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## 3.7 PLANT SYSTEMS

## 3.7.16 Spent Fuel Pool Boron Concentration

LCO 3.7.16

The spent fuel pool boron concentration shall be  $\geq$  2000 ppm.

APPLICABILITY: When fuel assemblies are stored in the spent fuel pool.

## **ACTIONS**

	CONDITION		REQUIRED ACTION	COMPLETION TIME
A.	Spent fuel pool boron concentration not within limit.	LCO 3.0.3 is not applicable.		
		A 1	Suspend movement of fuel assemblies in the spent fuel pool.	Immediately
		AND		
		A.2	Initiate action to restore spent fuel pool boron concentration to within limit.	Immediately

## SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.7.16.1	Verify the spent fuel pool boron concentration is within limit.	7 days

## 3.7 PLANT SYSTEMS

## 3.7.17 Spent Fuel Assembly Storage

LCO 3.7.17

Fuel assembly storage in the spent fuel pool shall be maintained such that any four cells shall be in a configuration as shown in Figure 3.7.17-1.

APPLICABILITY:

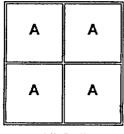
Whenever any fuel assembly is stored in the spent fuel pool.

## ACTIONS

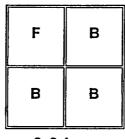
	CONDITION		REQUIRED ACTION	COMPLETION TIME
A.	Requirements of the LCO not met.	A.1	Initiate action to move the noncomplying fuel assembly into an acceptable storage location.	Immediately

## SURVEILLANCE REQUIREMENTS

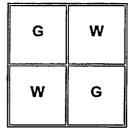
	SURVEILLANCE	FREQUENCY
SR 3.7.17.1	Verify by administrative means that the fuel assembly characteristics and its expected storage location is in accordance with LCO 3.7.17.	Prior to each fuel assembly move, when the assembly will be stored in the spent fuel pool.







2x2 Array Configuration



Checkerboard Configuration

## All Cell:

A Fuel assembly with a discharge burnup in the "acceptable" region of Figure 3.7.17-2.

## 2x2 Array:

- F (a) Fuel assembly with an initial enrichment ≤ 4.9 wt% U-235; or
  - (b) Fuel assembly with an initial enrichment ≤ 5.0 wt% U-235 and an IFBA loading equivalent to 16 rods each with 1.5 mg <sup>10</sup>B/in over 120 inches.
- B Fuel assembly with a discharge burnup in the "acceptable" region of Figure 3.7.17-3.

## **Checkerboard:**

- G Fuel assembly with an initial enrichment ≤ 5.0 wt% U-235.
- W Water cell locations where fuel assemblies are not present, non-fissile components are permitted.

FIGURE 3.7.17-1
ALLOWABLE STORAGE CONFIGURATIONS
(ALL CELL, 2X2 ARRAY, CHECKERBOARD)
FOR THE SPENT FUEL POOL

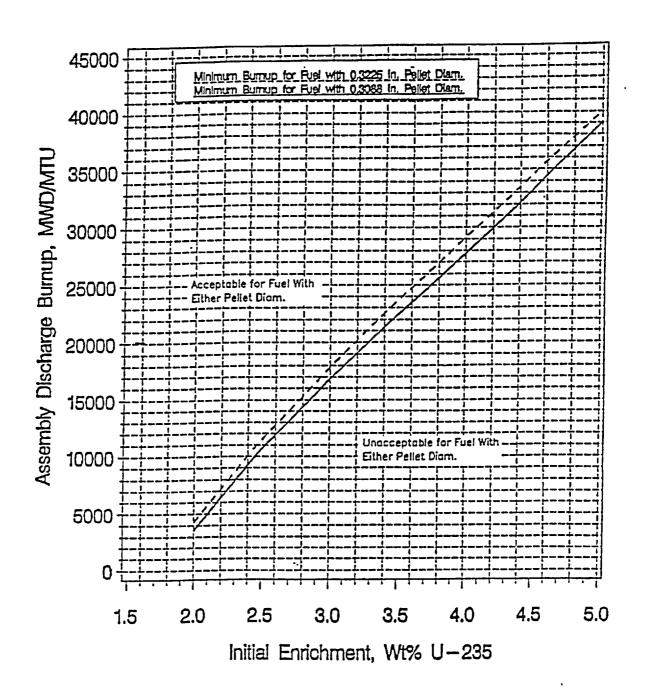


FIGURE 3.7.17-2
MINIMUM REQUIRED ASSEMBLY DISCHARGE BURNUP
AS A FUNCTION OF INITIAL ENRICHMENT AND FUEL PELLET DIAMETER
FOR AN ALL CELL STORAGE CONFIGURATION'

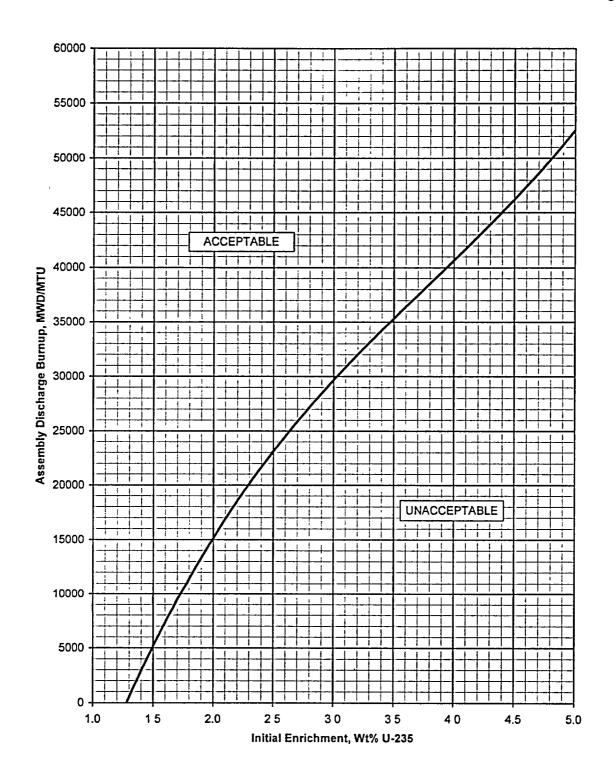


FIGURE 3.7.17-3
MINIMUM REQUIRED ASSEMBLY DISCHARGE BURNUP
AS A FUNCTION OF INITIAL ENRICHMENT
FOR A 2X2 ARRAY STORAGE CONFIGURATION

Spent Fuel Assembly Storage 3.7.17

THIS PAGE NOT USED

#### 4.0 DESIGN FEATURES

#### 4.1 Site Location

The DCPP site consists of approximately 750 acres which are adjacent to the Pacific Ocean in San Luis Obispo County, California, and is approximately twelve (12) miles west-southwest of the city of San Luis Obispo.

#### 4.2 Reactor Core

#### 4.2.1 Fuel Assemblies

The reactor shall contain 193 fuel assemblies. Each assembly shall consist of a matrix of Zircalloy or ZIRLO clad fuel rods with an initial composition of natural or slightly enriched uranium dioxide (UO<sub>2</sub>) as fuel material. Limited substitutions of zirconium alloy or stainless steel filler rods for fuel rods, in accordance with approved applications of fuel rod configurations, may be used. Fuel assemblies shall be limited to those fuel designs that have been analyzed with applicable NRC staff approved codes and methods and shown by tests or analyses to comply with all fuel safety design bases. A limited number of lead test assemblies that have not completed representative testing may be placed in nonlimiting core locations.

### 4.2.2 Control Rod Assemblies

The reactor core shall contain 53 control rod assemblies. The control rod material shall be silver, indium, and cadmium, as approved by the NRC.

#### 4.3 Fuel Storage

#### 4.3.1 Criticality

- 4.3.1.1 The spent fuel storage racks are designed and shall be maintained with:
  - a. Fuel assemblies having a maximum U-235 enrichment of 5.0 weight percent;
  - k<sub>eff</sub> < 1.0 if fully flooded with unborated water, which includes an allowance for uncertainties as described in Section 9.1.2.3 of the FSAR:
  - c.  $k_{eff} \le 0.95$  if fully flooded with water borated to 806 ppm, which includes an allowance for uncertainties as described in Section 9.1.2.3 of the FSAR;
  - d. A nominal 11 inch center to center distance between fuel assemblies placed in the fuel storage racks;

(continued)

#### 4.0 DESIGN FEATURES

## 4.3 Fuel Storage (continued)

- e. Fuel assemblies with a discharge burnup in the "acceptable" region of Figure 3.7.17-2 for the all cell configuration as shown in Figure 3.7.17-1;
- f. Fuel assemblies with a discharge burnup in the "acceptable" region of Figure 3.7.17-3 for the 2x2 array configuration as shown in Figure 3.7.17-1.

# 4.3.1.2 The new fuel storage racks are designed and shall be maintained with:

- a. Fuel assemblies having a maximum U-235 enrichment of 5.0 weight percent;
- b.  $k_{\text{eff}} \le 0.95$  if fully flooded with unborated water, which includes an allowance for uncertainties as described in Section 9.1.1.1 of the FSAR;
- c.  $k_{\text{eff}} \le 0.98$  if moderated by aqueous foam, which includes an allowance for uncertainties as described in Section 9.1.1.1 of the FSAR; and
- d. A nominal 22 inch center to center distance between fuel assemblies placed in the storage racks.

## 4.3.2 Drainage

The spent fuel storage pools are designed and shall be maintained to prevent inadvertent draining of the pool below elevation 133 ft.

#### 4.3.3 Capacity

The spent fuel storage pool is designed and shall be maintained with a storage capacity limited to no more than 1324 fuel assemblies.