

September 23, 2002

MEMORANDUM TO: James W. Andersen, Acting Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

FROM: Richard B. Ennis, Sr. Project Manager, Section 2 */RA/*
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

SUBJECT: MILLSTONE POWER STATION, UNIT NO. 2,
FACSIMILE TRANSMISSION, ISSUES TO BE DISCUSSED IN AN
UPCOMING CONFERENCE CALL (TAC NO. MB5799)

The attached information was transmitted by facsimile on September 19, 2002, to Mr. Ravi Joshi of Dominion Nuclear Connecticut, Inc. (the licensee). This information was transmitted to facilitate a upcoming conference call in order to clarify the licensee's amendment request dated August 1, 2002. The proposed amendment would revise TS 3.7.1.1, "Plant Systems: Turbine Cycle Safety Valves," to reflect results of a reanalysis of overpressurization events to reinstate the capability to operate, at corresponding reduced power levels, with up to four main steam line code safety valves in each main steam line inoperable. This memorandum and the attachment do not convey a formal request for information or represent an NRC staff position.

Docket No. 50-336

Attachment: Issues for Discussion in Upcoming Telephone Conference

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ISSUES FOR DISCUSSION IN UPCOMING TELEPHONE CONFERENCE

REGARDING PROPOSED AMENDMENT TO TECHNICAL SPECIFICATIONS
OPERATION WITH MAIN STEAM LINE CODE SAFETY VALVES INOPERABLE

MILLSTONE POWER STATION, UNIT NO. 2

DOCKET NO 50-336

By letter dated August 1, 2002, Dominion Nuclear Connecticut, Inc. (the licensee) submitted a proposed amendment to the Technical Specifications (TSs) for Millstone Power Station, Unit No. 2 (MP2). The proposed amendment would revise TS 3.7.1.1, "Plant Systems: Turbine Cycle Safety Valves," to reflect results of a reanalysis of overpressurization events to reinstate the capability to operate, at corresponding reduced power levels, with up to four main steam line code safety valves (MSSVs) in each main steam line inoperable.

The Nuclear Regulatory Commission (NRC) staff has reviewed the information the licensee provided that supports the proposed TS changes and would like to discuss the following issues to clarify the submittal dated August 1, 2002:

- 1) Was the reanalysis performed using computer codes and methods that were approved by the NRC staff?
- 2) Which computer code(s) and methodology was used for the reanalysis?
- 3) Discuss your method of analysis to determine the maximum power levels and corresponding high trip setpoints for the values shown in proposed TS Table 3.7.1.

Attachment