

June 4, 1999

Mr. Gregory M. Rueger
Senior Vice President and General Manager
Pacific Gas and Electric Company
Diablo Canyon Nuclear Power Plant
P. O. Box 3
Avila Beach, CA 93424

SUBJECT: CORRECTION TO AMENDMENT NO. 129 TO FACILITY OPERATING
LICENSE NO. DPR-80 (TAC NO. M98283) AND AMENDMENT NO. 127 TO
FACILITY OPERATING LICENSE NO. DPR-82 (TAC NO. M98284)

Dear Mr. Rueger:

On February 19, 1999, the Commission issued Amendment No. 129 to Facility Operating License No. DPR-80 and Amendment No. 127 to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, respectively. These amendments were in response to your application dated March 10, 1997, as supplemented by letters dated May 20, 1997; March 13, August 28, and October 22, 1998; and January 29 and February 2, 1999.

The amendments revised TS 3/4.4.5 and its associated Bases to allow the implementation of steam generator (SG) tube alternate repair criteria for axial indications in the Westinghouse explosive tube expansion (WEXTEX) region below the top of the tubesheet and below the bottom of the WEXTEX transition that may exceed the current TS depth-based plugging limit.

Page 3/4 4-11 of the technical specifications contained an incorrect paragraph number under Section 4.4.5.2; a number 2 appeared instead of a number 1. A corrected page is enclosed and its corresponding overleaf page to maintain document completeness. We apologize for any inconvenience this may have caused.

Sincerely,

/s/

Steven D. Bloom, Project Manager, Section 2
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-275
and 50-323

Enclosure: Page 3/4 4-11

cc w/encl: See next page

DISTRIBUTION:

Docket WBeckner
PUBLIC GHill (4)
PDIV-2 Reading OGC
JZwolinski/SBlack ACRS
SRichards LSmith, Region IV
SBloom
EPeyton
LHurley, Region IV
JKilcrease, Region IV

DF01

9906140090 990604
PDR ADDCK 05000275
P PDR

Document Name: G:\DIABLO\LTR98283.WPD

OFC	PDIV-2	PDIV-2	OGC NLO	PDIV-2
	EPeyton	SBloom	LHurley	SDembek
DATE	5/20/99	5/21/99	5/24/99	5/19/99

OFFICIAL RECORD COPY

NRC FILE CENTER COPY

6/4/99



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 4, 1999

Mr. Gregory M. Rueger
Senior Vice President and General Manager
Pacific Gas and Electric Company
Diablo Canyon Nuclear Power Plant
P. O. Box 3
Avila Beach, CA 93424

SUBJECT: CORRECTION TO AMENDMENT NO. 129 TO FACILITY OPERATING
LICENSE NO. DPR-80 (TAC NO. M98283) AND AMENDMENT NO. 127 TO
FACILITY OPERATING LICENSE NO. DPR-82 (TAC NO. M98284)

Dear Mr. Rueger:

On February 19, 1999, the Commission issued Amendment No. 129 to Facility Operating License No. DPR-80 and Amendment No. 127 to Facility Operating License No. DPR-82 for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, respectively. These amendments were in response to your application dated March 10, 1997, as supplemented by letters dated May 20, 1997; March 13, August 28, and October 22, 1998; and January 29 and February 2, 1999.

The amendments revised TS 3/4.4.5 and its associated Bases to allow the implementation of steam generator (SG) tube alternate repair criteria for axial indications in the Westinghouse explosive tube expansion (WEXTEX) region below the top of the tubesheet and below the bottom of the WEXTEX transition that may exceed the current TS depth-based plugging limit.

Page 3/4 4-11 of the technical specifications contained an incorrect paragraph number under Section 4.4.5.2; a number 2 appeared instead of a number 1. A corrected page is enclosed and its corresponding overleaf page to maintain document completeness. We apologize for any inconvenience this may have caused.

Sincerely,

A handwritten signature in black ink, appearing to read "S.D. Bloom".

Steven D. Bloom, Project Manager, Section 2
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-275
and 50-323

Enclosure: Page 3/4 4-11

cc w/encl: See next page

Diablo Canyon Power Plant, Units 1 and 2

cc w/encl:

**NRC Resident Inspector
Diablo Canyon Nuclear Power Plant
c/o U.S. Nuclear Regulatory Commission
P. O. Box 369
Avila Beach, California 93424**

**Dr. Richard Ferguson, Energy Chair
Sierra Club California
1100 11th Street, Suite 311
Sacramento, California 95814**

**Ms. Nancy Culver
San Luis Obispo
Mothers for Peace
P. O. Box 164
Pismo Beach, California 93448**

**Chairman
San Luis Obispo County Board of
Supervisors
Room 370
County Government Center
San Luis Obispo, California 93408**

**Mr. Truman Burns
Mr. Robert Kinosian
California Public Utilities Commission
505 Van Ness, Room 4102
San Francisco, California 94102**

**Mr. Steve Hsu
Radiologic Health Branch
State Department of Health Services
Post Office Box 942732
Sacramento, California 94232**

**Diablo Canyon Independent Safety
Committee
ATTN: Robert R. Wellington, Esq.
Legal Counsel
857 Cass Street, Suite D
Monterey, California 93940**

**Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
Harris Tower & Pavillion
611 Ryan Plaza Drive, Suite 400
Arlington, Texas 76011-8064**

**Christopher J. Warner, Esq.
Pacific Gas & Electric Company
Post Office Box 7442
San Francisco, California 94120**

**Mr. David H. Oatley, Vice President
Diablo Canyon Operations and
Plant Manager
Diablo Canyon Nuclear Power Plant
P.O. Box 3
Avila Beach, California 93424**

**Telegram-Tribune
ATTN: Managing Editor
1321 Johnson Avenue
P.O. Box 112
San Luis Obispo, California 93406**

REACTOR COOLANT SYSTEM

3/4.4.5 STEAM GENERATORS

LIMITING CONDITION FOR OPERATION

3.4.5 Each steam generator shall be OPERABLE.*#

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one or more steam generators inoperable, restore the inoperable generator(s) to OPERABLE status prior to increasing T_{avg} above 200°F.

SURVEILLANCE REQUIREMENTS

4.4.5.0 Each steam generator shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirement of Specification 4.0.5.

4.4.5.1 Steam Generator Sample Selection and Inspection - Each steam generator shall be determined OPERABLE during shutdown by selecting and inspecting at least the minimum number of steam generators specified in Table 4.4-1.

4.4.5.2 Steam Generator Tube Sample Selection and Inspection - The steam generator tube minimum sample size, inspection result classification, and the corresponding action required shall be as specified in Table 4.4-2. The inservice inspection of steam generator tubes shall be performed at the frequencies specified in Specification 4.4.5.3 and the inspected tubes shall be verified acceptable per the acceptance criteria of Specification 4.4.5.4. The tubes selected for each inservice inspection shall include at least 3% of the total number of tubes in all steam generators; the tubes selected for these inspections shall be selected on a random basis except:

- a. Where experience in similar plants with similar water chemistry indicates critical areas to be inspected, then at least 50% of the tubes inspected shall be from these critical areas;
- b. The first sample of tubes selected for each inservice inspection (subsequent to the preservice inspection) of each steam generator shall include:
 - 1) All nonplugged tubes that previously had detectable wall penetrations (greater than 20%).

* - Amendment Nos. 129 and 127 applicable for Units 1 and 2, Cycles 10 and 11 only.

- In-situ testing will be performed in accordance with PG&E letter DCL-98-148 dated October 22, 1998.

DIABLO CANYON - UNITS 1 & 2

3/4 4-11

Unit 1 - Amendment No. 129

Unit 2 - Amendment No. 127

9906140095 990604
PDR ADOCK 05000275
P PDR

SURVEILLANCE REQUIREMENTS (Continued)

- 2) Tubes in those areas where experience has indicated potential problems.
 - 3) A tube inspection (pursuant to Specification 4.4.5.4a.8) shall be performed on each selected tube. If any selected tube does not permit the passage of the eddy current probe for a tube inspection, this shall be recorded and an adjacent tube shall be selected and subjected to a tube inspection.
 - 4) Indications left in service as a result of application of the tube support plate voltage-based repair criteria shall be inspected by bobbin coil probe during all future refueling outages.
 - 5) Tubes identified as W* tubes having a previously identified indication within the W* length shall be inspected using a rotating pancake coil (RPC) probe for the full length of the W* region during all future refueling outages.
- c. The tubes selected as the second and third samples (if required by Table 4.4-2) during each inservice inspection may be subjected to a partial tube inspection provided:
- 1) The tubes selected for these samples include the tubes from those areas of the tube sheet array where tubes with imperfections were previously found, and
 - 2) The inspections include those portions of the tubes where imperfections were previously found.
- d. Implementation of the steam generator tube/tube support plate repair criteria requires a 100% bobbin coil inspection for hot-leg and cold-leg tube support plate intersections down to the lowest cold-leg tube support plate with known outside diameter stress corrosion cracking (ODSCC) indications. The determination of the lowest cold-leg tube support plate intersections having ODSCC indications shall be based on the performance of at least a 20% random sampling of tubes inspected over their full length.

The results of each sample inspection shall be classified into one of the following three categories:

<u>Category</u>	<u>Inspection Results</u>
C-1	Less than 5% of the total tubes inspected are degraded tubes and none of the inspected tubes are defective.
C-2	One or more tubes, but not more than 1% of the total tubes inspected are defective, or between 5% and 10% of the total tubes inspected are degraded tubes.
C-3	More than 10% of the total tubes inspected are degraded tubes or more than 1% of the inspected tubes are defective.
Note:	In all inspections, previously degraded tubes must exhibit significant (greater than 10%) further wall penetrations to be included in the above percentage calculations.