

Docket No. 50-362

April 9, 1990

Mr. Harold B. Ray
Vice President
Southern California Edison Co.
Irvine Operations Center
23 Parker Street
Irvine, California 92718

Mr. Gary D. Cotton
Senior Vice President
Engineering and Operations
San Diego Gas & Electric Co.
101 Ash Street
San Diego, California 92112

Gentlemen:

SUBJECT: ISSUANCE OF AMENDMENT NO.75 TO FACILITY OPERATING LICENSE NO. NPF-15
- SAN ONOFRE NUCLEAR GENERATING STATION, UNIT NO. 3 (TAC NO. 75937)

The Commission has issued the enclosed amendments to Facility Operating License No. NPF-15 for San Onofre Nuclear Generating Station, Unit No. 3. The amendment consist of changes to the Technical Specifications in response to your application dated February 12, 1990. This request was designated by you as PCN 317.

This amendment revises Surveillance Requirement 4.8.1.1.2.d of Technical Specification 3/4.8.1., "A.C. Sources." The change permits a one time extension during Cycle 4 operation from 24 months to 25 months of the surveillance interval for Surveillance Requirement 4.8.1.1.2.d, which requires certain maintenance and testing activities be performed.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Finally, administrative corrections have been made to the Technical Specifications for San Onofre Unit 2 on page 3/4 3-33, and for Unit 3 on pages 3/4 3-69, 3/4 3-70, 3/4 3-71, 3/4 3-74, and 3/4 3-75. These pages, and the appropriate overleaf pages, are enclosed. We regret the inconvenience that this may have caused your staff.

Sincerely,

ORIGINAL SIGNED BY L. KOKAJKO
Lawrence E. Kokajko, Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc w/enclosures:
See next page

*See previous concurrence

DRSP/PD5*
PShea
3/8/90

DRSP/PD5*
LKokajko:dr
3/2/90

OGC*
CBarth
3/15/90

for (A)DRSP/D:PD5
CTramme11
4/9/90

[S02/3 AMD 75937]

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Lawrence E. Kokajko, Project Manager
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DRSP/PD5*
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DRSP/PD5*
LKokajko:dr
3/2/90

OGC*
3/15/90

(A)DRSP/D:PD5
CTramme11
4/ /90

[S02/3 AMD 75937]

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See next page

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LKokajko:dr
3/13/90

OGC *CB*
3/15/90

(A) DRSP/D:PD5
CTrammell
4/14/90

[S02/3 AMD 75937]

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G. Hill (4)

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ACRS (10)

GPA/PA

OC/LFMB

PD5 Plant File

Region V (4)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 9, 1990

Docket No. 50-362

Mr. Harold B. Ray
Vice President
Southern California Edison Co.
Irvine Operations Center
23 Parker Street
Irvine, California 92718

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Senior Vice President
Engineering and Operations
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Sincerely,

A handwritten signature in cursive script, appearing to read "Lawrence E. Kokajko".

Lawrence E. Kokajko, Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
As stated

cc w/enclosures:
See next page

Messrs Ray and Cotton
Southern California Edison Company

San Onofre Nuclear Generating
Station, Units 2 and 3

cc:

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San Diego, California 92101



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

DOCKET NO. 50-362

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 75
License No. NPF-15

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the license for San Onofre Nuclear Generating Station, Unit 3 (the facility) filed by Southern California Edison Company (SCE) on behalf of itself and San Diego Gas and Electric Company, the City of Riverside, California and the City of Anaheim, California (licensees) dated February 12, 1990 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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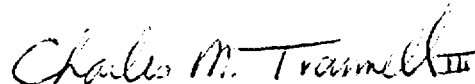
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. NPF-15 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 75, are hereby incorporated in the license. SCE shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and must be fully implemented no later than 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Charles M. Trammell III, Acting Director
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 9, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 75

FACILITY OPERATING LICENSE NO. NPF-15

DOCKET NO. 50-362

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed page. The revised page is identified by amendment number and contain marginal lines indicating the area of change. Also enclosed is the following overleaf page to the amended page.

AMENDMENT PAGE

3/4 8-3

OVERLEAF PAGE

3/4 8-4

ELECTRICAL POWER SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

5. Verifying the generator is synchronized, loaded to greater than or equal to 4700 kW in less than or equal to 77 seconds*, and operates with a load greater than or equal to 4700 kW for at least an additional 60 minutes, and
 6. Verifying the diesel generator is aligned to provide standby power to the associated emergency busses.
- b. At least once per 31 days and after each operation of the diesel where the period of operation was greater than or equal to 1 hour by checking for and removing accumulated water from the day tank.
- c.1. At least once per 92 days and from new fuel prior to addition to the storage tanks by verifying that a sample obtained in accordance with ASTM-D4057-81 has a water and sediment content of less than or equal to .05 volume percent and a kinematic viscosity @ 40°C of greater than or equal to 1.9 but less than or equal to 4.1 when tested in accordance with ASTM-D975-81.
2. At least once every 92 days by obtaining a sample of fuel oil in accordance with ASTM-D4057-81 and verifying that particulate contamination is less than 10mg/liter when checked in accordance with ASTM D2276-83, Method A.
- d. At least once per refueling interval** (the provisions of Technical Specification 4.0.2 are not applicable) by:
1. Subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for this class of standby service.
 2. Verifying the generator capability to reject a load of greater than or equal to 655.7 kW while maintaining voltage at 4360 ± 436 volts and frequency at 60 ± 6.0 Hz.
 3. Verifying the generator capability to reject a load of 4700 kW without tripping. The generator voltage shall not exceed 5450 volts during and following the load rejection.

*All engine starts for the purpose of this surveillance testing may be preceded by an engine prelube period and/or other warmup procedures recommended by the manufacturer so that mechanical stress and wear on the diesel engine is minimized.

**For Cycle 4 only, this surveillance interval may exceed the 24-month refueling interval but may not exceed 25 months.

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

4. Simulating a loss of offsite power by itself, and
 - a) Verifying de-energization of the emergency busses and load shedding from the emergency busses.
 - b) Verifying the diesel starts on the auto-start signal, energizes the emergency busses with permanently connected loads within 10 seconds and operates for greater than or equal to 5 minutes while its generator is loaded with the permanently connected loads. After energization, the steady state voltage and frequency of the emergency busses shall be maintained at 4360 ± 436 volts and 60 ± 1.2 Hz during this test.
5. Verifying that on an ESF test signal, without loss of offsite power, the diesel generator starts on the auto-start signal and operates on standby for greater than or equal to 5 minutes. The steady-state generator voltage and frequency shall be 4360 ± 436 volts and 60 ± 1.2 Hz within 10 seconds after the auto-start signal; the generator voltage and frequency shall be maintained within these limits during this test.
6. Deleted
7. Simulating a loss of offsite power in conjunction with an ESF test signal, and
 - a) Verifying de-energization of the emergency busses and load shedding from the emergency busses.
 - b) Verifying the diesel starts on the auto-start signal, energizes the emergency busses with permanently connected loads within 10 seconds, energizes the auto-connected emergency (accident) loads through the load sequence and operates for greater than or equal to 5 minutes while its generator is loaded with the emergency loads. After loading, the steady state voltage and frequency of the emergency busses shall be maintained at 4360 ± 436 volts and $60 + 1.2/-0.3$ Hz during this test.
 - c) Verifying that all automatic diesel generator trips, except engine overspeed, generator differential, and low-low lube oil pressure, are automatically bypassed.

TABLE 4.3-2 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
11. FUEL HANDLING ISOLATION (FHIS)				
a. Manual (Trip Buttons)	N.A.	N.A.	R	N.A.
b. Airborne Radiation				
i. Gaseous	S	R	M	*
ii. Particulate/Iodine	S	R	M	*
c. Automatic Actuation Logic	N.A.	N.A.	R(3)	*
12. CONTAINMENT PURGE ISOLATION (CPIS)				
a. Manual (Trip Buttons)	N.A.	N.A.	(6)	N.A.
b. Airborne Radiation				
i. Gaseous	S	(6)	M	1,2,3,4,6
ii. Particulate	W	(6)	M	1,2,3,4,6
iii. Iodine	W	(6)	M	6
c. Containment Area Radiation (Gamma)	S	(6)	M	1,2,3,4,6
d. Automatic Actuation Logic	N.A.	N.A.	(3), (6)	1,2,3,4,6

TABLE NOTATION

- (1) Each train or logic channel shall be tested at least every 62 days on a STAGGERED TEST BASIS.
- (2) Deleted.
- (3) Testing of Automatic Actuation Logic shall include energization/de-energization of each initiation relay and verification of the OPERABILITY of each initiation relay.
- (4) A subgroup relay test shall be performed which shall include the energization/de-energization of each subgroup relay and verification of the OPERABILITY of each subgroup relay. Relays exempt from testing during plant operation shall be limited to only those relays associated with plant equipment which cannot be operated during plant operation. Relays not testable during plant operation shall be tested during each COLD SHUTDOWN exceeding 24 hours unless tested during the previous 6 months.
- (5) Actuated equipment only; does not result in CIAS.
- (6) At least once per refueling interval.
- * With irradiated fuel in the storage pool.

INSTRUMENTATION

3/4.3.3 MONITORING INSTRUMENTATION

RADIATION MONITORING INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.1 The radiation monitoring instrumentation channels shown in Table 3.3-6 shall be OPERABLE with their alarm/trip setpoints within the specified limits.

APPLICABILITY: As shown in Table 3.3-6.

ACTION:

- a. With a radiation monitoring channel alarm/trip setpoint exceeding the value shown in Table 3.3-6, adjust the setpoint to within the limit within 4 hours or declare the channel inoperable.
- b. With one or more radiation monitoring channels inoperable, take the ACTION shown in Table 3.3-6.
- c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.1 Each radiation monitoring instrumentation channel shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL CALIBRATION and CHANNEL FUNCTIONAL TEST operations for the MODES and at the frequencies shown in Table 4.3-3.

TABLE 4.3-9 (Continued)

TABLE NOTATION

**During waste gas holdup system operation (treatment for primary system offgases).

- (1) The CHANNEL CALIBRATION shall include the use of standard gas samples containing a nominal:
 1. One volume percent hydrogen, balance nitrogen, and
 2. Four volume percent hydrogen, balance nitrogen.

- (2) The CHANNEL CALIBRATION shall include the use of standard gas samples containing a nominal:
 1. One volume percent oxygen, balance nitrogen, and
 2. Four volume percent oxygen, balance nitrogen.

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INSTRUMENTATIONLOOSE-PART DETECTION INSTRUMENTATIONLIMITING CONDITION FOR OPERATION

3.3.3.10 The loose-part detection system shall be OPERABLE.

APPLICABILITY: MODES 1 and 2.

ACTION:

- a. With one or more loose part detection system channels inoperable for more than 30 days, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 10 days outlining the cause of the malfunction and the plans for restoring the channel(s) to OPERABLE status.
- b. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.10 Each channel of the loose-part detection system shall be demonstrated OPERABLE by performance of a:

- a. CHANNEL CHECK at least once per 24 hours,
- b. CHANNEL FUNCTIONAL TEST at least once per 31 days, and
- c. CHANNEL CALIBRATION at least once per refueling interval.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 75 TO FACILITY OPERATING LICENSE NO. NPF-15

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS AND ELECTRIC COMPANY

THE CITY OF RIVERSIDE, CALIFORNIA

THE CITY OF ANAHEIM, CALIFORNIA

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT NO. 3

DOCKET NO. 50-362

1.0 INTRODUCTION

By letter dated February 12, 1990, Southern California Edison Company, et al. (the licensee) requested a change to the Technical Specifications for Facility Operating License No. NPF-15 that authorizes operation of San Onofre Nuclear Generating Station, Unit No. 3 in San Diego County, California. The licensee requested to revise Surveillance Requirement 4.8.1.1.2.d of Technical Specification 3/4.8.1., "A.C. Sources." The proposed change would permit a one time extension during Cycle 4 operations from 24 months to 25 months of the surveillance interval for Surveillance Requirement 4.8.1.1.2.d, which requires certain maintenance and testing activities be performed. These activities would be performed once per refueling interval, which is nominally 24 months.

2.0 EVALUATION

The licensee requests this one time, one month surveillance interval extension during Cycle 4 operation in order to avoid a potentially significant impact on the schedule for the Cycle 5 refueling outage, which is scheduled to begin during April 1990. The surveillance requirement under review requires extensive maintenance and testing be performed on the diesel generators on a refueling interval (nominal 24 months) basis. These activities include teardown and inspection, taking from 4 to 6 weeks to complete.

Schedule impact could occur since the specification surveillance interval for the "B" diesel generator could expire before the surveillance for "A" diesel generator can be completed. The possibility exists for the "B" diesel generator to be declared administratively inoperable while the "A" diesel generator is inoperable for surveillance activities. If this occurred, both diesel generators would be considered inoperable, causing refueling activities (fuel movement) to cease. This administrative problem could occur as early as May 17, 1990. The licensee has requested the one month surveillance extension to preclude this possibility.

The licensee states that the effect of a one month extension on a 24 month interval would be minimal. This extension increases the surveillance interval approximately 4 percent. All other diesel generator surveillances remain on their original schedules and serve to assure continued diesel generator operability. Since the diesel generator must run at least once every 31 days, operability problems are generally identified during this run rather than during the refueling interval maintenance activities. Moreover, other operability checks help ensure that the diesel generator remains operable. Additionally, the licensee states that there have been only three valid test failures out of over 260 valid test runs, indicating a 99 percent success record.

The staff has determined that this extension would not significantly affect the capability of the diesel generators to perform their intended function. The diesel generators are run at least every 31 days, which provides a high degree of confidence that they are operable. Also, the staff agrees that the Cycle 5 refueling outage schedule could be adversely affected by an administratively inoperable diesel generator. A one month extension of the refueling surveillance interval for inspection and testing would have no adverse effect upon the public health and safety. Therefore, based upon the information presented above, the staff approves the one time, one month extension during Cycle 4 operation of the Surveillance Requirement 4.8.1.1.2.d, effectively changing the surveillance interval from 24 to 25 months.

3.0 CONTACT WITH STATE OFFICIAL

The staff has advised the State Department of Health Services, State of California, of the proposed determination of no significant hazards consideration. No comments were received.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment involve changes to requirements with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or changes an inspection or surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

We have concluded, based on the considerations discussed above that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; (2) such activities will be conducted in compliance with the Commission's regulations; and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Lawrence E. Kokajko

Dated: April 9, 1990