

March 6, 1998

Mr. Gregory M. Rueger, Senior Vice President
and General Manager
Pacific Gas and Electric Company
Nuclear Power Generation N9B
P.O. Box 770000
San Francisco, California 94177

SUBJECT: ISSUANCE OF AMENDMENTS FOR DIABLO CANYON NUCLEAR POWER
PLANT, UNIT NO. 1 (TAC NO. M97472) AND UNIT NO. 2 (TAC NO. M97473)

Dear Mr. Rueger:

On February 17, 1998, the Commission issued Amendment No. 122 to Facility Operating License No. DPR-80 and Amendment No. 120 to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, respectively. The amendments revised the combined Technical Specifications (TS) to change the surveillance frequencies from at least once every 18 months to at least once per refueling interval (nominally 24 months) for the reactor trip system (RTS) and engineering safety features actuation systems (ESFAS) instrumentation channels, and also made changes in trip setpoints and allowance values due to a setpoint methodology change in support of the calibration extensions. Channel operational tests (COTs) and trip actuating device operational tests (TADOTs) associated with these channels were also extended. This was in response to your application dated December 9, 1996.

Due to an administrative error, previous amendment numbers were omitted from page 2-4 of the technical specifications. Enclosed is a corrected page 2-4. The overleaf page is also provided to maintain document completeness. Also, your staff identified a discrepancy on page 11 of the safety evaluation. Enclosed is a corrected page 11. We apologize for any inconvenience this may have caused.

Sincerely,
Original Signed By

Steven D. Bloom, Project Manager
Project Directorate IV-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. 50-275
and 50-323

Enclosures: 1. TS Page 2-4
2. Page 11 of SE

cc w/encls: See next page

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NRC Resident Inspector
Diablo Canyon Nuclear Power Plant
c/o U.S. Nuclear Regulatory Commission
P. O. Box 369
Avila Beach, California 93424

Dr. Richard Ferguson, Energy Chair
Sierra Club California
1100 11th Street, Suite 311
Sacramento, California 95814

Ms. Nancy Culver
San Luis Obispo
Mothers for Peace
P. O. Box 164
Pismo Beach, California 93448

Chairman
San Luis Obispo County Board of
Supervisors
Room 370
County Government Center
San Luis Obispo, California 93408

Mr. Truman Burns
Mr. Robert Kinosian
California Public Utilities Commission
505 Van Ness, Room 4102
San Francisco, California 94102

Mr. Steve Hsu
Radiologic Health Branch
State Department of Health Services
Post Office Box 942732
Sacramento, California 94232

Diablo Canyon Independent Safety
Committee
ATTN: Robert R. Wellington, Esq.
Legal Counsel
857 Cass Street, Suite D
Monterey, California 93940

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
Harris Tower & Pavillion
611 Ryan Plaza Drive, Suite 400
Arlington, Texas 76011-8064

Christopher J. Warner, Esq.
Pacific Gas & Electric Company
Post Office Box 7442
San Francisco, California 94120

Mr. Robert P. Powers
Vice President and Plant Manager
Diablo Canyon Nuclear Power Plant
P. O. Box 56
Avila Beach, California 93424

Telegram-Tribune
ATTN: Managing Editor
1321 Johnson Avenue
P.O. Box 112
San Luis Obispo, California 93406

2.2 LIMITING SAFETY SYSTEM SETTINGS

REACTOR TRIP SYSTEM INSTRUMENTATION SETPOINTS

2.2.1 The Reactor Trip System Instrumentation and Interlock Setpoints shall be set consistent with the Trip Setpoint values shown in Table 2.2-1.

APPLICABILITY: As shown for each channel in Table 3.3-1.

ACTION:

- a. With a Reactor Trip System Instrumentation or Interlock Setpoint less conservative than the value shown in the Trip Setpoint column but more conservative than the value shown in the Allowable Values column of Table 2.2-1, adjust the Setpoint consistent with the Trip Setpoint value.
- b. With a Reactor Trip System Instrumentation or Interlock Setpoint less conservative than the value shown in the Allowable Values column of Table 2.2-1, declare the channel inoperable and apply the applicable ACTION statement requirements of Specification 3.3.1 until the channel is restored to OPERABLE status with its Setpoint adjusted consistent with the Trip Setpoint value.

TABLE 2.2-1

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNIT	TRIP SETPOINT	ALLOWABLE VALUES
1. Manual Reactor Trip	N.A.	N.A.
2. Power Range, Neutron Flux a. Low Setpoint b. High Setpoint	$\leq 25\%$ of RATED THERMAL POWER $\leq 109\%$ of RATED THERMAL POWER	$\leq 26.2\%$ of RATED THERMAL POWER $\leq 110.2\%$ of RATED THERMAL POWER
3. Power Range, Neutron Flux High Positive Rate	$\leq 5\%$ of RATED THERMAL POWER with a time constant ≥ 2 seconds	$\leq 5.6\%$ of RATED THERMAL POWER with a time constant ≥ 2 seconds
4. Power Range, Neutron Flux High Negative Rate	$\leq 5\%$ of RATED THERMAL POWER with a time constant ≥ 2 seconds	$\leq 5.6\%$ of RATED THERMAL POWER with a time constant ≥ 2 seconds
5. Intermediate Range, Neutron Flux	$\leq 25\%$ of RATED THERMAL POWER	$\leq 30.6\%$ of RATED THERMAL POWER
6. Source Range, Neutron Flux	$\leq 10^5$ counts per second	$\leq 1.4 \times 10^5$ counts per second
7. Overtemperature ΔT	See Note 1	See Note 2
8. Overpower ΔT	See Note 3	See Note 4
9. Pressurizer Pressure-Low	≥ 1950 psig	≥ 1947.5 psig
10. Pressurizer Pressure-High	≤ 2385 psig	≤ 2387.5 psig
11. Pressurizer Water Level-High	$\leq 90\%$ of instrument span	$\leq 90.2\%$ of instrument span
12. Reactor Coolant Flow-Low	$\geq 90\%$ of minimum measured flow** per loop	$\geq 89.8\%$ of minimum measured flow** per loop

**Minimum measured flow is 89,800 gpm per loop for Unit 1 and 90,625 gpm per loop for Unit 2.

DIABLO CANYON - UNITS 1 & 2
 2-4
 Unit 1 - Amendment No. 37, 72, 84, 122
 Unit 2 - Amendment No. 36, 71, 83, 120

2. Table 2.2-1 and Table 3.3-4, T Notes 1 and 3, replace word "controller" with "compensator."
3. Table 2.2-1, Over temperature Delta T and Overpower Delta T Notes 1 and 3 clarification. Add words "loop specific" in front of word "indicated" in note 1 and note 3, and add words "loop specific Indicated" between words Nominal and T_{avg} in note 1.
4. Table 2.2-1, Overpower Delta T Note 3 clarification. Add words, "Nominal Loop specific" in front of the word "Indicated".
5. TS 3/4.3.1, Table 4.3-1, Reactor Trip System Instrumentation Surveillance Requirements, Functional Unit 13.b - Steam Generator Water Level, Low-Low, RCS Loop Delta T input. To clarify functional unit description add words "Equivalent to Power" after words Delta T.
6. TS 3/4.3.2, Table 4.3-2, Engineered Safety Features Actuation System Instrumentation Surveillance Requirements, Functional Unit 6.c.2) - Auxiliary Feedwater, Steam Generator Water Level- Low-Low, RCS Loop Delta T Input. To clarify functional unit description, add words "Equivalent to Power" after words Delta T.

Justification for the Change

The proposed changes to the above footnotes involve a spelling correction, terminology change and clarifying language and have no safety significance. In their submittal, the licensee stated that the term "compensator" is proposed in place of "controller" to better describe signal processing in the instrument racks. The term "loop specific" is used to better define the processing that occurs for each individual RCS loop. The term "equivalent to power" is added for two items to maintain consistency in the TS for all items which refer to RCS loop Delta T equivalent to power. These changes are administrative and editorial in nature, and do not affect the design, operation, or testing of the plant. These changes are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the California State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the