

	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
1 RADIOLOGICAL CONDITIONS	MEASURED EFFLUENTS ① Confirmed sample analysis for gaseous or liquid releases indicates concentrations or release rates greater than Offsite Dose Calculation Manual limits. 1 2 3 4 5 U U-1-a	Confirmed sample analysis for gaseous or liquid releases indicates concentrations or release rates greater than 10 times Offsite Dose Calculation Manual limits. 1 2 3 4 5 D A-1-a		
	AREA & AIRBORNE RADIATION LEVELS Unexpected area or airborne radiation levels 1000 times normal which could require off-site impact assessment. 1 2 3 4 5 U A-1-b			
2 FUEL DAMAGE	REACTOR WATER I-131 Rx water sample analysis exceeds 1.1 µCi/gm I-131 dose equivalent for > 24 hrs. OR Rx water sample analysis reveals an instantaneous spike of 4 µCi/gm I-131 dose equivalent. 1 2 3 U U-2-a	Reactor coolant sample activity > 300 µCi/gm I-131 dose equivalent. 1 2 3 U A-2-a		
	AEQG RADIATION LEVELS Valid AEGD RAD HIHI alarm (3-9-1) which does not clear within 30 minutes. 1 2 3 U U-2-b	Confirmed sample analysis determines that the release rate is ≥ 1.5 Ci/sec. ① 1 2 3 U A-2-b		
3 COOLANT INVENTORY	CONTAINMENT SUMP FLOW Reactor coolant leakage into the primary containment from unidentified sources > 6 gpm OR Total reactor coolant leakage into the primary containment > 25 gpm. 1 2 3 U U-3-a	Total reactor coolant leakage into the primary containment > 60 gpm. 1 2 3 U A-3-a		
	DRYWELL PRESSURE Drywell pressure ≥ 2.5 psig, AND Drywell floor drain sump hi level alarm energized, AND Drywell equipment drain sump hi level alarm energized. 1 2 3 U A-3-b			
	PRIMARY LEAK OUTSIDE CONTAINMENT WITH FAILURE TO ISOLATE Any 2 of the following 4 conditions: • High area temperature AND the appropriate PCIS valves fail to isolate the leak • High area radiation levels AND the appropriate PCIS valves fail to isolate the leak • High flow indications AND the appropriate PCIS valves fail to isolate the leak • Visual confirmation of a line break 1 2 3 4 U S-3-a			
	ON-SITE OR IN-PLANT FIRES Any unplanned on-site or in-plant fire not extinguished within 10 minutes. 1 2 3 4 5 U U-4-a	Any in-plant fire which affects or will likely affect safety system equipment required either: • For continued operation in the current operating mode. • If shutdown, to achieve and maintain cold shutdown conditions. 1 2 3 4 5 U A-4-a	Reactor pressure is > 100 psig and an in-plant fire disables HPCI, RCIC and SRVs OR The plant is not in cold shutdown and an in-plant fire disables both LPCI subsystems A and B AND both Core Spray subsystems A and B OR The plant is not shutdown and an in-plant fire disables EITHER RPS OR CRD (scram function). 1 2 3 U S-4-a	
4 FIRE	ON-SITE OR IN-PLANT FIRES Any unplanned on-site or in-plant fire not extinguished within 10 minutes. 1 2 3 4 5 U U-4-a	Any in-plant fire which affects or will likely affect safety system equipment required either: • For continued operation in the current operating mode. • If shutdown, to achieve and maintain cold shutdown conditions. 1 2 3 4 5 U A-4-a	Reactor pressure is > 100 psig and an in-plant fire disables HPCI, RCIC and SRVs OR The plant is not in cold shutdown and an in-plant fire disables both LPCI subsystems A and B AND both Core Spray subsystems A and B OR The plant is not shutdown and an in-plant fire disables EITHER RPS OR CRD (scram function). 1 2 3 U S-4-a	
	RIVER WATER LEVEL River water level > 230 ft or < 20 ft. 1 2 3 4 5 U U-5-a	River water level > 250 ft or < 20 ft. 1 2 3 4 5 U A-5-a	River water level > 25 ft. AND Flooding which renders safe shutdown equipment inoperable. 1 2 3 4 5 U S-5-a	
	WIND VELOCITY Sustained wind velocity > 70 mph being experienced on-site. 1 2 3 4 5 U U-5-b	Sustained wind velocity > 75 mph being experienced on-site. 1 2 3 4 5 U A-5-b	Sustained wind velocity > 80 mph which renders safe shutdown equipment inoperable. 1 2 3 4 5 U S-5-b	
	EARTHQUAKE Any earthquake sensed on-site as recognized by observation or confirmed detection. 1 2 3 4 5 U U-5-c	Earthquake exceeds the OBE or minor damage to non-nuclear safety class equipment. 1 2 3 4 5 U A-5-c	Earthquake damage to some safety class equipment. 1 2 3 4 5 U S-5-c	
5 NATURAL PHENOMENON	TORNADO Any on-site tornado. 1 2 3 4 5 U U-5-d	Any tornado striking a safety class structure as evidenced by physical damage. 1 2 3 4 5 U A-5-d		

	UNUSUAL EVENT	ALERT	SITE AREA EMERGENCY	GENERAL EMERGENCY
6 LOSS OF POWER	LOSS OF AC POWER CAPABILITY 345KV/115KV bus voltage becomes zero OR Unplanned loss of both startup transformers OR Unplanned loss of capability of both diesel generator supply Bus 3 or 4. 1 2 3 4 5 U U-6-a	No off-site or on-site AC power supply is capable of energizing 4160 volt Bus 3 within 15 minutes. AND No off-site or on-site AC power supply is capable of energizing 4160 volt Bus 4 within 15 minutes. 4 5 U A-6-a	No off-site or on-site AC power supply is capable of energizing 4160 volt Bus 3 within 15 minutes. AND No off-site or on-site AC power supply is capable of energizing 4160 volt Bus 4 within 15 minutes. 1 2 3 S-6-a	No off-site or on-site AC power supply is capable of energizing 4160 volt Bus 3 AND No off-site or on-site AC power supply is capable of energizing 4160 volt Bus 4 AND either: • Power restoration to at least one (Bus 3 or Bus 4) vital bus not likely within 6 hours OR • RPV water level TAF. 1 2 3 U G-6-a
	LOSS OF DC POWER CAPABILITY Both 125 VDC station battery systems 1 2 3 4 5 U A-6-b		Both 125 VDC station battery systems are de-energized. 1 2 3 S-6-b	
	PLANT SHUTDOWN NOT COMPLETED IN ACCORDANCE WITH LCO OF TECH SPECS. Required plant operating mode change or plant shutdown is not completed within the specified time of a Technical Specification action statement. 1 2 3 U U-7-a			
7 LOSS OF SYSTEMS OR EQUIPMENT	LOSS OF CR ALARMS OR INDICATORS Unplanned loss of most or all safety system annunciation or indication (CRP 3-9, 3-4 and 3-5) in the control room for greater than 15 min. 1 2 3 U U-7-b	Unplanned loss of most or all safety system annunciation or indication in the control room for greater than 15 min. AND either: • Plant transient is in progress OR • SPDS is unavailable. 1 2 3 U A-7-a	Unplanned loss of most or all safety system annunciation or indication in the control room AND Plant transient is in progress AND SPDS is unavailable AND Inability to monitor one or more EOP parameters. 1 2 3 S-7-a	
	LOSS OF COMMUNICATION CAPABILITY Loss of all on-site or all off-site communications capability. 1 2 3 4 5 U U-7-c			
	LOSS OF DECAY HEAT REMOVAL CAPABILITY Reactor coolant temperature cannot be maintained below 212 °F. 1 2 3 U A-7-b		Operation in the UNSAFE region of the HCL curve AND The main condenser is unavailable. 1 2 3 S-7-b	Reactor shutdown occurs but requisite decay heat removal systems are unavailable • Operation in the UNSAFE region of the HCL curve AND Main condenser is unavailable AND RHR System is unavailable as a heat sink for either the reactor or containment. 1 2 3 U G-7-a
8 FAILURE TO SCRAM	FAILURE TO SCRAM Automatic or manual SCRAM signal present AND It cannot be determined that the reactor will remain shutdown. 1 2 3 U A-7-c		Either: • Automatic and manual SCRAM signals present OR • Manual SCRAM signal present AND Reactor power remains above 2% 1 2 S-7-c	Automatic or manual SCRAM signal present AND Reactor power remains above 2% AND The main condenser is unavailable AND There exists uncontrolled decreasing reactor water level. 1 2 U G-7-b
	EXPLOSIONS Any near or on-site explosion. 1 2 3 4 5 U U-8-a	Any explosion which results in damage to inplant safety systems or vital structures. 1 2 3 4 5 U A-8-a	Any explosion which renders safe shutdown equipment inoperable. 1 2 3 4 5 U S-8-a	
	CRASH IMPACTS Vehicle crash into safety related structures or systems. 1 2 3 4 5 U U-8-b	Any crash impact which damages safety related structures or systems. 1 2 3 4 5 U A-8-b	Any crash impact which renders safe shutdown equipment inoperable. 1 2 3 4 5 U S-8-b	
9 OTHER HAZARDS OR CONDITIONS	MAN TURBINE FAILURE Turbine failure resulting in casing penetration or damage to turbine or generator seals. 1 2 3 U U-8-c	Turbine casing penetration resulting in damage to safety systems or vital structures. 1 2 3 U A-8-c	Uncontrolled entry of toxic or flammable gas into vital areas of the plant which renders safe shutdown equipment inoperable. 1 2 3 4 5 U S-8-c	
	RELEASE OF HAZARDOUS OR TOXIC SUBSTANCES Actual or potential release of hazardous substances which may affect personal safety and plant operation. 1 2 3 4 5 U U-8-d	Uncontrolled entry of toxic or flammable gas into vital areas of the plant which renders safe shutdown equipment inoperable. 1 2 3 4 5 U A-8-d	Uncontrolled entry of toxic or flammable gas into vital areas of the plant which renders safe shutdown equipment inoperable. 1 2 3 4 5 U S-8-c	
	EVACUATION OF THE CONTROL ROOM Security threat or credible security threat notification with potential to affect safe plant operation. 1 2 3 4 5 U U-9-a	On-going Security threat in progress which has the potential of affecting safe plant operation. 1 2 3 4 5 U A-9-a	Determination that loss of physical control of the plant has occurred. 1 2 3 4 5 U S-9-a	Determination that loss of physical control of the plant has occurred. 1 2 3 4 5 U G-9-a
10 SECURITY EVENTS	SAFE PLANT OPERATION THREATENED Security threat or credible security threat notification with potential to affect safe plant operation. 1 2 3 4 5 U U-9-a	On-going Security threat in progress which has the potential of affecting safe plant operation. 1 2 3 4 5 U A-9-a	Determination that loss of physical control of the plant has occurred. 1 2 3 4 5 U S-9-a	Determination that loss of physical control of the plant has occurred. 1 2 3 4 5 U G-9-a
	LOSS OF PHYSICAL CONTROL OF THE PLANT Other conditions exist which have potential to degrade safe plant operation which warrant plant staff notification and increased awareness for off-site authorities. 1 2 3 4 5 U U-10-a	Other events or conditions which warrant precautionary activation of the Technical Support Center and the Emergency Operations Facility. 1 2 3 4 5 U A-10-a	Events are in progress or have occurred which warrant activation of emergency centers AND either: • Activation of off-site monitoring teams OR • Precautionary notification of the public near site has occurred or is planned. 1 2 3 4 5 U S-10-a	Events are in progress or have occurred which warrant initiation of predetermined protective action for the public. 1 2 3 4 5 U G-10-a

NOTES	
①	To be determined by sample analysis. The following may be used to indicate when analysis should be performed: Gaseous: • Stack Gas Rad Mon HI/Alert Alarms referenced by parameters • 3-G-7: STACK GAS RAD MON SYS 1 TRBL • 3-G-8: STACK GAS RAD MON SYS 2 TRBL Liquid: • Parameters • 3-E-7: RADWASTE EFFLUENT RAD HI • 3-F-7: DISC CANAL RAD HI • 3-F-8: SERVICE WATER EFFLUENT RAD HI AEGD: • Parameter • 3-D-1: AEGD RAD HI - HI
②	Integrated dose to be determined by either: • Direct field monitoring OR • Projected off-site dose correlation.
③	If limits are exceeded, then protective action recommendations should be made per OP 351-1.
④	Per VYNS Safe Shutdown Capability Analysis Figure 2-3, Principal Safe Shutdown Systems are: • RPS (CRD SCRAM) • SRVs • Core Spray • RHR • HPCI • RCIC • RHR Service Water • Service Water
⑤	Earthquake detection criteria based on receipt of alarm from the seismic monitoring system confirmed by data analysis, multiple indications, observation, or seismic activity reports.
⑥	See DP3127, Appendix A "Seismic Damage Indicator Walkdown Check Sheet - SDI List 1 (NNS Components)".
⑦	See DP3127, Appendix A "Seismic Damage Indicator Walkdown Check Sheet - SDI List 2 (Safety Related Components)".
⑧	Each of the following is a means for determining that the reactor will remain shutdown: • all control rods are inserted to or beyond position 02 • all control rods are inserted to position 00 except one rod • TSC or RE has determined that sufficient control rod density exists.
⑨	Notification and response for Security Events at this level are performed in accordance with the Physical Security Plan also.

	COOLANT ACTIVITY	RPV WATER LEVEL	DRYWELL RADIATION	DRYWELL PRESSURE	RCS LEAKAGE	PRIMARY CONTAINMENT ISOLATION	COMBUSTIBLE GAS	GENERAL
FUEL CLAD Potential Loss		Uncontrolled decreasing RPV water level AND RPV water level < TAF	Containment radiation monitors reading > 1000 R/hr.					Any condition which indicates LOSS or POTENTIAL LOSS of the FUEL CLAD barrier.
RCS Potential Loss	Reactor coolant sample activity > 300 µCi/gm I-131 Dose Equivalent				Reactor coolant system leak rate > 90 gpm inside the drywell OR Unisolable reactor coolant leakage outside the drywell as indicated by high reactor building area temperatures or radiation levels.			Any condition which indicates LOSS or POTENTIAL LOSS of the REACTOR COOLANT SYSTEM barrier.
PRIMARY CONTAINMENT Potential Loss		RPV water level < 48 inches.	Containment radiation monitors reading > 10 R/hr.	Drywell pressure > 2.5 psig AND indication of a reactor coolant system leak inside the drywell.	Any primary coolant safety or safety relief valve fails to close.			Explosive hydrogen/oxygen mixture exists within either the drywell or toxic air space. (i.e., ≥ 6% H2 and ≥ 6% O2 mixture)
			Containment radiation monitors reading > 5000 R/hr.	Drywell pressure > 42 psig and increasing.		Intentional venting per EOPs or SAGs OR Failure of two in-series PCIS valves to isolate a release pathway OR Observed structural failure OR Unisolable reactor coolant leakage outside the drywell as indicated by high reactor building area temperatures or radiation levels.		Any condition which indicates LOSS or POTENTIAL LOSS of the PRIMARY CONTAINMENT barrier.

MODE APPLICABILITY: 1 Power Operations 2 StartUp/Hot Standby 3 Hot Shutdown 4 Cold Shutdown 5 Refuel D Defueled