

Docket Nos. 50-361
and 50-362

October 11, 1991

M

Mr. Harold B. Ray
Senior Vice President
Southern California Edison Co.
Irvine Operations Center
23 Parker Street
Irvine, California 92718

Mr. Gary D. Cotton
Senior Vice President
Engineering and Operations
San Diego Gas & Electric Co.
101 Ash Street
San Diego, California 92112

Gentlemen:

SUBJECT: CORRECTION TO AMENDMENT NOS. 99 AND 88 FOR THE SAN ONOFRE NUCLEAR
GENERATING STATION UNIT NOS. 2 AND 3 (TAC NOS. M80613 AND M80614)

On October 1, 1991, the Commission issued Amendment Nos. 99 and 88 to Facility
Operating License Nos. NPF-10 and NPF-15 for the San Onofre Nuclear
Generating Station, Unit Nos. 2 and 3, in response to your application dated
June 17, 1991 (PCN-339).

The amendments revised Technical Specification (TS) 3/4.7.1.2 and its
associated Bases to identify that the Auxiliary Feedwater System performs a
dual function in an event which requires steam generator isolation and
secondary heat removal. TS pages 3/4 7-4 for both units contained a
typographical error at 3.7.1.2.1.c.3. A manual crosstie valve was referred to
as number "130MU635." The correct reference is "1305MU635."

Corrected TS pages 3/4 7-4 with their backup pages are enclosed. Please
accept our apologies for any inconvenience this error may have caused you.

Sincerely,

Original Signed by:
Lawrence E. Kokajko, Project Manager
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosure:
TS pages 3/4 7-4

cc w/enclosure:
See next page

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OFC	:LA:PD5:DRPW	:PM:PD5:DRPW	:D:PD5:DRPW	:	:
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~~9110310059~~ L. Kokajko

DPD
Mr. Gmel.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

October 11, 1991

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Sincerely,

A handwritten signature in black ink, appearing to read "L. E. Kokajko".

Lawrence E. Kokajko, Project Manager
Project Directorate V
Division of Reactor Projects III/IV/V
Office of Nuclear Reactor Regulation

Enclosure:
TS pages 3/4 7-4

cc w/enclosure:
See next page

Messrs. Ray and Cotton
Southern California Edison Company

San Onofre Nuclear Generating
Station, Unit Nos. 2 and 3

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PLANT SYSTEMS

AUXILIARY FEEDWATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.1.2.1 At least three independent steam generator auxiliary feedwater pumps and associated flow paths shall be OPERABLE with:

- a. Two motor-driven auxiliary feedwater pumps, each capable of being powered from separate emergency busses,
- b. One steam turbine-driven auxiliary feedwater pump capable of being powered from an OPERABLE steam supply system, and
- c. Manual valves in the correct position and automatic valves each capable of being opened and closed, with the following exceptions:
 1. Motor-driven auxiliary feedwater pump discharge bypass control valves, HV-4762 and HV-4763, need only be capable of being closed,
 2. Steam turbine-driven auxiliary feedwater pump steam supply isolation valves, HV-8200 and HV-8201, and turbine stop valve, HV-4716, need only be capable of being opened, and
 3. Manual crosstie valves 1305MU634 and 1305MU635 may be open in Mode 3 provided a minimum of 2 hours has elapsed since reactor shutdown.

APPLICABILITY: MODES 1, 2 and 3.

ACTION:

- a. With one auxiliary feedwater pump or its associated flow path inoperable, restore the required auxiliary feedwater pump and its associated flow path to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.
- b. With either two auxiliary feedwater pumps, two flow paths, or one pump and one separate flow path inoperable, be in at least HOT STANDBY within 6 hours and in HOT SHUTDOWN within the following 6 hours.
- c. With three auxiliary feedwater pumps or flow paths inoperable, immediately initiate corrective action to restore at least one auxiliary feedwater pump and its associated flow path to OPERABLE status as soon as possible.
- d. With an automatic valve in any flow path incapable of closing upon receipt of a Main Steam Isolation Signal, close the affected valve or its block valve within 4 hours and enter actions a, b, or c if there is a loss of the flow path(s). Testing pursuant to Technical Specification 3.3.2 does not constitute entry into this ACTION Statement.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS

4.7.1.2.1.1 Each auxiliary feedwater pump and associated flow path shall be demonstrated OPERABLE:

- a. At least once per 31 days by:
 1. Testing the turbine driven pump and both motor driven pumps pursuant to Specification 4.0.5. The provisions of Specification 4.0.4 are not applicable for the turbine driven pump for entry into MODE 3.
 2. Verifying that each valve (manual, power operated or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.
 3. Verifying that both manual valves in the suction lines from the primary AFW supply tank (condensate storage tank T-121) to each AFW pump, and the manual discharge line valve of each AFW pump are locked in the open position.
 4. Verifying that the AFW piping is full of water by venting the accessible discharge piping high points.
- b. At least once per refueling interval during shutdown by:
 1. Verifying that each automatic valve in the flow path actuates to its correct position upon receipt of an EFAS test signal.
 2. Verifying that each motor driven pump starts and the steam inlet valves to the turbine driven pump open automatically upon receipt of an EFAS test signal.
 3. Verifying that each automatic valve in the flow path actuates to its correct position upon receipt of a MSIS test signal except for HV-8200 and HV-8201.

4.7.1.2.1.2 The auxiliary feedwater system shall be demonstrated OPERABLE prior to entering MODE 2 following each COLD SHUTDOWN by performing a flow test to verify the normal flow path from the primary AFW supply tank (condensate storage tank T-121) through each auxiliary feedwater pump to its associated steam generator.

PLANT SYSTEMS

AUXILIARY FEEDWATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.1.2.1 At least three independent steam generator auxiliary feedwater pumps and associated flow paths shall be OPERABLE with:

- a. Two motor-driven auxiliary feedwater pumps, each capable of being powered from separate emergency busses,
- b. One steam turbine-driven auxiliary feedwater pump capable of being powered from an OPERABLE steam supply system, and
- c. Manual valves in the correct position and automatic valves each capable of being opened and closed, with the following exceptions:
 1. Motor-driven auxiliary feedwater pump discharge bypass control valves, HV-4762 and HV-4763, need only be capable of being closed,
 2. Steam turbine-driven auxiliary feedwater pump steam supply isolation valves, HV-8200 and HV-8201, and turbine stop valve, HV-4716, need only be capable of being opened, and
 3. Manual crosstie valves 1305MU634 and 1305MU635 may be open in Mode 3 provided a minimum of 2 hours has elapsed since reactor shutdown.

APPLICABILITY: MODES 1, 2 and 3.

ACTION:

- a. With one auxiliary feedwater pump or its associated flow path inoperable, restore the required auxiliary feedwater pump and its associated flow path to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.
- b. With either two auxiliary feedwater pumps, two flow paths, or one pump and one separate flow path inoperable, be in at least HOT STANDBY within 6 hours and in HOT SHUTDOWN within the following 6 hours.
- c. With three auxiliary feedwater pumps or flow paths inoperable, immediately initiate corrective action to restore at least one auxiliary feedwater pump and its associated flow path to OPERABLE status as soon as possible.
- d. With an automatic valve in any flow path incapable of closing upon receipt of a Main Steam Isolation Signal, close the affected valve or its block valve within 4 hours and enter actions a, b, or c if there is a loss of the flow path(s). Testing pursuant to Technical Specification 3.3.2 does not constitute entry into this ACTION Statement.

TABLE 3.7-2

MAXIMUM ALLOWABLE VALUE LINEAR POWER LEVEL-HIGH TRIP WITH INOPERABLE
MAIN STEAM SAFETY VALVES DURING OPERATION WITH BOTH STEAM GENERATORS

<u>Maximum Number of Inoperable Safety Valves on Any Operating Steam Generator</u>	<u>Maximum Allowable Value Linear Power Level-High Trip (Percent of RATED THERMAL POWER)</u>
1	98.6
2	86.3
3	74.0
4	61.6