

September 18, 2002

Mr. Dale E. Young, Vice President  
Crystal River Nuclear Plant (NA1B)  
ATTN: Supervisor, Licensing and Regulatory Programs  
15760 W. Power Line Street  
Crystal River, Florida 34428-6708

SUBJECT: CRYSTAL RIVER UNIT 3 — REQUEST FOR ADDITIONAL INFORMATION  
RE: PROPOSED LICENSE AMENDMENT REQUEST ON POWER UPRATE  
TO 2568 MWT (TAC NO. MB5289)

Dear Mr. Young:

By letter dated June 5, 2002, you submitted an amendment application to revise the Crystal River Unit 3 (CR-3) Improved Technical Specifications Section 1.1 and License Condition 2.C.(1) to allow operation up to 2568 MWt. The enclosed Request for Additional Information (RAI) was previously discussed with your staff on September 10, 2002.

For the staff to complete its review on schedule, your response to this RAI is requested no later than October 11, 2002. This date was mutually agreed upon in a telephone conversation with Paul Infanger on September 10, 2002. If you need to revise the target date, please call me at the earliest opportunity at 301-415-2020.

Sincerely,

*/RA/*

Brenda L. Mozafari, Senior Project Manager, Section 2  
Project Directorate II  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosure: As stated

cc w/encl: See next page

REQUEST FOR ADDITIONAL INFORMATION

POWER UPRATE TO 2568 MWT

CRYSTAL RIVER UNIT 3

- 1) Section 4.7.2.4 of Attachment A to the amendment request indicates the  $RT_{PTS}$  value at end of license (EOL) for the limiting beltline material, upper shell longitudinal welds WF-18 and WF-8, is 206.0 °F. The attachment also indicates that neutron embrittlement analyses were performed using a 7-percent increase in neutron fluence and the chemical composition reported in Reference 7, BAW-2325, Revision 1, "Response to Request for Additional Information (RAI) Regarding Reactor Pressure Vessel Integrity," January 1999. This report identifies the EOL  $RT_{PTS}$  value for the upper shell longitudinal weld as 215.7 °F, which is greater than the value reported in this amendment. Since the neutron fluence was assumed to increase in this evaluation, the  $RT_{PTS}$  should have increased.

Explain this discrepancy and provide the following information:

- a) the neutron fluence at the clad-base metal interface for the upper shell longitudinal weld at EOL,
  - b) the amounts of copper and nickel for the upper shell longitudinal weld,
  - c) the unirradiated reference temperature for the upper shell longitudinal weld, and
  - d) the margin value utilized in the  $RT_{PTS}$  evaluation for the upper shell longitudinal weld.
- 2) In order to satisfy the reactor vessel Charpy upper-shelf energy requirements of Appendix G, 10 CFR Part 50, Florida Power Corporation has performed equivalent margins analyses for the beltline welds. What is the impact of the power uprate on the equivalent margins analyses? Provide the neutron fluence at the 1/4 thickness at EOL for the beltline welds and describe the impact of this neutron fluence on the equivalent margins analyses.

**ENCLOSURE**

September 18, 2002

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Mr. Dale E. Young  
Florida Power Corporation

**CRYSTAL RIVER UNIT NO. 3**

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