of item 13
c
365 py  -20.955 $ start of item 13
366 py  -19.05  $ end of item 13
367 py   19.05  $ start of item 13
368 py   20.955 $ end of item 13
c      cutouts in item 1
371 px  -123.19
372 px   123.19
373 py  -123.19
374 py   123.19
c      item 14
391 1 py  -0.9525 $ steel plate in concrete at 45/225 degrees
392 1 py   0.9525 $ steel plate in concrete at 45/225 degrees
393 1 px  -0.9525 $ steel plate in concrete at 135/315 degrees
394 1 px   0.9525 $ steel plate in concrete at 135/315 degrees
c
c      bottom shielding cross plates
c
201 px  -18.57375
202 px   -6.35
203 px   -5.715
204 px    5.715
205 px    6.35
206 px   18.57375
c
211 py  -18.57375
212 py   -6.35
213 py   -5.715
214 py    5.715
215 py    6.35
216 py   18.57375
c
221 pz  -32.8168
222 pz  -32.1818
223 pz  -24.3586
224 pz  -23.7236
225 pz  -15.9004
c
c      top shielding cross plates
c
231 px  -31.27375
232 px  -10.795
233 px  -10.16
234 px   10.16
235 px   10.795
236 px   31.27375
c
241 py  -31.27375
242 py  -10.795
243 py  -10.16
244 py   10.16
245 py   10.795
246 py   31.27375
c
251 pz   523.24
252 pz   523.875
253 pz   530.86
c      end of cross plates in openings
261 px  -107.315
262 px   107.315

```

263 py -107.315
264 py 107.315
c end of part of bottom cross plates
271 px -124.46
272 px 124.46
273 py -124.46
274 py 124.46
c
c radial planes in overpack
700 cz 93.345 \$ ID of overpack
701 cz 95.885
702 cz 98.5 \$ slightly diff from 311
703 cz 103.505
704 cz 108.585
705 cz 113.665
706 cz 118.745
707 cz 123.825
708 cz 128.905
709 cz 133.985
710 cz 139.065
711 cz 144.145
712 cz 149.225
713 cz 154.305
714 cz 159.385
715 cz 164.465
716 cz 168.275
717 cz 169.275
c
c planes in pedestal
c
801 pz 12.7
802 pz 10.16
803 pz 7.62
804 pz 5.08
805 pz 2.54 \$ bottom of item 24
806 pz -2.54
807 pz -7.62
808 pz -12.7
809 pz -17.78
810 pz -22.86
811 pz -27.94
812 pz -33.02
813 pz -38.1
814 pz -40.64 \$ start of item 1
815 pz -43.18 \$
816 pz -45.72 \$ ground
817 pz -76.20
c
c planes in lid
c
901 pz 501.65 \$ start of item 6
902 pz 502.285 \$ 0.25 inch segment from start
903 pz 504.825 \$ end of item 6
904 pz 509.905
905 pz 513.715
906 pz 516.3 \$ end of item 8 plus a little
907 pz 521.335
908 pz 526.415
909 pz 531.495 \$ end of concrete start of item 10
910 pz 534.035
911 pz 536.575
912 pz 539.115
913 pz 541.655 \$ end of item 10
914 pz 546.735
915 pz 551.815

916	pz	556.895	
917	pz	561.975	
918	pz	562.975	
c			
c	planes in overpack		
c			
931	pz	-15.24	\$ bottom of item 11
932	pz	-10.16	\$ top of item 11
933	pz	513.08	\$ bottom of item 8 and top of item 28
934	pz	516.255	\$ top of item 8
935	pz	529.59	\$ start of item 9
c			
c	for tallying		
c			
501	pz	-45.72	
502	pz	-30.48	
503	pz	-15.24	
504	pz	0.00	
505	pz	15.24	
506	pz	30.48	
507	pz	45.72	
508	pz	60.96	
509	pz	76.20	
510	pz	91.44	
511	pz	106.68	
512	pz	121.92	
513	pz	137.16	
514	pz	152.40	
515	pz	167.64	
516	pz	182.88	
517	pz	198.12	
518	pz	213.36	
519	pz	228.60	
520	pz	243.84	
521	pz	259.08	
522	pz	274.32	
523	pz	289.56	
524	pz	304.80	
525	pz	320.04	
526	pz	335.28	
527	pz	350.52	
528	pz	365.76	
529	pz	381.00	
530	pz	396.24	
531	pz	411.48	
532	pz	426.72	
533	pz	441.96	
534	pz	457.20	
535	pz	472.44	
536	pz	487.68	
537	pz	502.92	
538	pz	518.16	
539	pz	533.40	
c			
550	cz	15.24	
551	cz	30.48	
552	cz	45.72	
553	cz	60.96	
554	cz	76.20	
555	cz	91.44	
556	cz	106.68	
557	cz	121.92	
558	cz	137.16	
559	cz	152.40	
560	cz	167.64	

```

c
c   BLANK LINE

c   BLANK LINE
c
*tr1  0 0 0 45  315  90 135  45  90 90 90 0
c
c   PHOTON MATERIALS
c
c   fuel 3.4 w/o U235  10.412 gm/cc
m1      92235.01p  -0.029971
        92238.01p  -0.851529
        8016.01p   -0.1185
c   homogenized fuel density 3.8699 gm/cc
m2      92235.01p  -0.027652
        92238.01p  -0.719715
        8016.01p   -0.100469
        40000.01p  -0.149015
        50000.01p  -0.002587
        26000.01p  -0.000365
        24000.01p  -0.000198
c   zirconium 6.55 gm/cc
m3      40000.01p  1.          $ Zr Clad
c   stainless steel 7.92 gm/cc
m5      24000.01p  -0.19
        25055.01p  -0.02
        26000.01p  -0.695
        28000.01p  -0.095
c   boral 2.644 gm/cc
m6      5010.01p   -0.044226
        5011.01p   -0.201474
        13027.01p  -0.6861
        6000.01p   -0.0682
c   Concrete (NBS Ordinary) @ 2.35 g/cc (Ref: LA-12827-M)
m7      14000.01p  -0.315
        13027.01p  -0.048
        8016.01p   -0.500
        1001.01p   -0.006
        11023.01p  -0.017
        20000.01p  -0.083
        26000.01p  -0.012
        19000.01p  -0.019
c   carbon steel 7.82 gm/cc
m8      6000.01p  -0.005 26000.01p -0.995
c   air density 1.17e-3 gm/cc
m9      7014.01p  0.78 8016.01p 0.22
c
c   NEUTRON MATERIALS
c
c   fuel 3.4 w/o U235  10.412 gm/cc
c   m1      92235.50c  -0.029971
        92238.50c  -0.851529
        8016.50c   -0.1185
c   c   homogenized fuel density 3.8699 gm/cc
c   m2      92235.50c  -0.027652
        92238.50c  -0.719715
        8016.50c   -0.100469
        40000.35c  -0.149015
        50000.35c  -0.002587
        26000.55c  -0.000365
        24000.50c  -0.000198
c   c   helium 1e-4 gm/cc
c   m3      2004.50c  1.0
c   c   stainless steel 7.92 gm/cc
c   m5      24000.50c  -0.19

```

```

c          25055.50c  -0.02
c          26000.55c  -0.695
c          28000.50c  -0.095
c    c    boron  2.644 gm/cc
c    m6          5010.50c  -0.044226
c          5011.56c  -0.201474
c          13027.50c  -0.6861
c          6000.50c  -0.0682
c    c          Concrete (NBS Ordinary) @ 2.35 g/cc (Ref: LA-12827-M)
c    m7          14000.50c  -0.315
c          13027.50c  -0.048
c          8016.50c  -0.500
c          1001.50c  -0.006
c          11023.50c  -0.017
c          20000.50c  -0.083
c          26000.55c  -0.012
c          19000.50c  -0.019
c    mt7         lwtr.01t
c    c          carbon steel 7.82 gm/cc
c    m8          6000.50c  -0.005 26000.55c  -0.995
c    c          air density 1.17e-3 gm/cc
c    m9          7014.50c 0.78 8016.50c 0.22
c
phys:n  20 0.0
phys:p  100 0
c    imp:n  1 228r 0
c    imp:p  1 228r 0
nps    13500000
prtmp  j  -60  1  2
c    print  10 110 160 161 20 170
print
mode p
ssw  716 917
c
sdef  par=2  erg=d1  axs=0 0 1  x=d4  y=fx  d5  z=d3
c
c    energy dist for gammas in the fuel
c
c    si1  h  0.7 1.0 1.5 2.0 2.5 3.0
c    spl  0  0.43 0.27 0.22 0.04 0.04
c
c    energy dist for neutrons in the fuel
c
c    si1  h  0.1 0.4 0.9 1.4 1.85 3.0 6.43 20.0
c    spl  0  0.03787 0.1935 0.1773 0.1310 0.2320 0.2098 0.01853
c
c    energy dist for Co60 gammas
c
c    si1  1 1.3325 1.1732
c    spl  0.5  0.5
c
c    axial dist for phot in fuel
c
c    si3  h  53.2765 68.5165 83.7565 114.2365 175.1965 236.1565
c          297.1165 358.0765 388.5565 403.7965 419.0365
c    sp3  0  0.022854 0.035321 0.08975 0.184167 0.183 0.179833
c          0.175017 0.080033 0.030575 0.019458
c    sb3  0 1 1 1 1 1 1 1 1 1
c
c    axial dist for Co60 - a zero prob is in the fuel
c
c    si3  h  32.004 50.7365 419.0365 428.72025 439.83275 452.6915
c    sp3  0  0.44 0.0 0.05 0.05 0.46
c    sb3  0  0.50 0.0 0.05 0.10 0.35
c

```

```

si4 s      13 14
          12 13 14 15
          11 12 13 14 15 16
          11 12 13 14 15 16
          12 13 14 15
          13 14

sp4 1 23r
c
ds5 s      26 26
          25 25 25 25
          24 24 24 24 24 24
          23 23 23 23 23 23
          22 22 22 22
          21 21

c
si11 -79.25435 -57.61355
si12 -51.88077 -30.23997
si13 -24.50719 -2.86639
si14  2.86639  24.50719
si15  30.23997  51.88077
si16  57.61355  79.25435

c
si21 -79.25435 -57.61355
si22 -51.88077 -30.23997
si23 -24.50719 -2.86639
si24  2.86639  24.50719
si25  30.23997  51.88077
si26  57.61355  79.25435

c
sp11 0 1
sp12 0 1
sp13 0 1
sp14 0 1
sp15 0 1
sp16 0 1
sp21 0 1
sp22 0 1
sp23 0 1
sp24 0 1
sp25 0 1
sp26 0 1

c
# imp:p
314 2
315 2
316 2
317 2
318 2
319 2
320 2
321 2
346 2
353 2
360 2
361 2
362 2
363 2
364 2
365 2
366 2
367 2
414 2
415 2
416 2
417 2

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418 2
419 2
420 2
421 2
446 2
453 2
460 2
461 2
462 2
463 2
464 2
465 2
466 2
467 2
514 2
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516 2
517 2
518 2
519 2
520 2
521 2
546 2
553 2
560 2
561 2
562 2
563 2
564 2
565 2
566 2
567 2
614 2
615 2
616 2
617 2
618 2
619 2
620 2
621 2
646 2
653 2
660 2
661 2
662 2
663 2
664 2
665 2
666 2
667 2
714 2
715 2
716 2
717 2
718 2
719 2
720 2
721 2
746 2
753 2
760 2
761 2
762 2
763 2
764 2

765 2
766 2
767 2
322 1
323 1
324 1
325 1
326 1
327 1
328 1
329 1
347 1
355 1
368 1
369 1
370 1
371 1
372 1
373 1
374 1
375 1
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424 1
425 1
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427 1
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429 1
447 1
455 1
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469 1
470 1
471 1
472 1
473 1
474 1
475 1
522 1
523 1
524 1
525 1
526 1
527 1
528 1
529 1
547 1
555 1
568 1
569 1
570 1
571 1
572 1
573 1
574 1
575 1
622 1
623 1
624 1
625 1
626 1
627 1
628 1
629 1

647 1
655 1
668 1
669 1
670 1
671 1
672 1
673 1
674 1
675 1
722 1
723 1
724 1
725 1
726 1
727 1
728 1
729 1
747 1
755 1
768 1
769 1
770 1
771 1
772 1
773 1
774 1
775 1
330 2
331 2
332 2
333 2
334 2
335 2
336 2
337 2
348 2
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357 2
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378 2
379 2
380 2
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431 2
432 2
433 2
434 2
435 2
436 2
437 2
448 2
456 2
457 2
476 2
477 2
478 2
479 2
480 2
481 2
482 2

483	2
530	2
531	2
532	2
533	2
534	2
535	2
536	2
537	2
548	2
556	2
557	2
576	2
577	2
578	2
579	2
580	2
581	2
582	2
583	2
630	2
631	2
632	2
633	2
634	2
635	2
636	2
637	2
648	2
656	2
657	2
676	2
677	2
678	2
679	2
680	2
681	2
682	2
683	2
730	2
731	2
732	2
733	2
734	2
735	2
736	2
737	2
748	2
756	2
757	2
776	2
777	2
778	2
779	2
780	2
781	2
782	2
783	2
349	4
358	4
384	4
385	4
386	4
387	4
388	4

389	4
390	4
391	4
449	4
458	4
484	4
485	4
486	4
487	4
488	4
489	4
490	4
491	4
549	4
558	4
584	4
585	4
586	4
587	4
588	4
589	4
590	4
591	4
649	4
658	4
684	4
685	4
686	4
687	4
688	4
689	4
690	4
691	4
749	4
758	4
784	4
785	4
786	4
787	4
788	4
789	4
790	4
791	4
351	8
451	8
551	8
651	8
751	8
1051	16
1052	8
1053	4
1060	24
1061	24
1062	72
1063	72
1064	216
1065	216
1066	648
1067	648
1068	1944
1069	1944
1001	16
1003	8
1005	8
1007	4

1009	2
1011	4
1013	8
1014	16
1015	48
1017	24
1019	72
1021	72
1023	216
1025	216
1027	648
1028	648
1029	1944
1031	1944
c	
2001	32
2002	64
2003	128
2004	256
2005	512
2006	1024
2007	1024
2008	2048
2009	2048
2010	4096
2011	4096
2012	8192
2013	8192
2014	16384
c	
2016	1024
2017	1024
2028	2048
2019	2048
2020	4096
2021	4096
2022	8192
2023	8192
2024	16384
c	
2031	16384
2032	16384
2033	16384
c	
3001	3888
3002	3888
3003	3888
c	
3004	7776
3005	7776
3006	7776
3007	7776
3008	15552
3009	15552
c	
3010	7776
3011	7776
3012	7776
3013	7776
3014	15552
3015	15552
c	
3021	46656
3022	46656
3023	139968

3024	139968
C	
3030	1
3031	1
3032	1
3033	1
3034	1
C	
3035	279936
3036	279936
3037	559872
3038	559872
3039	1
C	
3040	279936
3041	279936
3042	559872
3043	559872
3044	1
C	
4000	32768
4001	32768
4002	32768
4003	32768
4004	16384
4005	16384
4006	8192
4007	8192
4008	4096
4009	4096
4010	2048
4011	2048
4012	1024
4013	512
4014	256
4015	128
4016	64
4017	32
4018	16
4019	8
4020	4
4021	8
4022	16
4023	32
4024	96
4025	48
4026	144
4027	144
4028	432
4029	432
4030	1296
4031	1296
4032	3888
4033	3888
4034	7776
4035	7776
4036	7776
4037	15552
4038	15552
4039	15552
4040	15552
4041	31104
4042	31104
4043	93312
4044	93312

4045	279936
4046	279936
4047	559872
4048	559872
4049	1119744
4050	1119744
4051	1
c	
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5003	32768
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5005	16384
5006	8192
5007	8192
5008	4096
5009	4096
5010	2048
5011	2048
5012	1024
5013	512
5014	256
5015	128
5016	64
5017	32
5018	16
5019	8
5020	4
5021	8
5022	16
5023	32
5024	96
5025	48
5026	144
5027	144
5028	432
5029	432
5030	1296
5031	1296
5032	3888
5033	3888
5034	7776
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5036	7776
5037	15552
5038	15552
5039	15552
5040	15552
5041	31104
5042	31104
5043	93312
5044	93312
5045	279936
5046	279936
5047	559872
5048	559872
5049	1119744
5050	1119744
5051	1
c	
5100	65536
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5102	65536
5103	65536

5104	32768
5105	32768
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5107	16384
5108	8192
5109	8192
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5112	2048
5113	1024
5114	512
5115	256
5116	128
5117	64
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5120	8
5121	16
5122	32
5123	64
5124	192
5125	96
5126	288
5127	288
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5129	864
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5131	2592
5132	7776
5133	7776
5134	15552
5135	15552
5136	15552
5137	31104
5138	31104
5139	31104
5140	31104
5141	62208
5142	62208
5143	186624
5144	186624
5145	559872
5146	559872
5147	1119744
5148	1119744
5149	2239488
5150	2239488
5151	1
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5200	131072
5201	131072
5202	131072
5203	131072
5204	65536
5205	65536
5206	32768
5207	32768
5208	16384
5209	16384
5210	8192
5211	8192
5212	4096
5213	2048
5214	1024
5215	512

5216	256
5217	128
5218	64
5219	32
5220	16
5221	32
5222	64
5223	128
5224	384
5225	192
5226	576
5227	576
5228	1728
5229	1728
5230	5184
5231	5184
5232	15552
5233	15552
5234	31104
5235	31104
5236	31104
5237	62208
5238	62208
5239	62208
5240	62208
5241	124416
5242	124416
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5244	373248
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5246	1119744
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5248	2239488
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5250	4478976
5251	1
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5303	262144
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5307	65536
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5321	64
5322	128
5323	256
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5327	1152

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5521 128
5522 256
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5716	2048
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c	
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5914	16384
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5919	512
5920	256
5921	512
5922	1024
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6118	2048
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10712 1
10713 1
99999 0
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c
c      neutron dose factors
c
c      2.5e-8  1.0e-7  1.0e-6  1.0e-5  1.0e-4  1.0e-3  1.0e-2  0.1
c      0.5    1.0    2.5    5.0    7.0    10.0   14.0   20.0
c      3.67e-6 3.67e-6 4.46e-6 4.54e-6 4.18e-6 3.76e-6 3.56e-6 2.17e-5
c      9.26e-5 1.32e-4 1.25e-4 1.56e-4 1.47e-4 1.47e-4 2.08e-4 2.27e-4
c
c      photon dose factors
c
c      0.01 0.03 0.05 0.07 0.1 0.15 0.2 0.25 0.3 0.35 0.4 0.45
c      0.5 0.55 0.6 0.65 0.7 0.8 1.0 1.4 1.8 2.2 2.6 2.8 3.25
c      3.75 4.25 4.75 5.0 5.25 5.75 6.25 6.75 7.5 9.0 11.0
c      13.0 15.0
c      3.96e-06 5.82e-07 2.90e-07 2.58e-07 2.83e-07 3.79e-07 5.01e-07
c      6.31e-07 7.59e-07 8.78e-07 9.85e-07 1.08e-06 1.17e-06 1.27e-06
c      1.36e-06 1.44e-06 1.52e-06 1.68e-06 1.98e-06 2.51e-06 2.99e-06
c      3.42e-06 3.82e-06 4.01e-06 4.41e-06 4.83e-06 5.23e-06 5.60e-06
c      5.80e-06 6.01e-06 6.37e-06 6.74e-06 7.11e-06 7.66e-06 8.77e-06
c      1.03e-05 1.18e-05 1.33e-05
c
c
c      PHOTON TALLIES
c
c      f102:p 716 917
c      ft102  scx 3
c      de102  0.01 0.03 0.05 0.07 0.1 0.15 0.2 0.25 0.3 0.35 0.4 0.45
c      0.5 0.55 0.6 0.65 0.7 0.8 1.0 1.4 1.8 2.2 2.6 2.8 3.25
c      3.75 4.25 4.75 5.0 5.25 5.75 6.25 6.75 7.5 9.0 11.0
c      13.0 15.0
c      df102  3.96e-06 5.82e-07 2.90e-07 2.58e-07 2.83e-07 3.79e-07 5.01e-07
c      6.31e-07 7.59e-07 8.78e-07 9.85e-07 1.08e-06 1.17e-06 1.27e-06
c      1.36e-06 1.44e-06 1.52e-06 1.68e-06 1.98e-06 2.51e-06 2.99e-06
c      3.42e-06 3.82e-06 4.01e-06 4.41e-06 4.83e-06 5.23e-06 5.60e-06
c      5.80e-06 6.01e-06 6.37e-06 6.74e-06 7.11e-06 7.66e-06 8.77e-06
c      1.03e-05 1.18e-05 1.33e-05
c      fq102  u s
c

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APPENDIX 5.D

DOSE RATE COMPARISON FOR DIFFERENT COBALT IMPURITY LEVELS

(Total number of pages in this appendix : 6)

The dose rate adjacent to and one meter from the 100-ton HI-TRAC and the HI-STORM overpack are presented on Tables 5.D.1 through 5.D.4 for the MPC-24 with different burnup and cooling times and different assumed Cobalt-59 impurity levels for inconel. The HI-TRAC results were calculated for an earlier design which utilized 30 steel fins 0.375 inches thick compared to 10 steel fins 1.25 inches thick. The change in rib design only affects the magnitude of the dose rates presented for the radial surface but does not affect the conclusions discussed below. The following burnup and cooling time combinations are presented.

100-ton HI-TRAC

- 35,000 MWD/MTU and 5 year cooling
1000 ppm (1.0 gm/kg) Cobalt-59 impurity in inconel
- 45,000 MWD/MTU and 9 year cooling
4700 ppm (4.7 gm/kg) Cobalt-59 impurity in inconel

HI-STORM

- 45,000 MWD/MTU and 5 year cooling
1000 ppm (1.0 gm/kg) Cobalt-59 impurity in inconel
- 45,000 MWD/MTU and 9 year cooling
4700 ppm (4.7 gm/kg) Cobalt-59 impurity in inconel

On Tables 5.D.1 through 5.D.4, the contribution to the dose rate from activation in incore grid spacers is explicitly shown.

These results demonstrate that the dose rates at the longer cooling time are essentially equivalent to (within 11%) or bounded by the dose rates at the shorter cooling times even though a very conservative Cobalt-59 impurity level of 4700 ppm was assumed for the longer cooling times.

Table 5.2.1 shows the masses of inconel and steel that are used in the modeling of the PWR fuel assembly. When 4700 ppm was used for the impurity level in the inconel, an effective Cobalt-59 impurity level was used for the regions containing both steel and inconel. The following table summarizes the impurity levels that were used.

Region	Regional Co-59 impurity when 1000 ppm in inconel assumed	Regional Co-59 impurity when 4700 ppm in inconel assumed
Lower end fitting	1000 ppm	1340 ppm
Incore grid spacers	1000 ppm	4700 ppm
Gas plenum springs	1000 ppm	3417 ppm
Gas plenum spacer	1000 ppm	3417 ppm
Upper end fitting	1000 ppm	1000 ppm

Table 5.D.1

DOSE RATES ADJACENT TO 100-TON HI-TRAC FOR NORMAL CONDITIONS
MPC-24 DESIGN BASIS ZIRCALOY CLAD FUEL

Dose Point † Location	Incore Grid Spacer ⁶⁰ Co Gammas (mrem/hr)	Fuel Gammas (mrem/hr)	(n,γ) Gammas (mrem/hr)	⁶⁰ Co Gammas (mrem/hr)	Neutrons (mrem/hr)	Totals (mrem/hr)
4700 ppm Co-59 in inconel						
45,000 MWD/MTU AND 9-YEAR COOLING						
1	12.20	11.89	12.65	507.16	171.98	715.87
2	345.15	332.90	52.81	0.75	88.26	819.87
3	4.72	5.19	1.98	369.24	214.70	595.83
4	7.13	8.11	0.97	197.76	180.55	394.52
5 (pool lid)	48.63	54.68	17.52	2557.68	1194.17	3872.68
5 (transfer lid)	137.52	155.17	0.97	3811.45	674.42	4779.53
1000 ppm Co-59 in inconel						
35,000 MWD/MTU AND 5-YEAR COOLING						
1	3.73	27.05	6.51	543.06	88.57	668.92
2	105.37	696.56	27.19	0.80	45.46	875.38
3	1.44	11.44	1.02	473.51	110.57	597.98
4	2.18	17.87	0.50	241.22	92.97	354.74
5 (pool lid)	28.09	186.49	8.27	2751.19	554.51	3528.55
5 (transfer lid)	41.98	293.57	0.50	4081.28	347.33	4764.66

† Refer to Figure 5.1.4.

Table 5.D.2

DOSE RATES AT 1 METER FROM 100-TON HI-TRAC FOR NORMAL CONDITIONS
MPC-24 DESIGN BASIS ZIRCALOY CLAD FUEL

Dose Point † Location	Incore Grid Spacer ⁶⁰ Co Gammas (mrem/hr)	Fuel Gammas (mrem/hr)	(n,γ) Gammas (mrem/hr)	⁶⁰ Co Gammas (mrem/hr)	Neutrons (mrem/hr)	Totals (mrem/hr)
4700 ppm Co-59 in inconel						
45,000 MWD/MTU AND 9-YEAR COOLING						
1	44.32	43.03	7.14	73.99	27.98	196.46
2	148.49	143.94	16.40	6.18	32.38	347.39
3	18.76	18.25	3.88	72.75	13.83	127.47
4	2.19	2.80	0.17	61.70	44.87	111.73
5(transfer lid)	55.57	64.25	0.18	1556.99	188.18	1865.16
1000 ppm Co-59 in inconel						
35,000 MWD/MTU AND 5-YEAR COOLING						
1	13.53	91.20	3.68	79.16	14.41	201.97
2	45.33	302.99	8.44	6.13	16.68	379.57
3	5.73	38.74	2.00	71.95	7.12	125.54
4	0.67	6.21	0.09	74.47	23.11	104.55
5(transfer lid)	16.96	128.14	0.09	1667.22	96.91	1909.32

† Refer to Figure 5.1.4.

Table 5.D.3

DOSE RATES ADJACENT TO HI-STORM OVERPACK FOR NORMAL CONDITIONS
MPC-24 DESIGN BASIS ZIRCALOY CLAD FUEL

Dose Point † Location	Incore Grid Spacer ⁶⁰ Co Gammas (mrem/hr)	Fuel Gammas †† (mrem/hr)	⁶⁰ Co Gammas (mrem/hr)	Neutrons (mrem/hr)	Totals (mrem/hr)
4700 ppm Co-59 in inconel					
45,000 MWD/MTU AND 9-YEAR COOLING					
1	0.54	2.95	3.86	2.37	9.72
2	7.73	10.40	0.03	1.48	19.63
3	0.36	1.97	2.59	1.18	6.11
4	0.10	0.53	0.33	3.10	4.06
1000 ppm Co-59 in inconel					
45,000 MWD/MTU AND 5-YEAR COOLING					
1	0.20	5.68	4.87	2.76	13.51
2	2.73	28.93	0.03	1.88	33.58
3	3.87	0.13	3.21	1.38	8.59
4	0.04	0.91	0.36	3.60	4.91

† Refer to Figures 5.1.1.

†† Gammas generated by neutron capture are included with fuel gammas.

Table 5.D.4

DOSE RATES ONE METER FROM HI-STORM OVERPACK FOR NORMAL CONDITIONS
MPC-24 DESIGN BASIS ZIRCALOY CLAD FUEL

Dose Point † Location	Incore Grid Spacer ⁶⁰ Co Gammas (mrem/hr)	Fuel Gammas †† (mrem/hr)	⁶⁰ Co Gammas (mrem/hr)	Neutrons (mrem/hr)	Totals (mrem/hr)
4700 ppm Co-59 in inconel					
45,000 MWD/MTU AND 9-YEAR COOLING					
1	0.77	2.01	2.30	0.46	5.54
2	3.90	5.14	0.39	0.64	10.08
3	0.44	1.09	1.72	0.18	3.42
4	0.05	0.23	0.14	0.94	1.37
1000 ppm Co-59 in inconel					
45,000 MWD/MTU AND 5-YEAR COOLING					
1	0.28	4.49	2.90	0.54	8.21
2	1.41	14.98	0.25	0.78	17.42
3	0.16	2.57	2.09	0.21	5.03
4	0.02	0.42	0.16	1.10	1.70

† Refer to Figures 5.1.1.

†† Gammas generated by neutron capture are included with fuel gammas.

APPENDIX 5.E

Dose Rates for a HI-STORM 100 Overpack With and Without an Inner Shield Shell

(Total number of pages in this appendix : 4)

In June 2001, the inner shield shell of the HI-STORM 100 overpack was removed. As a compensating change, the density of the concrete in the body of the overpack was increased to 155 lb/cuft as discussed in Section 5.3. This appendix presents a comparison of the dose rates calculated for a HI-STORM 100 overpack with and without an inner shield shell. The MPC-24 was used in this analysis. Table 5.E.1 presents the results for the overpack containing the inner shield shell and Table 5.E.2 presents the results for the overpack without the inner shield shell and the higher density concrete in the body of the overpack.

The results indicate that the change in shielding configuration does not significantly impact the dose rates. The dose rates for the surface of the ducts show a slight increase (7%) when the inner shield shell is removed while the midplane surface shows an even smaller increase (2%). The dose rates for the top of the overpack are reduced when the inner shield shell is removed and the concrete density is increased. All one meter locations are essentially identical.

Therefore, based on the results presented in this appendix, the analysis in the main body of the chapter uses the HI-STORM 100 overpack with the inner shield present.

Table 5.E.1

DOSE RATES FOR THE HI-STORM 100 OVERPACK FOR NORMAL CONDITIONS
 MPC-24 DESIGN BASIS ZIRCALOY CLAD FUEL AT BOUNDING
 BURNUP AND COOLING TIME
 45,000 MWD/MTU AND 5-YEAR COOLING
 INNER SHIELD SHELL IS PRESENT

Dose Point [†] Location	Fuel Gammas ^{††} (mrem/hr)	⁶⁰ Co Gammas (mrem/hr)	Neutrons (mrem/hr)	Totals (mrem/hr)
Surface				
1	5.88	4.87	2.76	13.51
2	31.67	0.03	1.88	33.58
3	4.00	3.21	1.38	8.59
4	0.95	0.36	3.60	4.91
One Meter				
1	4.77	2.90	0.54	8.21
2	16.39	0.25	0.78	17.42
3	2.73	2.09	0.21	5.03
4	0.44	0.16	1.10	1.70

[†] Refer to Figure 5.1.1.

^{††} Gammas generated by neutron capture are included with fuel gammas.

Table 5.E.2

DOSE RATES FOR THE HI-STORM 100 OVERPACK FOR NORMAL CONDITIONS
MPC-24 DESIGN BASIS ZIRCALOY CLAD FUEL AT BOUNDING
BURNUP AND COOLING TIME
45,000 MWD/MTU AND 5-YEAR COOLING
INNER SHIELD SHELL IS REMOVED

Dose Point [†] Location	Fuel Gammas ^{††} (mrem/hr)	⁶⁰ Co Gammas (mrem/hr)	Neutrons (mrem/hr)	Totals (mrem/hr)
Surface				
1	6.48	5.67	2.28	14.43
2	32.37	0.05	1.62	34.04
3	4.23	3.67	1.24	9.14
4	0.88	0.33	3.36	4.56
One Meter				
1	4.70	3.33	0.36	8.39
2	16.70	0.30	0.69	17.69
3	2.80	1.94	0.25	4.99
4	0.40	0.18	0.94	1.51

[†] Refer to Figure 5.1.1.

^{††} Gammas generated by neutron capture are included with fuel gammas.