

August 13, 2002

Mr. Eugene S. Grecheck
Dominion Energy, Inc.
5000 Dominion Boulevard
Glen Allen, VA 23060

Dear Mr. Grecheck:

I would like to confirm our invitation to you to participate in the upcoming Nuclear Regulatory Commission's Nuclear Safety Research Conference (NSRC), formerly known as the Water Reactor Safety Meeting. The NSRC is an international conference focused on regulatory issues and attracts researchers, regulators, and utility representatives from the United States and more than twenty other countries. This conference continues to be a leading forum in which participants are provided an opportunity to interact with NRC staff and colleagues to obtain research results and insights from research programs performed in support of the mission of the NRC. The conference will be held October 28-30, 2002 at the Marriot at Metro Center, 775 12th Street NW, Washington, DC.

This year, as one of our program activities, we are planning to have a panel discussion, composed of U.S. and international experts to discuss the regulatory research needed to support the licensing of advanced reactor designs and focus on the kind of research that is needed to resolve technical/policy issues with our external stakeholders. I believe that your organization's activities and views would be of interest to our audience and therefore, I would like to invite you to participate on this panel. The 2-hour panel discussion is scheduled for Monday, October 28, 2002, from 9:30 to 11:30am. The panel will be chaired by Commissioner Jeffrey S. Merrifield. Each panel member will be requested to give a brief (8-10 min) perspective on the following questions:

1. What are the best methods or primary means to demonstrate enhanced margins of safety for advanced reactor designs?
2. Should goals for advanced reactor designs be expanded to capture the expectations that advanced reactors provide enhanced margins of safety?
3. How should the concept of defense-in-depth be defined for advanced reactors? Specifically, should the traditional deterministic approach, probabilistic approach, or a combination of the two be implemented to specify specific design goals and attributes for future reactors?
4. In consideration of current PRA technology and data limitations on advanced designs, to what extent can a probabilistic approach be used to establish the licensing basis for advanced reactors?

This year's conference will also include a panel on risk informed initiatives, and technical review sessions on degradation of reactor coolant pressure boundary materials, fuels, formal decision methods and nuclear safety research, dry cask storage and transportation of spent nuclear fuels, control of slightly contaminated materials, and probabilistic risk assessment research. We expect the attendees, as in the past, will be from the NRC, national laboratories, private research firms, reactor vendors, utilities, universities, and many international organizations. The preliminary agenda is enclosed.

We are planning an exciting program this year and I hope that you can join us in October. I look forward to hearing from you soon.

Sincerely,

/RA/

Ashok C. Thadani, Director
Office of Nuclear Regulatory Research

Enclosure: 2002 NSRC Preliminary Agenda

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