

August 30, 2002

Mr. Mike Bellamy
Site Vice President
Entergy Nuclear Operations, Inc.
Pilgrim Nuclear Power Station
600 Rocky Hill Road
Plymouth, MA 02360

SUBJECT: PILGRIM NUCLEAR POWER STATION - REQUEST FOR ADDITIONAL
INFORMATION RE: APPENDIX K MEASUREMENT UNCERTAINTY
RECOVERY - POWER UPRATE REQUEST (TAC NO. MB5603)

Dear Mr. Bellamy:

By letter dated July 5, 2002, Entergy Nuclear Operations, Inc., submitted a license amendment request for proposed changes to the Operating License and Technical Specifications associated with an increase in the licensed power level for Pilgrim Nuclear Power Station from 1,998 MWt to 2,028 MWt (1.5%).

The U.S. Nuclear Regulatory Commission staff is reviewing your submittal and has determined that additional information is required to complete the review. The specific information requested is addressed in the enclosure to this letter and was discussed with Mr. Steve Brennon of your staff during a telephone call on August 21, 2002. In a follow up call on August 22, 2002, Mr. Brennon agreed that your staff will respond to the requested information within 30 days from the date of this letter. If circumstances result in the need to revise the response date, please contact me at (301) 415-8474.

Sincerely,

/RA/

Travis L. Tate, Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-293

Enclosure: Request for Additional Information

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION

APPENDIX K MEASUREMENT UNCERTAINTY RECOVERY - POWER UPRATE REQUEST

PILGRIM NUCLEAR POWER STATION

DOCKET NO. 50-293

1. Section 9.2 of Attachment 2 to the licensing amendment application dated July 5, 2002, addresses Design Basis Accidents (DBA). The submittal states that radiological consequences due to postulated DBAs have been calculated using a source term of 102% of the current licensed reactor thermal power. However, the staff finds that the analyses addressed in the Updated Final Safety Analysis Report (UFSAR) appears to indicate that the radiological consequences may have been determined at the current rated thermal power. Please provide discussion on the analyses/evaluation addressed in Section 9.2, identifying changes, if any, to the assumptions, methodologies, and accidents analyzed from those documented in the UFSAR. Please provide the results and acceptance criteria of the analyses referenced in Section 9.2.
2. Provide details about the grid stability analysis including assumptions and results and conclusions for the power uprate condition. Also, provide in detail (including the ratings) the effect of the power uprate on the following equipment:
 - a. Main generator
 - b. Isophase bus
 - c. Main power transformer
 - d. Startup transformer
 - e. Auxiliary transformer
 - f. Non-Class 1E loads increased

Enclosure

Pilgrim Nuclear Power Station

cc:

Resident Inspector
U. S. Nuclear Regulatory Commission
Pilgrim Nuclear Power Station
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Massachusetts Emergency Management
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