

Facility: Millstone Unit 3			Date of Exam: Week of 8/12/02						Exam Level: SRO				
Tier	Group	K/A Category Points											Point Total
		K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	
1. Emergency & Abnormal Plant Evolutions	1	9	4	1				0	4			6	24
	2	5	0	4				1	3			3	16
	3	2	0	0				1	0			0	3
	Tier Totals	16	4	5				2	7			9	43
2. Plant Systems	1	4	1	2	0	1	2	0	2	1	2	4	19
	2	0	1	1	1	1	0	2	3	1	3	4	17
	3	0	1	0	1	1	0	0	0	0	1	0	4
	Tier Totals	4	3	3	2	3	2	2	5	2	6	8	40
3. Generic Knowledge and Abilities					Cat 1	Cat 2	Cat 3	Cat 4					
					6	4	2	5	17				
<p>Note: 1. Ensure that at least two topics from every K/A category are sampled within each tier (i. e., the "Tier Totals" in each K/A category shall not be less than two).</p> <p>2. The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate by <math>\pm 1</math> from that specified in the table based on NRC revisions. The final exam must total 100 points.</p> <p>3. Select topics from many systems; avoid selecting more than two or three K/A topics from a given system unless they relate to plant-specific priorities.</p> <p>4. Systems/evolutions within each group are identified on the associated outline.</p> <p>5. The shaded areas are not applicable to the category/tier.</p> <p>6.* The generic K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system.</p> <p>7. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings for the SRO license level, and the point totals for each system and category. K/As below 2.5 should be justified on the basis of plant-specific priorities. Enter the tier totals for each category in the table above.</p>													

ES-401

PWR SRO Examination Outline  
Emergency and Abnormal Plant Evolutions - Tier 1/Group 1

Form ES-401-3

E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	Imp.	Points
000001 Continuous Rod Withdrawal / 1			02				Reasons for Tech Spec limits on Rod Operability	4.3	1
000003 Dropped Control Rod / 1	19						Operational implications of differential rod worth	2.9	1
000005 Inoperable/Stuck Control Rod / 1		02					Interrelation between rod, disconnects, control rm switches	2.6	1
000011 Large Break LOCA / 3	01						Operational implications of natural circulation cooling	4.4	1
W/E04 LOCA Outside Containment / 3					01		Selection of appropriate procedures during emergencies	4.3	1
W/E01 Rediagnosis / 3	03						Operational implications of indications and remedial actions	3.5	1
000015/17 RCP Malfunctions / 4	02						Operational implications of consequences of an RCP failure	4.1	1
W/E10 Natural Circ. / 4	02						Operational implications of EOPs (with a steam void)	3.6	1
000024 Emergency Boration / 1		04					Interrelation between emergency boration and pumps	2.5	1
000026 Loss of Component Cooling Water / 8					05		Determine and interpret CCW flow rates	2.5	1
000029 Anticipated Transient w/o Scram / 1	03						Operational implications of the effect of boron on reactivity	3.8	1
000040 Steam Line Rupture - Excessive Heat Transfer / 4		02					Interrelation between steam break, sensors, and detectors	2.6	1
W/E08 RCS Overcooling - PTS / 4						2.4.18	Knowledge of the specific bases for EOPs	3.6	1
W/E08 RCS Overcooling - PTS / 4	03						Operational implications of indications and remedial actions	4.0	1
PLANT SPECIFIC: Rapid Downpower						Plant	Rapid downpower AOP	N/A	1
000055 Station Blackout / 6					03		Determine and interpret actions necessary to restore power	4.7	1
000057 Loss of Vital AC Elec. Inst. Bus / 6						2.4.11	Knowledge of abnormal condition procedures	3.6	1
PLANT SPECIFIC: Loss of all AC Power Recovery with the SBO Diesel						Plant	Knowledge of ECA-0.3 Loss of All AC Power Recovery with the SBO diesel	N/A	1
000062 Loss of Nuclear Service Water / 4						2.4.4	Recognize entry conditions for AOPs	4.3	1
000067 Plant Fire On-Site / 9						2.4.7	Knowledge of event based EOP mitigation strategies	3.8	1
000068 Control Room Evac. / 8		03					Interrelation between evacuation, controllers and positioners	3.1	1
W/E14 Hi Containment Pressure / 5					01		Interpret conditions and select appropriate procedures	3.8	1
W/E06 Degraded Core Cooling / 4	02						Operational implications of EOPs	4.1	1
000076 High Reactor Coolant Activity / 9	06						Operational implications of chemical shock and crud burst	2.6	1
K/A Category Totals:	9	4	1	0	4	6	Group Point Total:		24

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PWR SRO Examination Outline  
Emergency and Abnormal Plant Evolutions - Tier 1/Group 2

Form ES-401-3

E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	Imp.	Points
000007 Reactor Trip - Stabilization - Recovery / 1			01				Reasons for actions contained in the EOP	4.6	1
000008 Pressurizer Vapor Space Accident / 3	01						Operational implications of thermodynamics and flow characteristics of open or leaking valves	3.7	1
000009 Small Break LOCA / 3	02						Operational implications of use of steam tables	4.2	1
W/E03 LOCA Cooldown/Depress. / 4	01						Operational implications of emergency systems	4.0	1
W/E11 Loss of Emergency Coolant Recirc. / 4				01			Ability to operate or monitor control and safety systems	4.0	1
000025 Loss of RHR System / 4						2.4.24	Knowledge of loss of cooling water procedures	3.7	1
000032 Loss of Source Range NI / 7	01						Operational implications of effects of voltage changes on performance	3.1	1
000033 Loss of Intermediate Range NI / 7					11		Determine and interpret loss of compensating voltage	3.4	1
PLANT SPECIFIC Severe Weather						Plant	Knowledge of Severe Weather AOP	N/A	1
000038 Steam Generator Tube Rupture / 3	02						Operational implications of leak rate vs. pressure drop	3.5	1
000054 Loss of Main Feedwater / 4			03				Reasons for manual control of AFW FCVs	4.1	1
W/E05 Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4			01				Reasons for facility operating characteristics	3.8	1
000058 Loss of DC Power / 6			02				Reasons for actions contained in the EOP	4.2	1
000061 ARM System Alarms / 7						2.4.48	Interpret indications and operator actions	3.8	1
W/E16 High Containment Radiation / 9					02		Adherence to appropriate procedures	3.3	1
000065 Loss of Instrument Air / 8					05		Determine and interpret when to commence plant shutdown	4.1	1
<b>K/A Category Point Totals:</b>	<b>5</b>	<b>0</b>	<b>4</b>	<b>1</b>	<b>3</b>	<b>3</b>	<b>Group Point Total:</b>		<b>16</b>

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PWR SRO Examination Outline  
Emergency and Abnormal Plant Evolutions - Tier 1/Group 3

Form ES-401-3

E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	Imp.	Points
000028 Pressurizer Level Malfunction / 2				08			Operate or monitor selection of an alternate PZR level channel if one has failed	3.6	1
W/E13 Steam Generator Over-pressure / 4	03						Operational implications of conditions and remedial actions	3.2	1
W/E15 Containment Flooding / 5	01						Operational implications of components, capacity, and function of emergency systems	3.0	1
K/A Category Point Totals:	2	0	0	1	0	0	Group Point Total:		3

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PWR SRO Examination Outline  
Plant Systems - Tier 2/Group 1

Form ES-401-3

System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	Imp.	Points
001 Control Rod Drive					97							Operational implications of Tave to Tref	3.6	1
003 Reactor Coolant Pump	01											Connections or cause-effect with Lube oil	2.8	1
004 Chemical and Volume Control		06										Power supply to control instrumentation	2.7	1
004 Chemical and Volume Control						07						Effect of loss or malfunction on heat exch.	2.8	1
013 Engineered Safety Feature Actuation											2.2.22	Knowledge of LCOs and safety limits	4.1	1
014 Rod Position Indication	01											Connections or cause-effect with CRDS	3.6	1
015 Nuclear Instrumentation	08											Cause-effect with RCS (pump start)	2.9	1
015 Nuclear Instrumentation						05						Effect of loss or malf on Audio Indication	2.6	1
017 In-core Temperature Monitor										02		Operate/monitor temperature for RCP start during inadequate core cooling condition	4.1	1
022 Containment Cooling											2.1.32	System limits and precautions	3.8	1
026 Containment Spray			02									Effect of loss or malf on Recirc Spray Sys.	4.3	1
026 Containment Spray									02			Monitor auto ops of cooling water to HX	4.2	1
056 Condensate								05				Predict impact/mitigate condensr tube leak	2.5	1
059 Main Feedwater			03									Effect of loss or malf of MFW on the SGs	3.7	1
061 Auxiliary Feedwater											2.2.24	Analyze effect of maintenance on LCOs	3.8	1
063 DC Electrical Distribution											2.2.22	Knowledge of LCOs and safety limits	4.1	1
068 Liquid Radwaste	05											Connections/cause-effect with CWS/CCWS	2.6	1
071 Waste Gas Disposal										14		Operate or monitor WDGS status alarm	3.0	1
PLANT SPECIFIC: N-16 monitors								P				Mitigate tube leakage based on N-16 data	N/A	1
K/A Category Point Totals:	4	1	2	0	1	2	0	2	1	2	4	Group Point Total:		19

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PWR SRO Examination Outline  
Plant Systems - Tier 2/Group 2

Form ES-401-3

System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	Imp.	Points
006 Emergency Core Cooling								08				Predict/mitigate power loss impact on valves	3.3	1
010 Pressurizer Pressure Control										03		Operate or monitor PORVs/block valves	3.8	1
011 Pressurizer Level Control							04					Predict or monitor changes in Tave	3.3	1
012 Reactor Protection											2.1.12	Ability to apply Tech Specs for a system	4.0	1
016 Non-nuclear Instrumentation								01				Predict/mitigate impact of detector failure	3.1	1
027 Containment Iodine Removal										01		Operate or monitor CIRS controls	3.3	1
028 Hydrogen Recombiner and Purge Control											2.1.32	Explain/apply system limits/precautions	3.8	1
034 Fuel Handling Equipment				01								Design features/interlocks that protect fuel from binding and dropping	3.4	1
035 Steam Generator			02									Effect of SG malfunction on ECCS	4.3	1
039 Main and Reheat Steam							07					Predict or monitor changes in MSS temperature to prevent exceeding limits	2.6	1
055 Condenser Air Removal											2.1.32	Explain/apply system limits/precautions	3.8	1
062 AC Electrical Distribution									01			Ability to monitor vital AC bus amps	3.1	1
064 Emergency Diesel Generator		02										Knowledge of fuel oil pump power supply	3.1	1
073 Process Radiation Monitoring								01				Predict/mitigate impact of failed power sply	2.9	1
075 Circulating Water										01		Operate or monitor emergency SWS pumps	3.2	1
079 Station Air											2.4.11	Knowledge of Abnormal Condition Procedures	3.6	1
086 Fire Protection					04							Personnel hazards from fire type and methods of protection	3.5	1
K/A Category Point Totals:	0	1	1	1	1	0	2	3	1	3	4	Group Point Total:		17

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PWR SRO Examination Outline  
Plant Systems - Tier 2/Group 3

Form ES-401-3

System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G	K/A Topic(s)	Imp.	Points
008 Component Cooling Water										05		Operate or monitor normal CCW flowrates	2.5	1
041 Steam Dump/Turbine Bypass Control				05								Design/interlocks that provided for plant startup	2.7	1
045 Main Turbine Generator					23							Operational implications of relationship between rod control and RCS boron during load increases	2.8	1
078 Instrument Air		01										Knowledge of power supply for the IAS compressor	2.9	1
K/A Category Point Totals:	0	1	0	1	1	0	0	0	0	1	0	Group Point Total:		4

## Plant-Specific Priorities

System / Topic	Recommended Replacement for...	Reason	Points
Rapid Downpower AOP	Tier 1, Gp 1, 051.A2.01	Millstone 3 has plant specific AOP for reducing power at a rate in excess of normal limits. 051.A2.01 is similar to 055.GEN.2.1.32	1
Loss of all AC Power Recovery with the SBO Diesel	Tier 1 Gp 1, 059.A1.02	Millstone 3 uses a plant-specific EOP for recovery from a loss of all AC Power using the SBO Diesel, which is necessary to maintain the core covered during extended SBO events. Selected topic (operate or monitor ARM system) is better tested on the operational portion of the exam.	1
Severe Weather Procedure	Tier 1 Gp 2, 037.A1.04	Millstone 3 has a fairly new SG Tube Leak procedure. Selected topic (operate or monitor the SJAE Rad Monitor) is better tested on the operational portion of the exam.	1
N-16 Detectors	Tier 2 Gp 1, 072.A1.01	Millstone 3 has just installed new N-16 monitors to improve sensitivity to primary to secondary leak detection. Selected topic was similar to 061.GEN.2.4.48 (Interpret ARM indications), which was also selected.	1
EOP User's Guide	Tier 3, GEN.2.4.23	Millstone 3 uses a plant-specific EOP Users' Guide. Selected topic is performed by the Shift Technician.	1
Plant-Specific Priority Total: (limit 10)			5

Facility: Millstone Unit 3		Date of Exam: Week of 08/12/02		Exam Level: SRO	
Category	K/A #	Topic	Imp.	Points	
Conduct of Operations	2.1.1	Knowledge of conduct of operations requirements	3.8	1	
	2.1.7	Evaluate plant performance and make operational judgements	4.4	1	
	2.1.12	Ability to apply technical specifications for a system	4.0	1	
	2.1.13	Facility requirements for controlling vital / controlled access	2.9	1	
	2.1.25	Ability to interpret graphs and tables	3.1	1	
	2.1.33	Recognize indications which are entry-level conditions for technical specifications	4.0	1	
	Total				6
Equipment Control	2.2.11	Knowledge of the process for controlling temporary changes	3.4	1	
	2.2.25	Knowledge of bases in technical specifications for LCOs and safety limits	3.7	1	
	2.2.27	Knowledge of the refueling process	3.5	1	
	2.2.29	Knowledge of SRO fuel handling responsibilities	3.8	1	
Total				4	
Radiation Control	2.3.1	Knowledge of 10CFR: 20 and related facility radiation control requirements	3.0	1	
	2.3.9	Knowledge of the process for performing a containment purge	3.4	1	
	Total				2
Emergency Procedures and Plan	2.4.2	Knowledge of system setpoints, interlocks and automatic actions associated with EOP entry conditions	4.1	1	
	2.4.6	Knowledge of symptom based EOP mitigation strategies	4.0	1	
	2.4.28	Knowledge of procedures relating to emergency response to sabotage	3.3	1	
	2.4.36	Knowledge of chemistry / health physics tasks during emergency operations	2.8	1	
	PLANT	Knowledge of Millstone 3 EOP User's Guide	N/A	1	
	Total				5
Tier 1 Target Point Total (SRO)				17	



Facility: <u>Millstone 3</u>		Date of Examination: <u>August 12-15</u>
Examination Level (circle one):      SRO (U) & (I)		Operating Test Number: <u>A-1</u>
Administrative Topic/Subject Description	Describe method of evaluation: 1. ONE Administrative JPM, OR 2. TWO Administrative Questions	
A.1.1	<b>Shift Turnover Practices</b> (GEN 2.1.3)	JPM SRO - Review Shift Log  (41.10/45.13)
A.1.2	<b>Non-Nuclear Safety Procedures</b> (GEN 2.1.25)	Evaluate Current Decay Heat Removal Requirements  41.10/43.5/45.12)
A.2	<b>Tagging and Clearance Procedures</b> (GEN 2.2.13)	Given a completed clearance and reference material, review, amend and approve the clearance.  (41.10/45.13)
A.3	<b>Review and Approve Release Permits</b> (GEN 2.3.6)	Review and Approve a Radioactive Liquid Waste discharge permit.  (43.4/45.10)
A.4	<b>E-Plan Actions</b> (GEN 2.4.44)	Provide initial Protective Action Recommendations after an event is classified.  (43.5/45.11)

Facility: <u>Millstone Unit 3</u>		Date of Examination: <u>August 7-10, 2002</u>	
Exam Level (circle one): SRO(I)		Operating Test No.: <u>B-1</u>	
<b>B.1 Control Room Systems</b>			
	System / JPM Title	Type Code*	Safety Function
a.	S.01 (#135) Line-Up RHR in the Injection Mode (Respond to an RCS Leak)	M, S, L, E	4.1 - 5
b.	S.02 (#50A) Pressurizer Pressure Control Following Reactor Trip	D, A, E, S, P	3 - 10
c.	S.03 (New) Energize the AC Emergency BUS through the RSSA During ECA-0.0	N, A, E, S	6 - 62
d.	S.04 (#112) Dropped Rod Recovery at Power	D, S	1 - 1
e.	S.05 (#046) Shift AFW Suction to CST (DWST Lo Level)	D, E, S	4.2 - 61
f.	S.06 (#51N) Performance of the Immediate Actions of E-0	D, A, E, S	2 - 13
g.	S.07 (#031) Subsequent Actions in Response to Aux. Bldg. Rad Monitor (3HVR-RE13) Alarm	D, S	7 - 73
<b>B.2 Facility Walk-Through</b>			
a.	P.01 (#137) Establish Feed and Bleed on Charging Pump Cooling (EOP 3501)	D, E, R, L	2 - 4
b.	P.02 (New) Establish Service Water to Train "A" Control Building Chilled Water Cross Tie	N	4.2 - 76
c.	P.03 (#15A) Secondary Side Actions on a Control Room Evacuation Due to a Fire	M, A, E	6 - 62
*Type Codes: (D)irect from bank, (M)odified from bank, (N)ew, (A)lternate path, (C)ontrol room, (S)imulator, (L)ow-Power, (R)CA, (E)OP, (P)revious NRC Exam			

Facility: <u>Millstone Unit 3</u>		Date of Examination: <u>August 7-10, 2002</u>	
Exam Level (circle one): SRO(U)		Operating Test No.: <u>B-1</u>	
<b>B.1 Control Room Systems</b>			
	System / JPM Title	Type Code*	Safety Function
a.	S.01 (#135) Line-Up RHR in the Injection Mode (Respond to an RCS Leak) <i>6/7</i>	M, S, L, E	4.1 - 5
b.	S.02 (#50A) Pressurizer Pressure Control Following Reactor Trip	D, A, E, S, P	3 - 10
<b>B.2 Facility Walk-Through</b>			
a.	P.01 (#137) Establish Feed and Bleed on Charging Pump Cooling (EOP 3501)	D, E, R, L	2 - 4
b.	P.02 (New) Establish Service Water to Train "A" Control Building Chilled Water Cross Tie	N	4.2 - 76
c.	P.03 (#15A) Secondary Side Actions on a Control Room Evacuation Due to a Fire	M, A, E	6 - 62
*Type Codes: (D)irect from bank, (M)odified from bank, (N)ew, (A)lternate path, (C)ontrol room, (S)imulator, (L)ow-Power, (R)CA, (E)OP, (P)revious NRC Exam			

# Initial Dynamic Simulator Scenario

NUREG-1021, Appendix D, Attachment 1

Title: ATWS with Faulted S/G

ID Number: 2K2 NRC-001

Revision: 0

I. Summary:

Facility: Millstone 3 PWR: \_\_\_\_\_ Scenario No: 2K2NRC-001 Op-Test No: 2K2

Examiners: \_\_\_\_\_ Operators: \_\_\_\_\_

Initial Conditions: IC-22; 50% power, end of life

The "A" PORV is out of service due to an electrical short in the control circuitry

Event No:	Malf. No.	Event Type *	Event Description
1		N(US)	Pump the Containment Drains Transfer Tank
2	CC03B	C(RO)	Partial Loss of RCP B Cooling water Supply
3	ED08C	C(ALL)	Loss of Instrument Bus VIAC-3
4	RX09A	I(RO)	Pressurizer Pressure Transmitter PT455 Failure
5	I/Os 32N RP09A/B RP10A/B	M(ALL)	ATWS; 32N Fails to De-energize
6	CV18	C(RO)	Charging Flow Control Failure
7	RP11E	C(RO)	SIH Fails to Align on SIS
8	MS01D	M(ALL)	Main Steam Line D Rupture Inside CTMT
9	RP08	I(US/ BOP)	MSI Fails to Auto Actuate

\*( N )ormal ( R )eactivity ( I )nstrument ( C )omponent ( M )ajor

# Initial Dynamic Simulator Scenario

NUREG-1021, Appendix D, Attachment 1

Title: SGTR WITHOUT PZR PRESSURE CONTROL

ID Number: 2K2 NRC-002

Revision: 0

I. Summary:

Facility: <u>Millstone 3</u> PWR: _____ Scenario No: <u>2K2NRC-002</u> Op-Test No: <u>2K2</u>			
Examiners: _____		Operators: _____	
<u>Initial Conditions:</u> IC-21; 100% power, end of life			
The "A" PORV is out of service due to an electrical short in the control circuitry			
Event No:	Malfunction No.	Event Type *	Event Description
1		N(ALL)	Reduce CTMT Pressure
2	RD10-59	C(RO)	Control Rod Position Data A Failure
3	SG03C	C(ALL)	Steam Generator "C" Tube Leak
4	RD02I	C(RO)	Rod Bank CB-D Continuous Rod Insertion
5	SG01C ED01	M(ALL)	Steam Generator "C" Tube Rupture Loss of Offsite Power
6	SW01B SW02D	C(RO)	B SWP Trips D SWP Fails to Auto Start
7	MS12C	C(BOP)	MSIV on Ruptured S/G Fails to Close
8	I/Os: PCV455 PCV456 IAS*V72	M(ALL)	Inability to De-pressurize RCS

\*( N )ormal ( R )eactivity ( I )nstrument ( C )omponent ( M )ajor

# Initial Dynamic Simulator Scenario

NUREG-1021, Appendix D, Attachment 1

Title: STEAM LINE BREAK AND FR-H.1

ID Number: 2K2 NRC-003

Revision: 0

I. Summary:

Facility: <u>Millstone 3</u>		PWR: _____	Scenario No: <u>2K2NRC-003</u>	Op-Test No: <u>2K2</u>
Examiners: _____		Operators: _____		
<u>Initial Conditions:</u> IC-21; 100% power, end of life				
Event No:	Malfunction No.	Event Type *	Event Description	
1	ANN I/O MB7C, 3-7	C(BOP) R(ALL)	Exciter Field Ground AOP 3575, Rapid Downpower	
2	I/O 3CHS* MV8104	C(RO)	Emergency Boration Valve Fails to Open	
3	RX16A	I(RO / BOP)	Pimp (PT-505) Instrument failed as-is	
4	CC01B	C(RO)	B CCP Pump Trips	
5	ANN I/O MB7C, 4-5	C(BOP)	Generator Core Monitor Level Hi	
6	MS01B	M(ALL)	Trip and Steam Break	
7	FW21A FW20B FW19	C(BOP)	AFW Pumps Fail to Provide Adequate Flow	
8	RP06A RP06B	C(RO)	CDA Fails to Automatically Actuate	
9	3RCS* P1C	C(RO)	RCP Fails to Trip	

\*( N )ormal ( R )eactivity ( I )nstrument ( C )omponent ( M )ajor

# Initial Dynamic Simulator Scenario

NUREG-1021, Appendix D, Attachment 1

Title: INTER-SYSTEM LOCA

ID Number: 2K2NRC-004

Revision: 0

I. Summary:

Facility: <u>Millstone 3</u> PWR: _____ Scenario No: <u>2K2NRC-004</u> Op-Test No: <u>2K2</u>			
Examiners: _____		Operators: _____	
Initial Conditions: <u>IC-21; 100% power, end of life</u>			
Event No:	Malfunction No.	Event Type *	Event Description
1	None	N(RO)	Shift running CHS pumps using OP 3204A, Charging and Letdown, Section 4.4
2	NI09B	I(RO) or R(RO)	Power Range NI Channel 42 Lower Detector Fails High
3	FW13N	C(RO/ BOP) R(ALL)	LP Feedwater 6B Heater Leak
4	RC14B	C(RO) R(ALL)	RPCCW Leak into RCP B Upper Oil Reservoir Rapid Downpower per AOP 3575
5	SI06B	M(ALL)	RCS to SI Inter-System LOCA
6	RP11D	C(ALL)	ESF Bldg Ventilation Fails to Automatically Actuate

\*( N )ormal ( R )eactivity ( I )nstrument ( C )omponent ( M )ajor