

August 28, 2002

Mr. John T. Conway
Site Vice President
Nine Mile Point Nuclear Station, LLC
P. O. Box 63
Lycoming, NY 13093

SUBJECT: NINE MILE POINT NUCLEAR STATION, UNIT NO. 1 - ISSUANCE OF
AMENDMENT RE: SNUBBER INSPECTION SCHEDULE (TAC NO. MB4427)

Dear Mr. Conway:

The Commission has issued the enclosed Amendment No. 175 to Facility Operating License No. DPR-63 for the Nine Mile Point Nuclear Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated March 15, 2002.

The amendment revised Section 4.6.4, "Shock Suppressors (Snubbers)," following the guidance of Generic Letter 90-09, "Alternative Requirements for Snubber Visual Inspection and Corrective Actions," dated December 11, 1990.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

/RA/

Peter S. Tam, Senior Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-220

Enclosures: 1. Amendment No. 175 to DPR-63
2. Safety Evaluation

cc w/encls: See next page

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Accession Number: ML022420114

OFFICE	PDI-1\PM	PDI-1\LA	EMEB\SC	OGC	PDI-1\SC
NAME	PTam	SLittle#	KManoly	RHoefling	RLaufer**
DATE	8/9/02	8/27/02	7/18/02*	8/20/02	8/20/02

*SE transmitted by memo of 7/18/02 and no major changes were made.

#Concurred by phone. **P. Milano concurred for R. Laufer.

OFFICIAL RECORD COPY

DATED: August 28, 2002

AMENDMENT NO. 175 TO FACILITY OPERATING LICENSE NO. DPR-63 NINE MILE POINT
UNIT NO. 1

PUBLIC
PDI R/F
RLaifer
SLittle
PTam
OGC
GHill (2)
WBeckner
ALee
ACRS
BPlatchek, RI

cc: Plant Service list

NINE MILE POINT NUCLEAR STATION, LLC (NMPNS)

DOCKET NO. 50-220

NINE MILE POINT NUCLEAR STATION UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 175
License No. DPR-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Nine Mile Point Nuclear Station, LLC (the licensee) dated March 15, 2002, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-63 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, which is attached hereto, as revised through Amendment No. 175, is hereby incorporated into this license. Nine Mile Point Nuclear Station, LLC shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented prior to the spring 2003 refueling outage.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA by PMilano for RLaufer/

Richard J. Laufer, Chief, Section I
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 28, 2002

ATTACHMENT TO LICENSE AMENDMENT NO. 175

TO FACILITY OPERATING LICENSE NO. DPR-63

DOCKET NO. 50-220

Replace the following pages of Appendix A, Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages

259
260
261
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Insert Pages

259
260
261
263a

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 175 TO FACILITY OPERATING LICENSE NO. DPR-63
NINE MILE POINT NUCLEAR STATION, LLC (NMPNS)
NINE MILE POINT NUCLEAR STATION, UNIT NO. 1
DOCKET NO. 50-220

1.0 INTRODUCTION

By letter dated March 15, 2002, Nine Mile Point Nuclear Station, LLC (NMPNS or the licensee) submitted a request for changes to the Nine Mile Point Nuclear Station, Unit No. 1 (NMP1), Technical Specifications (TSs). The requested changes would revise Section 4.6.4, "Shock Suppressors (Snubbers)," consistent with the snubber visual inspection and acceptance requirements of the model requirements included in Generic Letter (GL) 90-09, "Alternative Requirements for Snubber Visual Inspection and Corrective Actions," dated December 11, 1990.

2.0 REGULATORY EVALUATION

The Nuclear Regulatory Commission (NRC) staff finds that the licensee identified the applicable regulatory requirements in the the Background section, Attachment B of the March 15, 2002, application. The licensee stated that the current NMP1 snubber TSs requirements were issued on September 23, 1985, via Amendment No. 74. This amendment revised NMP1 Sections 3.6.4 and 4.6.4 to be consistent with the intent of the model snubber requirements included in GL 84-13, "Technical Specification for Snubbers," dated May 3, 1984. Subsequently, the NRC issued GL 90-09, which provided alternative requirements for snubber visual inspection intervals. In the March 15, 2002, application, the licensee requested a revision of the NMP1 TSs to reflect the alternate requirements included in GL 90-09.

3.0 TECHNICAL EVALUATION

The existing NMP1 Section 4.6.4.a.(i) specifies a schedule for snubber visual inspections that is based on the number of inoperable snubbers found during the previous visual inspection, regardless of the size of the snubber population inspected. Because of this, inspection intervals could be significantly shortened due to findings in a small portion of the snubbers inspected. To alleviate this generic industry concern with snubber visual inspections, the NRC issued GL 90-09, which provided an alternate schedule for visual inspections that maintain the same confidence level as the existing schedule and will allow performance of snubber visual inspections and corrective actions during refueling outages. The licensee stated in its March 15, 2002, application that the proposed change revises Section 4.6.4 to incorporate the snubber visual inspection surveillance schedule and acceptance criteria contained in GL 90-09. The alternate inspection schedule is based on the number of unacceptable snubbers found during the previous inspection in proportion to the sizes of snubber populations or categories.

Based on GL 90-09 guidance and the successful results of recent snubber inspections at NMP1, the licensee stated that the adoption of the alternate snubber inspection schedule is justified and has significant benefit for NMP1 while maintaining the same level of confidence in plant safety as provided by the existing requirements. The NRC staff has reviewed the licensee's application and finds that it provides an acceptable basis for adopting the alternate inspection schedule of snubbers. The NRC staff also concurs with the licensee that, with the alternate inspection schedule, favorable results of snubber visual inspection can lead to reduction of the inspection frequency from every refueling outage to every other refueling outage. This reduction will save outage manpower and reduce occupational exposure associated with snubber visual inspections.

The licensee stated that certain variations from the GL 90-09 model were necessary or desirable while incorporating the requirements of the model into the NMP1 existing TSs. These are described and evaluated as follows.

1. Currently, the Applicability section of Section 4.6.4 refers to "periodic testing requirement" for snubbers while the Specification portion of Section 4.6.4 refers to the "inspection program" for snubbers. The two sections have been revised to clarify that both periodic inspection and periodic testing requirements for snubbers are included in Section 4.6.4. This clarification also serves to show that the term "inspection program" as used in Section 3.6.3 (which is not being revised) encompasses both inspection and testing requirements. The NRC staff finds this variation to involve only an administrative change, and is, therefore, acceptable.
2. The model TS Section 4.7.9 reference to Generic Specification 4.0.5 has not been incorporated into the Specification portion of TSs Section 4.6.4. Generic Specification 4.0.5 applies to surveillance requirements for inservice inspection and testing of American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) Class 1, 2, and 3 components. The inservice inspection portion of NMP1 TSs Section 4.2.6 (equivalent to that in Generic Specification 4.0.5) has already been deleted from the TSs by Amendment No. 173, dated August 5, 2002. Additionally, the reference to inservice testing of ASME Code Class 1, 2, and 3 components is unnecessary as the ASME Code-required inservice testing is implemented per 10 CFR 50.55a. Therefore, the reference to Generic Specification 4.0.5 need not be incorporated. The NRC staff finds this variation to be acceptable.
3. The definition of "type of snubber" found in Section 4.7.9.a of the model TSs has been incorporated into Section 4.6.4.a.(i), "Visual Inspection Frequency," and is not presented as a separate item. The NRC staff finds this variation to be administrative, and is acceptable.
4. The statement in model TS 4.7.9.b concerning determination of the first inspection interval using the new visual inspection acceptance criteria of model TS Table 4.7-2 has not been incorporated into Section 4.6.4.a.(i). The new visual inspection criteria were already included in the procedure utilized for the last visual inspection of the snubbers. Therefore, the proposed frequency criteria can be adopted upon receiving approval of the proposed amendment and there is no need to include the statement from model TS 4.7.9.b. The NRC staff finds this variation to be administrative, and is acceptable.

5. The model TS table containing snubber visual inspection intervals (Table 4.7-2) includes snubber populations up to "1000 or greater." The equivalent NMP1 table (Table 4.6.4-1) extends only to snubber populations up to 200. This population encompasses all the snubbers that are subject to visual inspection per Section 4.6.4, regardless of type or category. This variation reflects the as-built conditions of NMP1, and is acceptable.
6. Note 1 of Table 4.6.4-1 is an editorially revised version of Note 1 of the equivalent model TS Table 4.7-2. The NRC staff agrees that this variation is editorial, and acceptable.
7. Note 6 of model TSs Table 4.7-2 references Generic Specification 4.0.2, which addresses the maximum allowable extension in the specified time intervals associated with TS surveillance requirements. Note 6 of the equivalent NMP1 Table 4.6.4-1 references Specification 4.0.1, which is also equivalent to Generic Specification 4.0.2. The NRC staff finds this variation to be editorial, and is acceptable.

On the basis of the information reviewed, the NRC staff concludes that the licensee's proposed alternate snubber inspection schedule is acceptable. This conclusion is based on the fact that the proposed schedule, which is similar to the one incorporated in the GL 90-09, ensures the same confidence level as the existing schedule.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (67 FR 18645). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The NRC staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the

Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: A. Lee

Date: August 28, 2002

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Unit No. 1

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